

JUN 14 1979

Docket Nos. 50-338  
and 50-339

Mr. W. L. Proffitt  
Senior Vice President - Power  
Operations  
Virginia Electric & Power Company  
P. O. Box 26666  
Richmond, Virginia 23261

Dear Mr. Proffitt:

SUBJECT: ISSUANCE OF AMENDMENT NO. 11 TO FACILITY OPERATING LICENSE NO.  
NPF-4 - NORTH ANNA POWER STATION, UNIT NO. 1

The Nuclear Regulatory Commission (the Commission) has issued the enclosed  
Amendment No. 11 to Facility Operating License NPF-4.

Amendment No. 11 incorporates a new condition in NPF-4 concerning surveillance  
requirements of the emergency diesel generator batteries. The amendment also  
revises Appendices A and B to the Technical Specifications reflecting a re-  
organization of the Production Operation and Maintenance Department both  
onsite and offsite as requested by you. The revised Technical Specification  
page changes are attached to the license amendment.

Our Safety Evaluation supporting these changes and a copy of the Federal  
Register Notice concerning issuance of Amendment No. 11 are also enclosed.

Sincerely,

Original Signed By  
O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
Division of Project Management

Enclosures:

1. Amendment No. 11 with revised pages of Appendices A & B
2. Safety Evaluation
3. Federal Register Notice

CP 1

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cc w/enclosures:  
See next page

OFFICE →	LWR #3:LA	LWR #3:LPM	OELD <i>DL</i>	LWR #3:BC	
SURNAME →	MRushbrock/LN	ADomerick	DSwanson	ODParr	
DATE →	5/25/79	5/25/79	5/27/79	6/14/79	

Mr. W. L. Proffitt

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JUN 14 1979

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JUN 14 1972

Mr. W. L. Proffitt

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Mr. J. B. Jackson, Jr.  
Commonwealth of Virginia  
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903 9th Street Office Building  
Richmond, Virginia 23129



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

FACILITY OPERATING LICENSE

License No. NPF-4  
Amendment No. 11

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The applications for amendment to the North Anna Power Station, Unit No. 1 license, NPF-4, by Virginia Electric and Power Company (licensee), dated August 14, 1978 and August 31, 1978, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to Appendices A & B as indicated in the attachment to this license amendment.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 11, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. The license is further amended by adding condition 2.D(3)o. to read as follows:

2.D(3)o. The provisions of Specification 4.0.4 are not applicable to the performance of surveillance activities associated with the diesel generator battery technical specification (4.8.1.1.3) until the completion of the initial surveillance interval associated with that specification.

4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Olan D. Parr*  
Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
Division of Project Management

Date of Issuance: **JUN 14 1979**

Enclosure:  
Revised pages to Appendices A & B  
Technical Specifications

VIRGINIA ELECTRIC AND POWER COMPANY

DOCKET NO. 50-338

NORTH ANNA POWER STATION, UNIT NO. 1

FACILITY OPERATING LICENSE

License No. NPF-4  
Amendment No. 11

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OFFICE >						
SURNAME >						
DATE >						

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4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
Division of Project Management

Date of Issuance: JUN 14 1979

Enclosure:  
Revised pages to Appendices A & B  
Technical Specifications

LWR- DSE/EP-2

\*See Previous Yellow for Concurrences

\*WRegan 5/30/79

OFFICE	LWR-3:LA	LWR-3:LPM	DOR	AD/EP	OELD	LWR-3:EP
SURNAME	M. Bush Cook	*A. Dromerick	*Drinkman	*VMoore	*D. Swanson	ODParr
DATE	6/14/79	5/29/79	5/30/79	5/30/79	5/31/79	6/11/79

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FOR THE NUCLEAR REGULATORY COMMISSION

Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
Division of Project Management

Date of Issuance:

Enclosure:

Revised pages to Appendices A & B  
Technical Specifications

\*SEE PREVIOUS YELLOW FOR CONCURRENCES

LWR #3:LPM  
\*ADromerick  
5/29 /79

OFFICE	LWR #3:LA	DSE/EP-2	AD/EP	DOR	OEL	LWR #3:BC
SURNAME	MRush <i>MR</i> /LM	*WRegan	*VMoore	DBrinkman	DSwanson	ODParr
DATE	5/30/79	5/30/79	5/30/79	5/30/79	5/31/79	5/ /79

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4. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Roger S. Boyd, Director  
Division of Project Management  
Office of Nuclear Reactor Regulation

Date of Issuance:

Enclosure:

Revised pages to Appendices A & B  
Technical Specifications

LWR #3:PM  
ADomerick  
5/29/79

DSE/EP  
WRegan  
5/30/79

DOR  
D. Brinkman  
5/ /79

OFFICE	LWR #3:LA	AD/EP	OELD D F	LWR #3:BC	LWR:AD	D:DPM
SURNAME	MRushDook LM	V Moore	DSwanson	ODParr	DBVassallo	RSBoyd
DATE	5/25/79	5/30/79	5/31/79	5/ /79	5/ /79	5/ /79

ATTACHMENT TO LICENSE AMENDMENT NO. 11

FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Pages

3/4 8-3a (was 3/4 8-4)

3/4 8-4 (added)

3/4 8-5.

6-2

6-3

6-5

6-6

6-8

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

3. Verifying the fuel transfer pump can be started and transfers fuel from the storage system to the day tank.
  4. Verifying the diesel starts from ambient condition and accelerates to at least 900 rpm in  $\leq 10$  seconds.
  5. Verifying the generator is synchronized, loaded to  $\geq 1375$  kw, and operates for  $\geq 60$  minutes, and
  6. Verifying the diesel generator is aligned to provide standby power to the associated emergency busses.
- b. At least once per 92 days by verifying that a sample of diesel fuel from the fuel storage tank is within the acceptable limits specified in Table 1 of ASTM D975-74 when checked for viscosity, water and sediment.
- c. At least once per 18 months during shutdown by:
1. Subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for this class of standby service,
  2. Verifying the generator capability to reject a load of  $\geq 610$  kw without tripping,
  3. Simulating a loss of offsite power in conjunction with a safety injection test signal, and:
    - a) Verifying de-energization of the emergency busses and load shedding from the emergency busses.
    - b) Verifying the diesel starts from ambient condition on the auto-start signal, energizes the emergency busses with permanently connected loads, energizes the auto-connected emergency loads through the timers and operates for  $\geq 5$  minutes while its generator is loaded with the emergency loads.

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- c) Verifying that all diesel generator trips, except engine overspeed, generator differential and breaker overcurrent are automatically bypassed upon loss of voltage on the emergency bus and/or safety injection actuation signal.
- 4. Verifying the diesel generator operates for  $\geq 60$  minutes while loaded to  $\geq 2750$  kw.
- 5. Verifying that the auto-connected loads to each diesel generator do not exceed the 2000 hour rating of 3000 kw.
- 6. Verifying that the load sequencing times are within the tolerances shown in Table 4.8-1.

4.8.1.1.3 Each diesel generator 125-volt battery bank and charger shall be demonstrated OPERABLE:

- a. At least once per 7 days by verifying that:
  - 1. The electrolyte level of each pilot cell is between the minimum and maximum level indication marks,
  - 2. The pilot cell specific gravity, corrected to 77°F and full electrolyte level, is  $\geq 1.200$ ,
  - 3. The pilot cell voltage is  $\geq 2.08$  volts, and
  - 4. The overall battery voltage is  $\geq 125$  volts.
- b. At least once per 92 days by verifying that:
  - 1. The voltage of each connected cell is  $\geq 2.08$  volts under float charge and has not decreased more than 0.05 volts from the value observed during the previous test,
  - 2. The specific gravity, corrected to 77°F and full electrolyte level, of each connected cell is  $\geq 1.200$  and has not decreased more than 0.08 from the value observed during the previous test, and

## ELECTRICAL POWER SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

3. The electrolyte level of each connected cell is between the minimum and maximum level indication marks.
- c. At least once per 18 months, by verifying that:
1. The cells, cell plates and battery racks show no visual indication of physical damage or abnormal deterioration.
  2. The cell-to-cell and terminal connections are clean, tight and coated with anti-corrosion material.
  3. The resistance of each cell-to-cell and terminal connection is  $\leq 0.01$  ohms.
  4. The battery charger will supply at least 10 amperes at 125 volts for at least 4 hours.
- d. At least once per 60 months, during shutdown, by verifying that the battery capacity is at least 80% of the manufacturer's rating when subjected to a performance discharge test.

## ELECTRICAL POWER SYSTEMS

### SHUTDOWN

### LIMITING CONDITION FOR OPERATION

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3.8.1.2 As a minimum, the following A.C. electrical power sources shall be OPERABLE:

- a. One circuit between the offsite transmission network and the onsite Class 1E distribution system, and
- b. One diesel generator with:
  1. A day tank containing a minimum volume of 750 gallons of fuel,
  2. A fuel storage system containing a minimum volume of 45,000 gallons of fuel, and
  3. A fuel transfer pump.

APPLICABILITY: MODES 5 and 6.

### ACTION:

With less than the above minimum required A.C. electrical power sources OPERABLE, suspend all operations involving CORE ALTERATIONS or positive reactivity changes until the minimum required A.C. electrical power sources are restored to OPERABLE status.

### SURVEILLANCE REQUIREMENTS

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4.8.1.2 The above required A.C. electrical power sources shall be demonstrated OPERABLE by the performance of each of the Surveillance Requirements of 4.8.1.1.1, 4.8.1.1.2 (except for requirement 4.8.1.1.2a.5), and 4.8.1.1.3

ELECTRICAL POWER SYSTEMS

3/4.8.2 ONSITE POWER DISTRIBUTION SYSTEMS

A.C. DISTRIBUTION - OPERATING

LIMITING CONDITION FOR OPERATION

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3.8.2.1 The following A.C. electrical busses shall be OPERABLE and energized with tie breakers open between redundant busses:

4160	volt Emergency Bus # 1H
4160	volt Emergency Bus # 1J
480	volt Emergency Bus # 1H, 1H1
480	volt Emergency Bus # 1J, 1J1
120	volt A.C. Vital Bus # 1-I
120	volt A.C. Vital Bus # 1-II
120	volt A.C. Vital Bus # 1-III
120	volt A.C. Vital Bus # 1-IV

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With less than the above complement of A.C. busses OPERABLE, restore the inoperable bus to OPERABLE status within 8 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

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4.8.2.1 The specified A.C. busses shall be determined OPERABLE with tie breakers open between redundant busses at least once per 7 days by verifying correct breaker alignment and indicated power availability.

## 6.0 ADMINISTRATIVE CONTROLS

### 6.1 RESPONSIBILITY

6.1.1 The Station Manager shall be responsible for overall facility operation and shall delegate in writing the succession to this responsibility during his absence.

### 6.2 ORGANIZATION

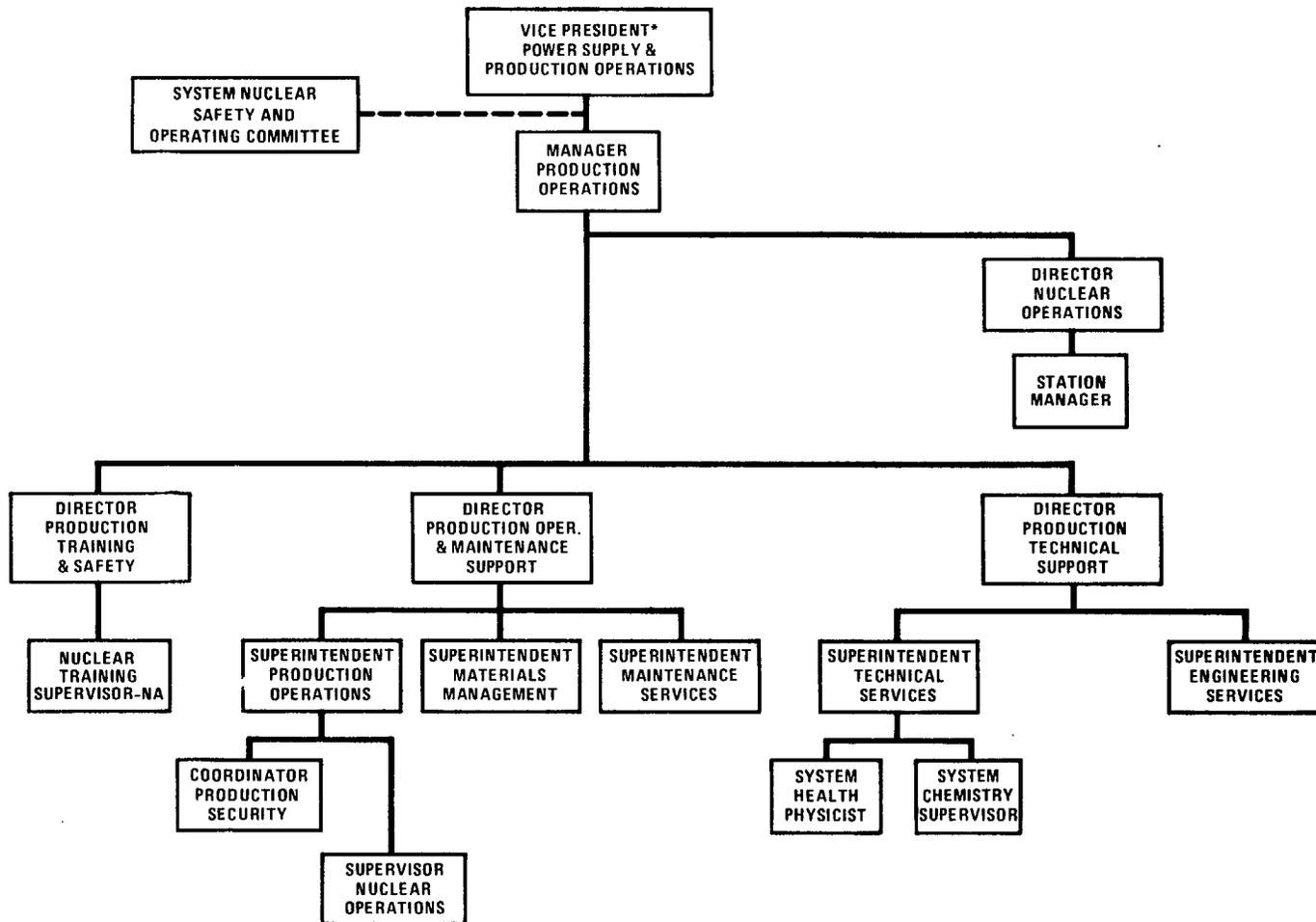
#### OFFSITE

6.2.1 The offsite organization for facility management and technical support shall be as shown on Figure 6.2-1.

#### FACILITY STAFF

6.2.2 The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.
- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. All CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. A Fire Brigade of at least 5 members shall be maintained onsite at all times. The Fire Brigade shall not include the minimum shift crew shown in Table 6.2-1 or any personnel required for other essential functions during a fire emergency.



\*Responsible for Corporate Fire Protection Program.

Figure 6.2-1 Offsite Organization for Facility Management and Technical Support

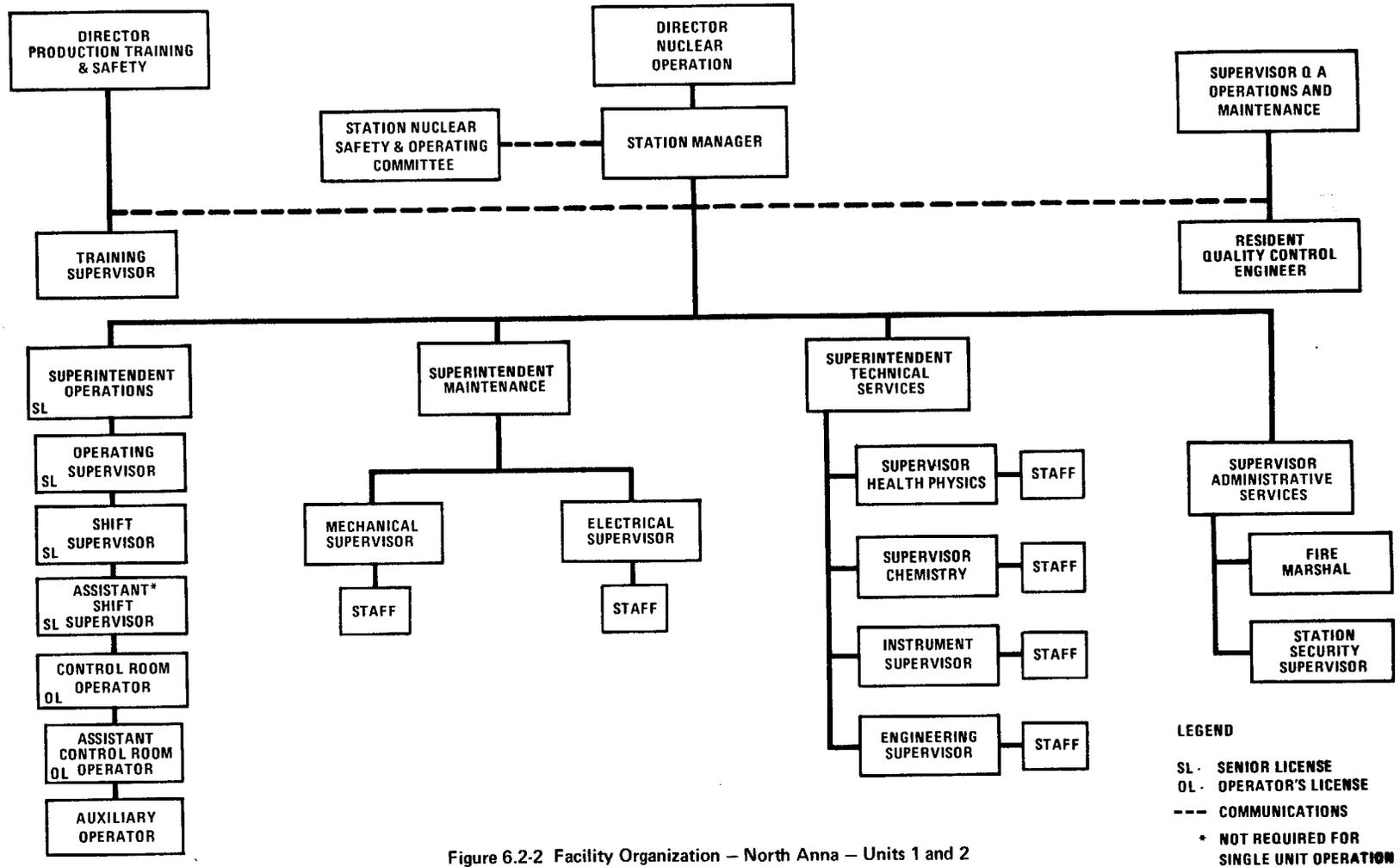


Figure 6.2-2 Facility Organization - North Anna - Units 1 and 2

TABLE 6.2-1

MINIMUM SHIFT CREW COMPOSITION#

LICENSE CATEGORY	APPLICABLE MODES	
	1, 2, 3 & 4	5 & 6
SOL	1	1*
OL	2	1
Non-Licensed	2	1

\*Does not include the licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling, supervising CORE ALTERATIONS.

#Shift crew composition (including an individual qualified in radiation protection procedures) may be less than the minimum requirements for a period of time not to exceed 2 hours in order to accommodate unexpected absence of on duty shift crew members provided immediate action is taken to restore the shift crew composition to within the minimum requirements of Table 6.2-1.

## ADMINISTRATIVE CONTROLS

### 6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Supervisor-Health Physics who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

### 6.4 TRAINING

6.4.1 The Station Manager shall assure that retraining and replacement training programs for the facility staff are maintained and that such programs meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55. These programs shall be directed by the Director-Production Training and Safety.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Fire Marshal and shall meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for Fire Brigade training sessions and drills which shall be held at least once per 92 days.

### 6.5 REVIEW AND AUDIT

#### 6.5.1 STATION NUCLEAR SAFETY AND OPERATING COMMITTEE (SNSOC)

##### FUNCTION

6.5.1.1 The SNSOC shall function to advise the Station Manager on all matters related to nuclear safety.

##### COMPOSITION

6.5.1.2 The SNSOC shall be composed of the:

Chairman:	Station Manager
Vice-Chairman:	Superintendent - Operations
Member:	Superintendent - Maintenance
Member:	Superintendent - Technical Services

##### ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the SNSOC Chairman to serve on a temporary basis; however, no more than one alternate shall participate as a voting member in SNSOC activities at any one time.

## ADMINISTRATIVE CONTROLS

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### MEETING FREQUENCY

6.5.1.4 The SNSOC shall meet at least once per calendar month and as convened by the SNSOC Chairman or his designated alternate.

### QUORUM

6.5.1.5 A quorum of the SNSOC shall consist of the Chairman or Vice-Chairman and two members including alternates.

### RESPONSIBILITIES

6.5.1.6 The SNSOC shall be responsible for:

- a. Review of 1) all procedures required by Specification 6.8.1 and changes thereto, 2) any other proposed procedures or changes thereto as determined by the Station Manager to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to Appendix "A" Technical Specifications.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence to the Director, Nuclear Operations and to the Chairman of the System Nuclear Safety and Operating Committee.
- f. Review of events requiring 24 hour written notification to the Commission.
- g. Review of facility operations to detect potential nuclear safety hazards.

## ADMINISTRATIVE CONTROLS

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- h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Station Nuclear Safety and Operating Committee.
- i. Review of the Plant Security Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.
- j. Review of the Emergency Plan and implementing procedures and shall submit recommended changes to the Chairman of the Station Nuclear Safety and Operating Committee.

## AUTHORITY

6.5.1.7 The SNSOC shall:

- a. Recommend to the Station Manager written approval or disapproval of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide written notification within 24 hours to the Director, Nuclear Operations and the Chairman of the System Nuclear Safety and Operating Committee of disagreement between the SNSOC and the Station Manager; however, the Station Manager shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

## RECORDS

6.5.1.8 The SNSOC shall maintain written minutes of each meeting and copies shall be provided to the Director, Nuclear Operations and Chairman of the System Nuclear Safety and Operating Committee.

## 6.5.2 SYSTEM NUCLEAR SAFETY AND OPERATING COMMITTEE (SyNSOC)

### FUNCTION

6.5.2.1 The SyNSOC shall function to provide independent review and audit of designated activities in the areas of:

## ADMINISTRATIVE CONTROLS

- a. Nuclear power plant operations
- b. Nuclear engineering
- c. Chemistry and radiochemistry
- d. Metallurgy
- e. Instrumentation and control
- f. Radiological safety
- g. Mechanical and electrical engineering
- h. Quality assurance practices
- i. Other appropriate fields associated with the unique characteristics of the nuclear power plant

## COMPOSITION

6.5.2.2 The SyNSOC shall be composed of the Chairman and four other members. Membership shall be composed of the Managers of the Power Station Engineering, Fuel Resources and Production Operations Departments, and Executive Manager Licensing and Quality Assurance or qualified designees from these departments and a fifth qualified member selected by the four other members. Members of the SYNOC shall have an academic degree in an engineering or physical science field and, in addition, shall have a minimum of five years technical experience, of which a minimum of three years shall be in one or more areas given in Section 6.5.2.1.

## ALTERNATES

6.5.2.3 All alternate members shall be appointed in writing by the SyNSOC Chairman to serve on a temporary basis.

## CONSULTANTS

6.5.2.4 Consultants shall be utilized as determined by the SyNSOC Chairman to provide expert advice to the SyNSOC.

ATTACHMENT TO LICENSE AMENDMENT NO. 11

FACILITY OPERATING LICENSE NO. NPF-4

DOCKET NO. 50-338

Replace the following pages of the Appendix "B" Technical Specifications with the enclosed pages as indicated. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Pages

5-1

5-2

5.0 ADMINISTRATIVE CONTROLS

Administrative controls established below are deemed adequate to implement the Environmental Technical Specifications which provide continuing protection to the environment.

5.1 Responsibility

The responsibility for implementing the Environmental Technical Specifications is assigned to the Director of Nuclear Operations at the corporate level and to the Station Manager at the station level. The Superintendent - Operations shall be responsible for ensuring that the station is operated in accordance with the Limiting Conditions of Operation. The Station Supervisor - Health Physics shall be responsible for the radiological environmental surveillance requirements. The Executive Manager of Environmental Services shall be responsible for providing services which will fulfill the nonradiological environmental surveillance requirements.

5.2 Organization

The relationship between the Production Operations Department and the Environmental Services Department is shown in Figure 5.2-1.

5.3 Review and Audit

5.3.1 Station Nuclear Safety and Operating Committee (SNSOC)

5.3.1.1 Function

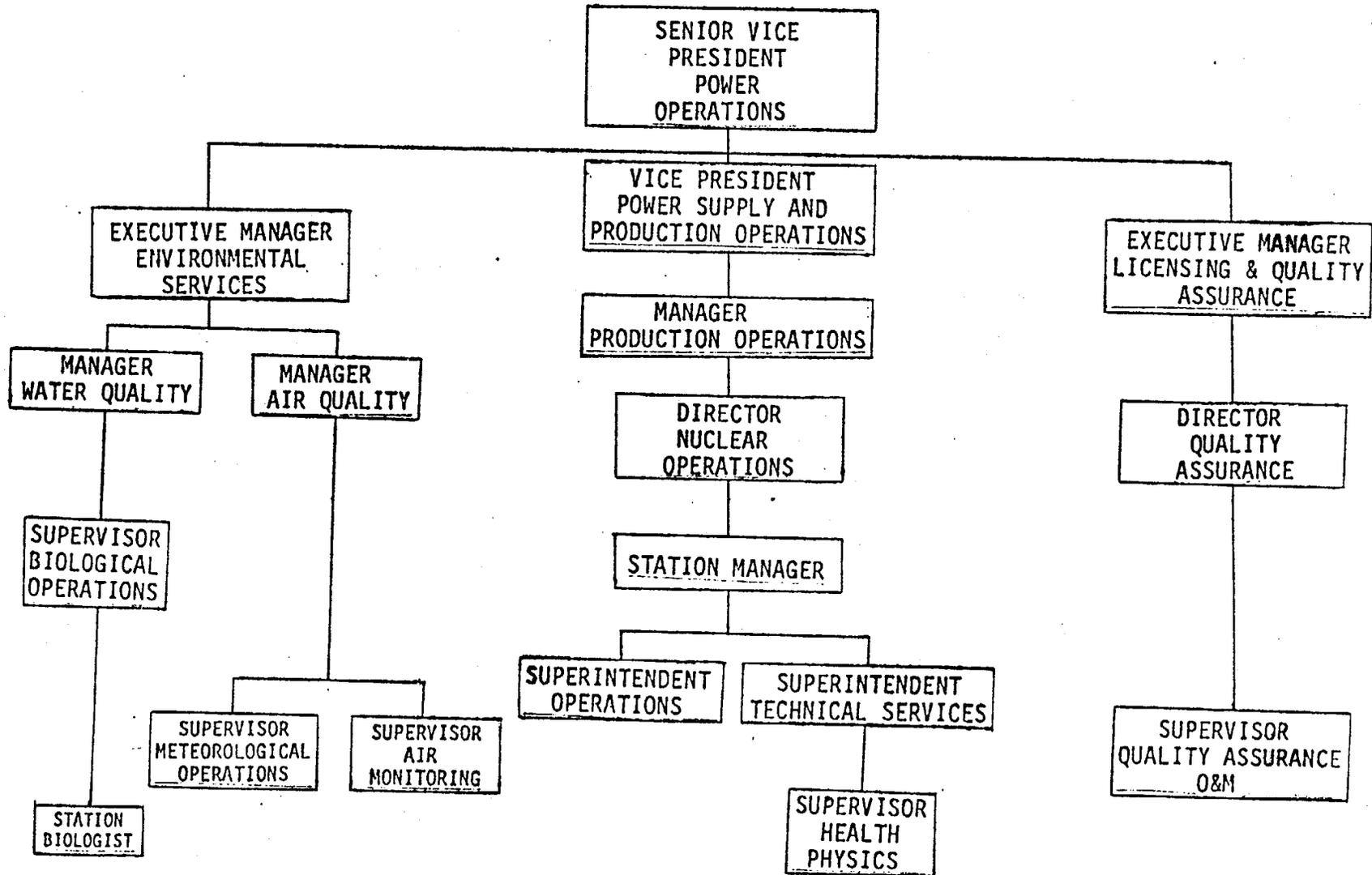
The SNSOC, as described in Section 6.5.1 of Appendix A of this license, shall function to advise the Station Manager on matters related to the environmental impact of the station. When the SNSOC is exercising its responsibility for non-radiological aspects of the ETS, the Station Biologist or his alternate shall be consulted.

5.3.1.2 Responsibility

The SNSOC shall be responsible for:

- a. Coordination of the Environmental Technical Specifications with the Safety Technical Specifications (Appendix A) to avoid conflicts and maintain consistency.
- b. Review of changes to the Environmental Technical Specifications and the evaluation of the environmental impact of the change.

ORGANIZATION CHART  
ENVIRONMENTAL TECH SPEC IMPLEMENTATION



JUN 14 1979

SAFETY EVALUATION BY THE OFFICE OF

NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 11 TO

LICENSE NO. NPF-4

VIRGINIA ELECTRIC AND POWER COMPANY

A. Evaluation in Support of Reorganization of the North Anna Power Station, Unit 1 Production Operations Department, Onsite and Offsite

In a letter dated August 31, 1978, the Virginia Electric and Power Company requested an amendment to Operating License NPF-4 for North Anna Power Station, Unit 1 to reflect a reorganization of the Production Operations Department, onsite and offsite. In a letter dated December 26, 1978, the Virginia Electric and Power Company provided additional supplemental information regarding the offsite reorganization.

We reviewed the information submitted by the Virginia Electric and Power Company and determined additional information was required. As a result of our request for additional information, the Virginia Electric and Power Company submitted the information in letters dated March 8, 1979 and April 27, 1979.

The revised organization for the North Anna plant staff is structured to accomplish its function by four groups: the operations group under the supervision of the Superintendent, Operations; the maintenance group under the supervision of the Superintendent, Maintenance; the technical services group under the supervision of the Superintendent, Technical Services; and an administrative group under the supervision of the Supervisor, Administrative Services.

On the basis of our review, we conclude that the plant staff organization meets the provision of American Nuclear Society Standard N18.7-1976/ANS-3.2 as endorsed by Regulatory Guide 1.33, "Quality Assurance Program Requirements (Operation)", and is therefore acceptable to us for the operation of the North Anna Power Station.

B. Evaluation In Support of Proposed Changes Related to Surveillance Requirements For The Diesel Generator Batteries

In a letter dated June 14, 1978, we advised the Virginia Electric and Power Company that they propose surveillance requirements which demonstrate the operability of the diesel generator batteries.

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In a letter dated August 14, 1978, the Virginia Electric and Power Company proposed changes to specification 4.8.1.1.3 and 4.8.1.2. The proposed changes identified additional surveillance requirements for the diesel generator batteries.

The Virginia Electric and Power Company's proposal was modified to make it acceptable to us. VEPCO has agreed to these changes.

We have reviewed the surveillance requirements proposed by the Virginia Electric and Power Company related to the diesel generator batteries and have determined that their proposed changes to the Technical Specifications 4.8.1.1.3 and 4.8.1.2 meet our surveillance requirements for diesel generator batteries, and therefore, are acceptable.

C. Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

D. Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered or a significant decrease in any safety margin, it does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public. Also, we reaffirm our conclusions as otherwise stated in our Safety Evaluation Report and its Supplements.

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-338

VIRGINIA ELECTRIC AND POWER COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY

OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 11 to Facility Operating License No. NPF-4, issued to Virginia Electric and Power Company (licensee), which revised Technical Specifications, Appendices A and B for operation of the North Anna Power Station, Unit No. 1 to reflect a reorganization of the Production Operation and Maintenance Department both onsite and offsite. The license was also amended to add a new condition which provides for surveillance of the emergency diesel generator batteries. The North Anna Power Station, Unit No. 1 is located in Louisa County, Virginia. The amendment is effective as of its date of issuance.

The applications for the amendment comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's requirements. The licensee requested an amendment to reflect a reorganization of the Production Operations Department in a letter dated, August 31, 1978 and supplemented by a letter, dated December 26, 1978. By NRC letter, dated June 14, 1978, we requested that the licensee propose surveillance requirements which demonstrate the operability of the diesel generator batteries. The licensee responded by letter, dated August 14, 1978 proposing changes to the surveillance requirements for the diesel generator batteries. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are

OFFICE	Set forth in the license amendment.	Prior public notice of this amendment	
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was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment, dated August 31, 1978 as supplemented on December 26, 1978, and NRC letter, dated June 14, 1978, and the licensee's reply, dated August 14, 1978, (2) Amendment No. 11 to License No. NPF-4, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Board of Supervisor's Office, Louisa County Courthouse, Louisa, Virginia 23093 and at the Alderman Library, Manuscripts Department, University of Virginia, Charlottesville, Virginia 22901. A copy of these items may be obtained upon request to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Project Management.

Dated at Bethesda, Maryland this 14<sup>th</sup> day of June, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
O. D. Parr

Olan D. Parr, Chief  
Light Water Reactors Branch No. 3  
Division of Project Management

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