

April 3, 1987

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Amat. 116 to DPR-41

Docket Nos.: 50-250  
and 50-251

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Mr. C. O. Woody, Group Vice President  
Nuclear Energy Department  
Florida Power and Light Company  
Post Office Box 14000  
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Dear Mr. Woody:

On March 6, 1987, the Commission issued Amendment No. 123 to Facility Operating License No. DPR-31 and Amendment No. 116 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Unit Nos. 3 and 4, respectively. The amendment revised the immediate notification requirements and the Licensee Event Reporting System, as well as revising the Off-Site Organization for Facility Management and Technical Support and the Plant Organization Chart to reflect the current structure and position titles. The amendments also corrected areas in the Technical Specifications which contained conflicting submittal directions to be in accordance with the final rule on "Communications, Procedure Amendments, 10 CFR Part 50."

Due to an administrative error, page B4.2-13, which was issued with the amendments, should not have been included in the Technical Specification changes. Page B4.2-13 had previously been deleted in Amendment Nos. 119 and 113, issued on October 27, 1986. Please remove this page from your Technical Specifications.

Please accept our apologies for this error.


Sincerely,

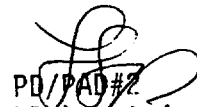
/s/

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Mr. C. O. Woody  
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Turkey Point Plant

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### Item 7.3 - Steam Generator Tube Inspection

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those parameter limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these parameter limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system (primary-to-secondary leakage = 1 gallon per minute, total). Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 1 gallon per minute can readily be detected by radiation monitors of steam generator blowdown. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged.

Wastage-type defects are unlikely with the all volatile treatment (AVT) of secondary coolant. However, even if a defect of similar type should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging will be required of all tubes with imperfections exceeding the plugging limit which, by the definition of Specification 4.2.5.4.a is 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, of Table 4.2-3, these results shall be reported to the Commission pursuant to Specification 6.9.3.k.