

NOVEMBER 8 1978

Docket Nos. 50-250
50-251

F

Florida Power and Light Company
ATTN: Dr. Robert E. Uhrig, Vice President
Advanced Systems & Technology
Post Office Box 529100
Miami, Florida 33152

Gentlemen:

The Commission has issued the enclosed Amendment No. 4² to Facility Operating License No. DPR-31 and Amendment No. 4 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Units Nos. 3 and 4. These amendments implement Appendix A Technical Specifications for fire protection in response to your application of December 22, 1977 (L-77-390).

These amendments revise the common station Technical Specifications to incorporate limiting conditions for operation and surveillance requirements for existing fire protection systems and administrative controls. These changes to the Turkey Point Technical Specifications are supported by the Safety Evaluation issued with our letter of November 25, 1977, except for those changes which are discussed herein and to which your staff disagreed.

1. In order to achieve expeditious implementation of the fire protection Technical Specifications, Specification 6.2.2.f is being issued at this time with the minimum number of on-site fire brigade members specified as three (3) as you proposed. This number is less than the minimum number of five (5) given in the generic staff position, Minimum Fire Brigade Shift Size, which was an attachment to the Safety Evaluation Report issued with our letter to you dated November 25, 1977. However, we are evaluating your justification for this smaller brigade size and when our evaluation is complete the minimum number will be increased if we do not agree with your position.
2. In Specification 3.14.1.b, we are not requiring hourly inspection of the two cable penetration areas inside containment (zones 5 and 6) should the fire detectors in these areas become inoperable. Our evaluation was based on the fire hazards analyses presented in your submittal of February 25, 1977 (L-77-57). As discussed in sections 6.4, 7.3, 8.5.2.21 and 8.5.2.22 and in Table 7.9 of your analyses, 1) there are two separated penetration areas, 2) cable trays in the

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Florida Power & Light Company - 2 -

two penetration areas are sprayed with Flamemastic 71A, 3) once the trays leave the penetration area they are well separated, 4) there are no combustible sources in the area of the penetrations other than the cured paint on the walls of the containment and 5) the three sensors that measure ambient temperatures in the containment area and continually record this data in the control room provide an acceptable degree of backup to the ionization detectors.

3. In Specification 3.14.1.d a requirement has been established to inspect a turbine area once per hour in lieu of provisions for installed fire detectors. The staff has evaluated the fire protection afforded by this inspection frequency in the turbine area and found that because the safety related cables in this area are covered with flame retardant coating and the only fire hazard in the area would be transient combustibles this alternative provides an acceptable degree of fire protection. On this basis the proposed inspection frequency is acceptable.
4. In Specification 3.14.4.a, you proposed to modify the requirement to maintain a continuous surveillance of a non-functional fire barrier to require inspection of the inoperable barrier at least once per hour until the affected penetration fire barrier is restored on the basis that the areas containing safety-related equipment are also protected by operable fire detectors. We are still evaluating the justification for your proposal. If we do not agree with your positions after we complete our evaluation we will require additional protection in the event of a non-functional fire barrier.
5. In Specification 4.15.2.a.1 an increased inspection frequency of the water level has been proposed to accommodate the fact that the water supply is not physically dedicated to fire protection but rather is administratively controlled. Administrative controls are not as positive a control as the physical dedication of the water supply. The staff has reviewed the proposed increase in inspection frequency and finds that the increased frequency provides the necessary added assurance that the water supply will be adequate during the period that we are evaluating your position. However, if we do not agree with your position after we complete our evaluation we will require additional protection.
6. In Specification 4.15.2.a.3, we retained the requirement that the position of all valves be verified monthly. Originally, our position had been that which you proposed - namely, that valves that were locked, sealed or otherwise secured in position did not have to be verified. Our subsequent evaluation indicates that the position of all valves

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in the flow path should be checked. On the basis of our subsequent evaluation we find that the requirement that the positions of all valves be verified monthly is adequate.

7. In Specification 4.15.2.a.7, we changed the requirement on the system flow test to be equivalent to a standard reference (i.e., NFPA). The test you proposed will be acceptable if it is equivalent to NFPA.
8. In Specification 4.15.3.a.2, the interval for visual inspection of hose, reracking and replacement of degraded gaskets in couplings was reduced from "once per 18 month" to "annually" as you proposed. On the basis that it is a more frequent inspection the staff finds an annual inspection acceptable.
9. In Specification 4.15.3.a.4, we accepted your proposed requirement that the hose hydrostatic test be at a pressure at least equal to the system pressure. We have not completed our review of your justification for using system pressure. We do find that it is acceptable as an interim measure. Should our review indicate that the higher test pressure is necessary we will change the requirement to the higher pressure.
10. In our letter of November 25, 1977, Specification 6.4.2 concerning fire brigade training requires that the training program meet or exceed the intent of NFPA Code-1975 Section 27. This would have imposed a requirement that training be conducted monthly. However, our letter of August 19, 1977, forwarded a copy of "Nuclear Plant Fire Protection Functional Responsibilities, Administrative Controls and Quality Assurance", which indicated that fire brigade training shall be conducted at least each three (3) months. We have therefore modified this specification to require training at quarterly intervals.
11. In specification 3.14.2.a you have proposed a time interval of one shift to run a fire hose to an area with an inoperable hose on the basis that one shift (8 hours) is not significantly different, from a fire protection viewpoint, than the one hour proposed by the staff. We have not completed our review of your justification for the 8 hour period. In addition, the hose stations and deluge systems which protect safety related equipment are not specifically listed. We find that this specification is acceptable only on an interim basis. Should our review indicate that this specification is inadequate we will require additional fire protection in this area.

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12. Specification 6.8.1 allows a total of six months for establishing procedures (60 days by virtue of the amendment effective date and four months by this specification). We do not agree with this length of time as proposed in your Technical Specification but will accept it on an interim basis. When our evaluation is completed, should we still not agree, we will require a shorter period of time.

As we have discussed with you items 1, 4, 5, 9, 11 and 12 are subject to further review. We will notify you of our final decision on these matters when our review of your justifications for these items is complete.

During our review of the proposed request, we found that certain changes were necessary. Your staff agreed to these changes and they have been incorporated.

A copy of the FEDERAL REGISTER notice of this license amendment is also enclosed.

Sincerely,

ORIGINAL SIGNED

A. Schwencer
 Operating Reactors Branch #1
 Division of Operating Reactors

Enclosures:

1. Amendment No. ⁴² to License DPR-31
2. Amendment No. ³⁴ to License DPR-41
3. FEDERAL REGISTER NOTICE

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DATE →		9/25/78	9/25/78	11/6/78	11/3/78	11/ /78

November 8, 1978

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 42
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated December 22, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 42, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective 60 days from the date of its issuance. The surveillance requirements associated with Specification 4.15.1, 4.15.2, 4.15.3, 4.15.4, 6.5.2.8.d, 6.5.2.8.h and 6.5.2.8.i become effective upon completion of the first surveillance interval following the issuance of this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 8, 1978



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 34
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated December 22, 1977, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-41, is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 34, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective 60 days from the date of its issuance. The surveillance requirements associated with Specification 4.15.1, 4.15.2, 4.15.3, 4.15.4, 6.5.2.8.d, 6.5.2.8.h and 6.5.2.8.i become effective upon completion of the first surveillance interval following the issuance of this amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: November 8, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 42

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-31

DOCKET NO. 50-250

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Replace</u>	<u>Add</u>
ii	ii	-
iii	iii	-
-	-	3.14-1
-	-	3.14-2
-	-	3.14-3
Table 4.1-2 (sheet 2 of 3)	Table 4.1-2 (sheet 2 of 3)	-
-	-	4.15-1
-	-	4.15-2
-	-	B3.14-1
-	-	B4.13-1
6-2	6-2	
6-3	6-3	
6-5	6-5	
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6-13	6-13	
6-14	6-14	
6-22	6-22	

ATTACHMENT TO LICENSE AMENDMENT NO. 34

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NO. 50-251

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

<u>Remove</u>	<u>Replace</u>	<u>Add</u>
ii	ii	-
iii	iii	-
-	-	3.14-1
-	-	3.14-2
-	-	3.14-3
Table 4.1-2 (sheet 2 of 3)	Table 4.1-2 (sheet 2 of 3)	
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-	-	4.15-2
-	-	B3.14-1
-	-	B4.13-1
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3.14 FIRE PROTECTION SYSTEMS

Applicability: Applies to the availability of fire protection systems in nuclear safety related areas.

Objective: To define those conditions of fire protection availability.

Specification: 1. FIRE DETECTION SYSTEM

- a. The Fire Detection System instrumentation for each protected zone shall be operable, when safety related equipment located in the zone is required to be operable, with not less than the minimum number of detectors (listed in Table 3.14-1) installed in the zone operable.
- b. If Specification 3.14.1.a cannot be met within one (1) hour, a fire watch patrol shall inspect the inoperable zone(s) at least once per hour, as operational conditions permit, until the zone(s) is restored to operable status. Exempt from this Specification are those detectors in Zones 5 and 6 which are located inside of containment where surveillance would be inconsistent with A.L.A.R.A. criterion.
- c. If the operability requirements of Specification 3.14.1.a cannot be met within fourteen (14) days, prepare and submit a Report to the Commission pursuant to Specification 6.9.2 .b.
- d. A fire watch patrol shall be established to inspect the 18 foot level of the turbine area once each hour.*

2. FIRE PROTECTION WATER SYSTEM

- a. The Fire Protection Water System shall have:
 1. Two (2) high pressure fire pumps operable and aligned to the high pressure fire header.
 2. Separate water supplies, with a minimum contained volume of 30,000 gallons in the elevated storage tank (EST) and 150,000 gallons in the raw water storage tank.
 3. An operable flow path capable of taking suction from the raw water storage tank and transferring the water through distribution piping.

*To be implemented as of the date of approval of the modified amended security plan but not later than February 1979.

- b.1. With one pump and/or one water supply inoperable, restore the inoperable equipment to operable status within seven (7) days or prepare and submit a Report to the Commission pursuant to Specification 6.9.2.b.
 - b.2. With one water supply below the minimum specified limit for one day connect the spool piece to make the screen wash pump available for fire water supply.*
 - c. With no Fire Protection Water System operable:
 1. Establish a backup fire protection water system within twenty-four (24) hours, and
 2. Submit a Report in accordance with Specification 6.9.2. a.
3. FIRE HOSE STATIONS
- a. Fire hose stations in the vicinity of safety related equipment shall be operable at all times when the safety related equipment in their area of protection is required to be operable; or within eight (8) hours, an equivalent capacity fire hose shall be run from an equivalent water source to the inoperable location.
4. PENETRATION FIRE BARRIERS
- a. All electrical penetration fire barriers protecting safety related equipment shall be functional when safety related equipment in the area is required to be operable; or within one (1) hour of the barrier being found not intact; a fire watch patrol shall inspect at least one side of the inoperable barrier at least once per hour, as operational conditions permit, until the affected penetration fire barrier is restored.

*This specification becomes effective upon completion of the necessary modifications. These modifications to be completed no later than November 1979.

TABLE 3.14-1
FIRE DETECTION SYSTEM

<u>ZONE</u>	<u>ZONE LOCATION</u>	<u>MINIMUM NUMBER OF OPERABLE</u>	
		<u>HEAT DETECTORS</u>	<u>SMOKE DETECTORS</u>
2	Cable Spreading Room, El. 30'	2	10
3	Auxiliary Building, El. 18'	0	11
4	Auxiliary Building, El. 10'	0	8
5	Containment Electrical Penetration Rooms, Unit 3	0	16
6	Containment Electrical Penetration Rooms, Unit 4	0	12
11	Battery Rooms, El. 42'	2	3

TABLE 4.1-2 (Sheet 2 of 3)
 MINIMUM FREQUENCIES FOR EQUIPMENT AND SAMPLING TESTS

11.	Reactor Coolant System Leakage	Evaluate	Daily	NA
12.	Diesel Fuel Supply	Fuel inventory	Weekly	10
13.	Spent Fuel Pit	Boron Concentration	Prior to refueling	NA
14.	Secondary Coolant	I-131 Concentration	Weekly * †	10
15.	Vent Gas & Particulates	I-131 & Particulate Activity	Weekly *	10
16.	Fire Protection Pump & Power Supply	Operable	Monthly	45
17.	Turbine Stop and Control Valves, Reheater Stop and Intercept Valves	Closure	Monthly ***	45
18.	LP Turbine Rotor Inspection (w/o rotor disassembly)	V, MT, PT	Every 5 Years	6 Years
19.	Spent Fuel Cask Crane Interlocks	Functioning	Within 7 days of using crane to lift spent fuel cask	7 days when crane is being used to maneuver spent fuel cask

† - N.A. during cold or refueling shutdowns. The specified tests, however, shall be performed prior to heatup above 200 F.

* When activity exceeds 10% of specification, frequency shall be changed to daily.

4.15 FIRE PROTECTION SYSTEMS

Applicability: Applies to the surveillance of the Fire Protection Systems.

Objective: To verify the operability of the Fire Protection Systems.

Specification: 1. FIRE DETECTION SYSTEM

- a. The Fire Detection System shall be demonstrated operable by:

<u>TEST</u>	<u>INTERVAL</u>
1. Channel functional test	Semi-annual
2. Test NFPA Code 72D Class A supervised circuits associated with the detector alarms	Semi-annual

2. FIRE PROTECTION WATER SYSTEM

- a. The Fire Protection Water System shall be demonstrated operable by:

<u>TEST</u>	<u>INTERVAL</u>
1. Verify water supply volume(s)	Daily
2. Simulate automatic actuation of pumps on recirculation flow	Monthly
3. Verify that each valve, (manual, power operated or automatic) in the flow path is in its correct position	Monthly
4. System flush	Annually
5. Cycle each testable valve	Annually
6. System functional test, which includes simulated automatic actuation of the system throughout its operating sequence, and:	18 Months
a. Verify that each pump develops at least 94% rated flow and 93% system head.	
b. Cycle each valve which is not testable during plant operation.	

Amendment No. 42 Unit 3

Amendment No. 34 Unit 4

- c. Verify that each pump starts sequentially to maintain the Fire Protection Water System pressure \geq 125 psig.

- 7. System flow test equivalent with Chapter 5, Section 11 of the Fire Protection Handbook, 14th Edition, published by the National Fire Protection Association. 3 Years

3. FIRE HOSE STATIONS

- a. Hose stations in the vicinity of safety related equipment shall be demonstrated operable by:

<u>TEST</u>	<u>INTERVAL</u>
1. Visual inspection of hose station equipment	Monthly
2. Visual inspection of hose, re-racking, and replacement of degraded gaskets in couplings.	Annually
3. Partially open each hose station valve to verify valve operability and no flow blockage.	3 Years
4. Hose hydrostatic test at a pressure at least equal to the system pressure available at that hose station.	3 Years

4. PENETRATION FIRE BARRIERS

- a. Electrical penetration fire barriers protecting safety related areas shall be verified to be functional (intact) by a visual inspection:
 - 1. At least once per 18 months, and
 - 2. Prior to declaring a penetration fire barrier functional following repairs or maintenance.

Amendment No. 42 Unit 3
Amendment No. 34 Unit 4

B3.13 BASES FOR FIRE PROTECTION SYSTEMS

OPERABILITY of the fire detection instrumentation ensures that adequate warning capability is available for the prompt detection of fires. This capability is desired in order to detect and locate fires in their early stages. Prompt detection of fires will reduce the potential for damage to safety related equipment.

In the event that a portion of the fire detection instrumentation is inoperable, the establishment of a fire patrol(s) in the affected area(s) is required to provide detection capability until the inoperable instrumentation is restored to OPERABILITY.

The OPERABILITY of the fire suppression systems ensures that adequate fire suppression capability is available to confine and extinguish fires occurring in any portion of the facility where safety related equipment is located. The fire suppression system consists of the water system, and fire hose stations. The collective capability of the fire suppression systems is adequate to minimize potential damage to safety related equipment.

In the event that portions of the fire suppression systems are inoperable, alternate backup fire fighting equipment is required to be made available in the affected areas until the inoperable equipment is restored to service.

In the event the fire protection water system becomes inoperable, corrective measures must be taken since this system provides the major fire suppression capability of the plant. The requirement for a twenty-four hour report to the Commission provides for prompt evaluation of the acceptability of the corrective measures to provide adequate fire suppression capability.

The functional integrity of the penetration fire barriers ensures that fires will be adequately retarded from spreading to adjacent portions of the facility. This design feature minimizes the possibility of a single fire rapidly involving several areas of the facility prior to detection and extinguishment. The penetration fire barriers are a passive element in the facility fire protection program and are subject to periodic inspections.

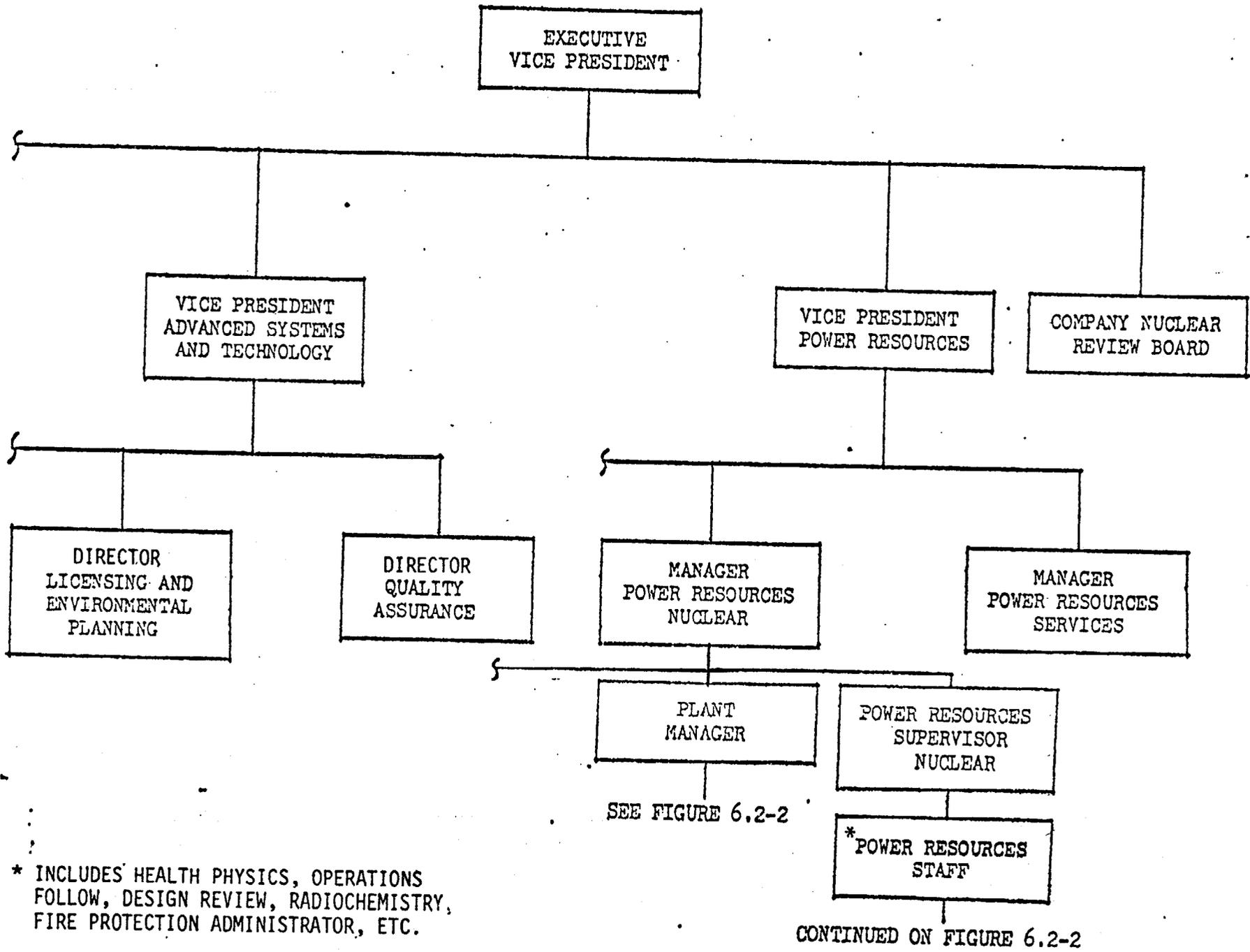
During periods of time when the barriers are not intact, a fire watch patrol is required until the barrier is restored to its intact status.

B4.13 BASES FOR FIRE PROTECTION SYSTEMS

The surveillance specified is designed to demonstrate the OPERABILITY of the system.

Reference: B3.13

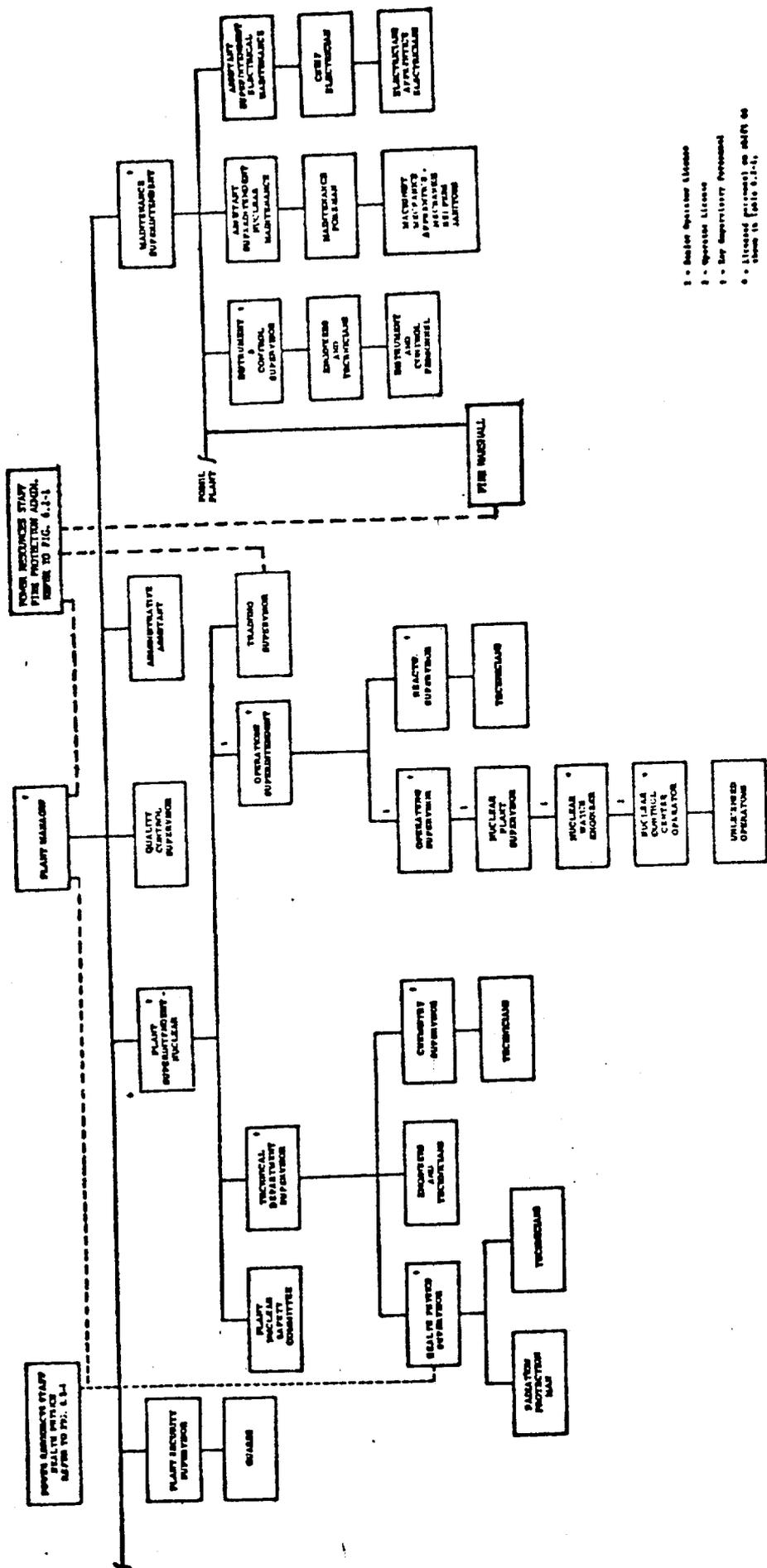
Amendment No. 42 Unit 3
Amendment No. 34 Unit 4



* INCLUDES HEALTH PHYSICS, OPERATIONS FOLLOW, DESIGN REVIEW, RADIOCHEMISTRY, FIRE PROTECTION ADMINISTRATOR, ETC.

FIGURE 6.2-1 OFFSITE ORGANIZATION CHART

Amendment No. 42 Unit 3
 Amendment No. 34 Unit 4



- 1 - Nuclear Operator License
- 2 - Operator License
- 3 - Ray Supervisory Personnel
- 4 - Licensed Personnel per 10 CFR 40

PLANT ORGANIZATION CHART
FIGURE 4.1-1

Amendment No. 42 Unit 3
Amendment No. 34 Unit 4

- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.
- f. At least three (3) persons shall be maintained on site at all times for Fire Emergency response. This excludes two (2) members of the shift crew.

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5, ANSI N18.1-1971 and Appendix A to 10 CFR Part 55.

- 6.4.2 A training program for the Fire Emergency response members shall be maintained under the direction of the Fire Protection Administrator and should meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for fire Brigade training sessions which shall be held at least quarterly. *

6.5 REVIEW AND AUDIT

- 6.5.1 Plant Nuclear Safety Committee PNSC.

6.5.1.1 FUNCTION

The PNSC shall function to advise the Plant Superintendent-Nuclear on all matters related to nuclear safety.

*Effective 90 days from issuance of this amendment.

- h. Any indication of an unanticipated deficiency in some aspect of design or operation of safety related structures, systems, or components.
- i. Reports and meeting minutes of the Plant Nuclear Safety Committee.

6.5.2.8 AUDITS

Audits of facility activities shall be performed under the cognizance of the CNRB. These audits shall encompass:

- a. The conformance of facility operation to all provisions contained within the Technical Specifications and applicable license conditions at least once per year.
- b. The performance, training and qualifications of the entire facility staff at least once per year.
- c. The results of all actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per six months.
- d. The performance of all activities required by the Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per two years.
- e. The Emergency Plans and implementing procedures at least once per two years.
- f. The Security Plan and implementing procedures at least once per two years.
- g. The Facility Fire Protection Program and implementing procedures at least once per two years.

- h. An independent fire protection and loss prevention inspection and audit shall be performed annually utilizing either qualified licensee personnel or an outside fire protection firm.
- i. An inspection and audit of the fire protection and loss prevention program shall be performed by an outside qualified fire consultant at intervals no greater than three (3) years.
- j. Any other area of facility operation considered appropriate by the CNRB or the Executive Vice President.

6.5.2.9 AUTHORITY

The CNRB shall report to and advise the Executive Vice President on those areas of responsibility specified in Section 6.5.2.7 and 6.5.2.8.

6.5.2.10 RECORDS

Records of CNRB activities shall be prepared, approved and distributed as indicated below:

- a. Minutes of each CNRB meeting shall be prepared, approved and forwarded to the Executive Vice President within fourteen days following each meeting.
- b. Reports of reviews encompassed by Section 6.5.2.7.e, f, g and h above, shall be prepared, approved and forwarded to the Executive Vice President within fourteen days following completion of the review.
- c. Audit reports encompassed by Section 6.5.2.8 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within thirty (30) days after completion of the audit.

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:

- a. The REPORTABLE OCCURRENCE shall be reported to the Commission pursuant to the requirements of Section 6.9.
- b. A Reportable Occurrence Report shall be prepared. The report shall be reviewed by the Plant Nuclear Safety Committee.
- c. The Reportable Occurrence Report shall be submitted to the CNFB, the Vice President of Power Resources, and the Commission within the time allotted in Section 6.9.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.
- b. The Safety Limit violation shall be reported immediately to the Commission, the Vice President of Power Resources and to the CNRS.
- c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PNSC. This report shall describe
 - 1) applicable circumstances preceding the violation,
 - 2) effects of the violation upon facility components, systems or structures,
 - and 3) corrective action taken to prevent recurrence.
- d. The Safety Limit Violation Report shall be submitted to the CNFB, the Vice President of Power Resources and the Commission within ten (10) days of the violation.

6.8 PROCEDURES

6.8.1 Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5.1 and 5.3 of ANSI N18.7-1972, Appendix "A" of USNRC Regulatory Guide 1.33, and the Facility Fire Protection Program except as provided in 6.8.2 and 6.8.3 below. This requirement shall be implemented within four (4) months from the effective date of this amendment.

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKETS NOS. 50-250 AND 50-251
FLORIDA POWER AND LIGHT COMPANY
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 42 and 34 to Facility Operating Licenses Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company. These amendments revise the Appendix A Technical Specifications for operation of the Turkey Point Plant Unit Nos. 3 and 4, located in Dade County, Florida. The amendments are effective 60 days after the date of issuance.

The amendments revise the Technical Specifications to incorporate limiting conditions for operation and surveillance requirements for existing fire protection systems and administrative controls.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

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For further details with respect to this action, see (1) the application for amendments dated December 22, 1977, (2) Amendments Nos. 42 and 34 to Licenses Nos. DPR-31 and DPR-41 and (3) the Commission's letter of November 25, 1977 transmitting proposed fire protection Technical Specifications and the related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland this 8th day of November 1978.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors