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August 13, 1986

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Docket Nos. 50-250
and 50-251

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Mr. C. O. Woody, Group Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
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Dear Mr. Woody:

The Commission has issued the enclosed Amendment No. 118 to Facility Operating License No. DPR-31 and Amendment No. 112 to Facility Operating License No. DPR-41 for the Turkey Point Plant Units Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated January 30, 1986, as supplemented July 30, 1986.

These amendments will revise the Technical Specifications to provide administrative control of the Standby Feedwater System. These controls will provide assurance that the system will be available during plant operations. The system is normally used during startup. However, this system is capable of supplying feedwater to the steam generators in the event of an accident, thus providing additional in-depth defense for mitigating the consequences. The controls include surveillance requirements, limiting conditions of operation, supporting basis and reporting requirements.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular bi-weekly Federal Register notice.

Sincerely,

/s/

Daniel G. McDonald, Jr., Project Manager
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 118 to DPR-31
2. Amendment No. 112 to DPR-41
3. Safety Evaluation

cc: w/enclosures
See next page

*See previous concurrence

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 118
License No. DPR-31

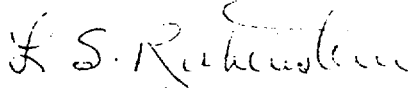
1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated January 30, 1986, as supplemented July 30, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 118, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 13, 1986



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 112
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated January 30, 1986, as supplemented July 30, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendix A and B, as revised through Amendment No. 112, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective immediately and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Lester S. Rubenstein, Director
PWR Project Directorate #2
Division of PWR Licensing-A
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 13, 1986

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 118 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 112 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NO. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages

i
ii
iii
iv
1.9
3.20-1
4.21-1
6.23
B3.20-1
B4.21-1

Insert Pages

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ii
iii
iv
1.9
3.20-1
4.21-1
6.23
B3.20-1
B4.21-1

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1.30 PURGE - PURGING

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

1.31 VENTING

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is not provided or required during VENTING. Vent, used in system names, does not imply a VENTING process.

1.32 SITE BOUNDARY

The SITE BOUNDARY shall be that line beyond which the land is neither owned, leased nor otherwise controlled by the licensee.

1.33 UNRESTRICTED AREA

An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation from radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional and/or recreational purposes.

1.34 MEMBER(S) OF THE PUBLIC

MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the licensee, its contractors, vendors or members of the Armed Forces using property located within the SITE BOUNDARY. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational or other purposes not associated with the plant.

1.35 HEAVY LOADS

Any load in excess of the nominal weight of a fuel and control rod assembly and associated handling tool. For the purpose of this specification, HEAVY LOADS will be defined as loads in excess of 2000 pounds.

1.36 OPERATIONAL MODE - MODE

An OPERATIONAL MODE (i.e., MODE) shall correspond to any one inclusive combination of core reactivity condition, power level, and average reactor coolant temperature specified in Table 1.1.

1.37 STAGGERED TEST BASIS

A STAGGERED TEST BASIS shall consist of:

- a. A test schedule for (n) systems, subsystems, trains, or other designated components obtained by dividing the specified test interval into (n) equal subintervals, and
- b. The testing of one system, subsystem, train, or other designated component at the beginning of each subinterval.

STANDBY FEEDWATER SYSTEM

3.20.1 Two standby feedwater pumps shall be available* and at least 60,000 gallons of water (available volume), shall be in the Demineralized Water Storage Tank.**

APPLICABILITY: MODES 1, 2, 3

ACTION:

1. With one standby feedwater pump unavailable, restore the unavailable pump to available status within 30 days or submit a SPECIAL REPORT per 6.9.3.j.
2. With both standby feedwater pumps unavailable:
 - a) Within 24 hours, notify the NRC and provide cause for unavailability and plans to restore pump(s) to available status and,
 - b) Submit a SPECIAL REPORT per 6.9.3.j.
3. With less than 60,000 gallons of water in the Demineralized Water Storage Tank restore the available volume to at least 60,000 gallons within 24 hours or submit a SPECIAL REPORT per 6.9.3.j.
4. The provisions of Specifications 3.0.1 and 3.0.4 are not applicable.

* These pumps are not safety related equipment and do not require plant safety related emergency power sources for availability.

** The Demineralized Water Storage Tank is non-safety grade.

4.21 STANDBY FEEDWATER SYSTEM

APPLICABILITY: Applies to the periodic surveillance of the STANDBY FEEDWATER SYSTEM.

OBJECTIVE: To demonstrate availability of the STANDBY FEEDWATER SYSTEM*.

*NOTE: The standby feedwater pumps are not safety related equipment and do not require plant safety related emergency power sources for availability.

SPECIFICATION:

1. The Demineralized Water Storage tank water volume shall be determined to be within limits at least once per 24 hours.
2. At least monthly verify the standby feedwater pumps are available by testing in recirculation on a STAGGERED TEST BASIS.
3. During each refueling outage, verify availability of the respective standby feedwater pump by powering from the non-safety grade diesel generators and providing feedwater to the steam generators.

- g. With untreated gaseous effluents exceeding the limits of 3.9.2.e pursuant to Specification 3.9.2.e.3, submit a report which includes the following information:
- (1) Identification of the inoperable equipment or subsystems and the reason for inoperability,
 - (2) Action(s) taken to restore the inoperable equipment to OPERABLE status, and
 - (3) Summary description of action(s) taken to prevent a recurrence.
- h. With the annual (calendar year) dose or dose commitment to any MEMBER OF THE PUBLIC from all uranium fuel cycle sources exceeding the limits of Technical Specification 3.9.2.h, submit a report that defines the corrective action to be taken to reduce subsequent releases to prevent recurrence of exceeding the limits of Specification 3.9.2.h and includes the schedule for achieving conformance with those limits. This report, as defined in 10 CFR Part 20.405c, shall include an analysis that estimates the radiation exposure (dose) to a MEMBER OF THE PUBLIC from uranium fuel cycle sources, including all effluent pathways and direct radiation, for the calendar year that includes the release(s) covered by this report. It shall also describe levels of radiation and concentrations of radioactive material involved, and the cause of the exposure levels or concentrations. If the estimated dose(s) exceeds the limits of Specification 3.9.2.h and if the release condition resulting in violation of 40 CFR Part 190 has not already been corrected, the report shall include a request for a variance in accordance with the provisions of 40 CFR Part 190. Submittal of the report is considered a timely request, and a variance is granted until staff action on the request is complete.
- i. With the measured levels of radioactivity in environmental samples as a result of plant effluents pursuant to Specification 4.12.1.b, submit a report that identifies the cause(s) for exceeding the limit(s) and defines the corrective actions to be taken to reduce radioactive effluents so that the potential dose to a MEMBER OF THE PUBLIC is less than the limits of Specifications 3.9.1.b, 3.9.2.b and 3.9.2.c.
- j. If the Limits of Technical Specifications 3.20 are exceeded, submit a report describing the cause of the unavailability, action taken and a schedule for restoration within 30 days.

BASES FOR LIMITING CONDITIONS FOR OPERATION**STANDBY FEEDWATER SYSTEM**

The purpose of this specification and the supporting surveillance requirements is to provide for administrative controls which will assure availability and performance of the non-safety grade standby feedwater system. The term availability is used rather than operability so as to positively avoid any implication or connotation that the standby feedwater system is designed or documented to meet safety system requirements such as seismic loads, environmental qualification, safety grade emergency power supply, etc. The system does consist of commercial grade components designed and constructed to industry and FPL standards for this class of equipment located in the outdoor plant environment typical of FPL facilities system wide. The system is expected to perform with high reliability, i.e., comparable to that typically achieved with this class of equipment. FPL intends to maintain the system in good operating condition with regard to appearance, structures, supports, component maintenance, calibrations, etc., but would not document these activities to the degree required for safety grade systems.

The standby feedwater system can be used as a backup to the auxiliary feedwater (AFW) system in the event the AFW system does not function properly. The system would be manually started and controlled by the operator when needed. In the event of a loss of offsite power the pumps can be powered via the non-safety grade diesel generators connected to the non-vital 4160 volt bus.

A supply of 60,000 gallons from the Demineralized Water Storage Tank for the Standby Feedwater Pumps is sufficient water to remove decay heat from the reactor for six (6) hours for a single unit or two (2) hours for two units. This was the basis used for requiring 60,000 gallons of water in the non-safety grade Demineralized Water Storage Tank and is judged to provide sufficient time for restoring the AFW System or establishing make-up to the Demineralized Water Storage Tank.

The motor driven standby feedwater pumps are not designed to NRC requirements applicable to emergency feedwater systems and not required to meet design basis events. These pumps may be out of service for up to 24 hours before initiating formal notification because of the extremely low probability of such an event.

The guidelines for NRC notification in case of both pumps being out of service for longer than 24 hours are provided in applicable plant procedures, as a VOLUNTARY 4 hour notification.

BASES FOR SURVEILLANCE REQUIREMENTS

STANDBY FEEDWATER SYSTEM

Adequate demineralized water for the standby feedwater system will be verified once per 24 hours. The Demineralized Water Storage Tank provides a source of water to several systems and therefore, requires daily verification.

The standby feedwater pumps will be verified available monthly on a STAGGERED TEST BASIS by starting and operating them in the recirculation mode typically from their normal power supply. Also, during each unit's refueling outage, the respective standby feedwater pump will be powered from the unit's C bus utilizing Units 1 and 2 non-safety grade diesel generators and flow tested to the nuclear unit's steam generators. Prior to this test, the refueling unit's C bus will be de-energized and the necessary loads will be transferred to the other unit's C bus.

This surveillance regimen will thus demonstrate availability of the entire flow path, backup non-safety grade power supply and pump associated with a unit at least each refueling outage. The pump, motor driver, and normal power supply availability would typically be demonstrated by operation of the pumps in the recirculation mode monthly on a staggered test basis.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 118 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 112 TO FACILITY OPERATING LICENSE NO. DPR-41
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

I. INTRODUCTION

On November 1, 1983, the NRC issued Generic Letter 83-37 to all licensees, including Florida Power and Light Company (FPL), the licensee for Turkey Point Plant, Units 3 and 4, requesting it to determine whether the auxiliary feedwater (AFW) system design and Technical Specifications are consistent with the guidance provided in the Generic Letter for improved reliability and performance of the AFW system. In this regard, the safety related AFW system at Turkey Point Plant, Units 3 and 4, was judged to not have appropriate diversity. As a result, by letter dated November 1, 1985, FPL committed to implement appropriate Technical Specification requirements, in accordance with the Generic Letter, for the Standby Feedwater System to provide greater assurance of its availability.

By letter dated January 30, 1986, and supplemented July 30, 1986, FPL submitted a request for amendments to the facility operating licenses, proposing Technical Specifications for the non-safety grade Standby Feedwater System (SFS). Staff review of this matter included the existing AFWS Technical Specifications, as well as the proposed changes to them, and the supporting bases.

II. EVALUATION

The licensee indicated that the purpose of the proposed Technical Specifications are to ensure the availability of the two non-safety grade motor driven, standby feedwater pumps during plant operation. These pumps are normally powered by the non-safety 4160 Volt C-bus; in the event of a loss of off-site power, five non-safety grade diesel generator have the capability to provide backup power.

The licensee has proposed that with one standby feedwater pump unavailable, the unavailable pump must be restored to available status within 30 days, or a report will be submitted to the NRC within 30 days describing the cause of the unavailability, and providing a schedule for the resolution. With both pumps unavailable or less than 60,000 gallons of water in the Demineralized Water Storage Tank (DWST), the problems will be corrected within 24 hours or a Special Report will be submitted to the NRC which includes details of the corrective actions. Further, each standby feedwater pump shall be demonstrated operable at least once per 31 days on a staggered test basis. The licensee

initially proposed a quarterly test interval which is inconsistent with the test interval in the Westinghouse Standard Technical Specifications for auxiliary feedwater pumps. Based on discussions with the staff, the licensee agreed to the test interval which is consistent with the Westinghouse Standard Technical Specifications, and modified and supplemented its amendment request by letter dated July 30, 1986.

This change resulted in the adding the definition of Staggered Test Basis to page 1.9 of the Technical Specifications and changing pages 4.21-1 and B4.21-1 to reflect the requirement. In addition, the change in frequency of the test to achieve consistency results in a minor change to the action previously noticed, and is within the scope of the action described in the notice. Thus, this amendment request was not renoticed.

The system is not automatically initiated and would require operator action if used. Although the system was installed and is used for a source of feedwater during normal startup, it could provide an additional source of water during accident mitigation for decay heat removal if multiple failures would result in loss of the safety-related water. The addition of Technical Specification requirements for surveillance and Limiting Conditions for Operation (LCO's) on this non-safety system will provide a higher degree of assurance that this additional source of water would be available for decay heat removal.

III. FINDINGS

The staff finds that the proposed changes to the Turkey Point, Units 3 and 4 Technical Specifications, regarding surveillance of the non-safety grade standby feedwater system, are consistent with the Standard Technical Specifications and will provide a higher degree of assurance that the system will be available if needed. The reporting requirements for loss of a single pump, both pumps or inadequate volume of water will assure that the NRC staff is aware of the conditions, plans for correcting the conditions and the time necessary to restore the system to operable status, thus assuring the system will not be unavailable for extended periods of time. Therefore, based on our evaluation, we conclude that the proposed changes are acceptable.

IV. ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation, use, or surveillance of the facilities components located within the restricted areas as defined in 10 CFR 20. The staff has determined that these amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

V. CONCLUSION

We have concluded, based on the considerations discussed above, that:
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: August 13, 1986

Principal Contributor:

A. Singh