

DO NOT REMOVE



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

November 4, 1982

Docket Nos. 50-250  
and 50-251

*Posted  
Amndt. 83  
to DPR-41*

Dr. Robert E. Uhrig, Vice President  
Advanced Systems and Technology  
Florida Power and Light Company  
Post Office Box 529100  
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 89 to Facility Operating License No. DPR-31 and Amendment No. 83 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated May 6, 1982.

These amendments change the Technical Specifications to conform with the Commission's Bulletins and Orders Task Force review regarding Auxiliary Feedwater Pump requirements following the Three Mile Island Accident.

The issuance of the enclosed amendments and the Safety Evaluation concludes our review of your responses to the September 21, 1979 letter and TMI Action Items II.E.1.1 and II.E.1.2 are complete.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Daniel G. McDonald, Project Manager  
Operating Reactors Branch #1  
Division of Licensing

Enclosures:

- 1. Amendment No. 89 to DPR-31
- 2. Amendment No. 83 to DPR-41
- 3. Safety Evaluation
- 4. Notice of Issuance

cc w/enclosures:  
See next page

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 89  
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated May 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

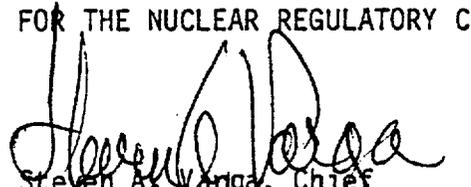
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 89, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Yanga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 4, 1982



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 83  
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated May 6, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;  
and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

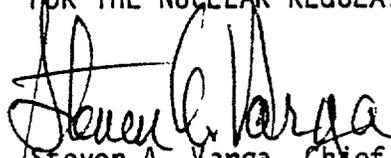
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 83, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 4, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 89 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Page

4.10-1

Insert Page

4.10-1

4.10

AUXILIARY FEEDWATER SYSTEM

Applicability:

Applies to periodic testing requirements of the auxiliary feedwater system.

Objective:

To verify the operability of the Auxiliary Feedwater and its ability to respond properly when required.\*

Specifications:

1. Each turbine-driven auxiliary feedwater pump shall be started at intervals not greater than one month, run for 15 minutes and a flow rate of 600 gpm established to the steam generators. The monthly frequency is not intended to require the test while at cold shutdown. The testing requirement is met by performing this test during startup subsequent to cold shutdown.
2. The auxiliary feedwater discharge valves shall be tested by operator action during pump tests.
3. Steam supply and turbine pressure valves shall be tested during pump tests.
4. These tests shall be considered satisfactory if control panel indication and visual observation of the equipment demonstrate that all components have operated properly.
5. At least once per 18 months:
  - a. Verify that each automatic valve in the flow path actuates to its correct position upon receipt of each auxiliary feedwater actuation test signal.
  - b. Verify that each auxiliary feedwater pump receives a start signal as designed automatically upon receipt of each auxiliary feedwater actuation test signal.

N.A. during cold or refueling shutdowns (only for the Unit at cold or refueling shutdown). The specified tests, however, shall be performed within one surveillance interval prior to starting the turbine.

- \* NOTE: If any local manual realignment of valves is required when operating the Auxiliary Feedwater pumps, a dedicated individual, who is in communication with the control room, shall be stationed at the auxiliary feedwater pump area. Upon instructions from the control room, this operator would realign the valves in the AFW system train to its normal operational alignment.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 89 TO FACILITY OPERATING LICENSE NO. DPR-31  
AND AMENDMENT NO. 83 TO FACILITY OPERATING LICENSE NO. DPR-41  
FLORIDA POWER AND LIGHT COMPANY  
TURKEY POINT PLANT UNIT NOS. 3 AND 4  
DOCKET NOS. 50-250 AND 50-251

I. INTRODUCTION

By letter dated May 6, 1982, the Florida Power and Light Company (the licensee) submitted a request to modify the Technical Specifications for the Turkey Point Plant Unit Nos. 3 and 4. The amendments would change the Technical Specifications to conform to the Bulletins and Orders Task Force review following the Three Mile Island Accident. The requirements of this Task Force regarding Auxiliary Feedwater System were set forth in our letter dated October 16, 1979.

BACKGROUND

The licensee has addressed this issue in their responses dated December 20, 1979, July 22, 1980 and January 14, April 13 and July 23, 1981. We reviewed their responses and issued Amendment Nos. 75 and 69 on December 24, 1981 based on our initial Safety Evaluation (SE). We indicated in our SE that open items remained for which an additional Technical Specification change and information was necessary to complete our review.

The automatic initiation and flow indication, NUREG-0737 Item II.E.1.2, portion of the auxiliary feedwater review were determined to be in compliance with the long term safety grade requirements. The acceptability of this portion of the Auxiliary Feedwater System and the SE were sent to the licensee in a letter from Steven A. Varga dated September 15, 1982.

III. IMPLEMENTATION OF OUR RECOMMENDATIONS

We have completed our review of the licensee's responses dated January 7, May 6 and June 9, 1982 which address the open items identified in our initial SE. The results of the review and implementation of our recommendations are:

A. Recommendation GS-4 - "Emergency procedures for transferring to alternate sources of AFW supply should be available to the plant operators. These procedures should include criteria to inform the operators when, and in what order, the transfer to alternate water sources should take place. The following cases should be covered by the procedures.

1. The case in which the primary water supply is not initially available. The procedures for this case should include any operator actions required to protect the AFW system pumps against self-damage before water flow is initiated; and,
2. The case in which the primary water supply is being depleted. The procedure for this case should provide for transfer to the alternate water sources prior to draining of the primary water supply."

Technical Specifications are in place which insure the minimum condensate storage tank level is 185,000 gallons. This quantity of water is sufficient to maintain a hot standby condition for 15 hours and a subsequent cooldown to 350°F followed by initiation of the residual heat removal system. Alarms warn the operator of low level situations so that make-up flow can be established from alternate water supplies to the condensate storage tank. Procedures are in effect for initiation of makeup from the condenser hotwell, the primary water system or the service water system. Thus, it is insured that water will always be initially available for the AFW pumps.

Following automatic initiation of the auxiliary feedwater pumps, the operator ascertains adequate condensate storage tank level, sufficient flow to the steam generators and notifies the Nuclear Turbine Operator to inspect the pumps for proper operation.

The minimum condensate storage tank level control and the emergency procedures to establish alternate water supplies, insure an adequate quantity of water is available to the pumps, for the case there the condensate storage tank is being depleted.

We conclude that this recommendation is satisfactorily met.

B. Additional Short Term Recommendations

1. Recommendation - "Licensees with plants which require local manual realignment of valves to conduct periodic tests on one AFW system train, and there is only one remaining AFW train available for operation, should propose Technical Specifications to provide that a dedicated individual who is in communication with the control room be stationed at the manual valves. Upon instruction from the control room, this operator would realign the valves in the AFW system train from the test mode to its operational alignment."

By letter dated May 6, 1982, the licensee submitted a proposed amendment to Technical Specification 4.10 which applies to periodic test requirements of the auxiliary feedwater system. The amended Technical Specification requires that:

"If any local manual realignment of valves is required when operating the Auxiliary Feedwater pumps, a dedicated individual, who is in communication with the control room, shall be stationed at the auxiliary feedwater pump area. Upon instructions from the control room, this operator would realign the valves in the AFW system train to its normal operational alignment."

We conclude that the amended Technical Specification fulfills the requirements of the additional short term recommendation, pending approval of it by the Division of Licensing.

- C. Recommendation - "The AFW pump discharge lines and turbine driven AFW pump steam supply lines for each unit combine into single lines through which all water and steam respectively from either unit must flow. A pipe break in either of these single flow paths would cause loss of the capability to provide AFW flow to all the steam generators of one unit. The licensee should evaluate the consequences of a postulated pipe break in these sections of the AFW discharge or steam supply assuming a concurrent single active failure and 1) determine any AFW system modifications or procedures necessary to detect and isolate the break, and direct the required AFW flow to the steam generators before they boil dry or 2) describe how the plant can be brought to a safe shutdown condition by use of other available systems following such a postulated pipe break."

The licensee has developed procedures regarding shutdown through the use of alternate systems and regarding isolation of AFWS steam supply or feedwater line piping breaks.

The proposed AFW pipe modifications, described in the June 7, 1982 letter in response to the above requirement, will provide sufficient redundancy to eliminate the concern of a single pipe break disrupting feedwater flow to the steam generators.

We conclude that this recommendation has been satisfactorily met.

- D. Basis for Auxiliary Feedwater System Flow Requirements

The licensee was requested to supply Auxiliary Feedwater (AFW) design basis information in Enclosure 2 of our October 16, 1979 letter. By letter dated January 7, 1982 the licensee supplied the AFW system flow design bases and criteria.

The licensee verified that the AFW minimum flow rate requirements could be maintained under the following transient conditions:

1. Loss of main feedwater
2. Loss of main feedwater with concurrent loss of offsite AC power
3. Loss of main feedwater with concurrent loss of onsite and offsite AC power
4. Rupture of a main steam line
5. Small break Loss of Coolant Accident (LOCA)
6. Turbine trip with and without turbine bypass valve
7. Main steam isolation valve closure
8. Plant cooldown
9. Startup (Enclosure 3 of June 9, 1982 letter)

Based on our review of the licensee's submittal, we conclude that the AFW system can provide sufficient flow for heat removal requirements following any design basis transient or accident with a concurrent worst case single active failure.

#### E. SUMMARY

We have determined that the licensee has fulfilled all the requirements necessary to conform with the Commission's Bulletins and Orders Task Force review regarding Auxiliary Feedwater Pump requirements following the Three Mile Island Accident.

#### IV. ENVIRONMENTAL CONSIDERATION

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of an accident previously evaluated, do not create the possibility of an accident of a type different from any evaluated previously, and do not involve a significant reduction in a margin of safety, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 4, 1982

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-250 AND 50-251FLORIDA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 89 to Facility Operating License No. DPR-31, and Amendment No. 83 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of Turkey Point Plant, Unit Nos. 3 and 4 (the facilities) located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments change the Technical Specifications to conform with the Commission's Bulletin and Orders Task Force review regarding Auxiliary Feedwater Pump requirements following the Three Mile Island Accident.

The application for amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated May 6, 1982, (2) Amendment Nos. 89 and 83 to License Nos. DPR-31 and DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 4th day of November, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Steven A. Varga, Chief  
Operating Reactors Branch #1  
Division of Licensing