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AUG 11 1981

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice-President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

*Correction Ltr.
to Amdt. 58*

Dear Dr. Uhrig:

On May ¹² 8, 1981 we issued Amendments ⁶⁶ 65 and ⁵⁹ 57, on June 23, 1981 we issued Amendments 68 and 60 and on July 6, 1981 we issued Amendments 70 and 63 for Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. Enclosed are corrected pages 3.0-1, 3.2-3, 3.3-1 and 3.5-1 which were issued in the above amendments and which contained administrative errors. Please replace the above pages in your Technical Specifications with the corrected pages enclosed.

Sincerely,

Original signed by:
S. A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure: As Stated

cc w/enclosure:
See next page

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Robert E. Uhrig
Florida Power and Light Company

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3.0 LIMITING CONDITIONS FOR OPERATION

Applicability

- 3.0.1 If one of the below listed limiting conditions for operation can not be satisfied because fewer components are operable than are required, the unit shall be placed in hot shutdown within seven hours and cold shutdown within the following 30 hours unless appropriate corrective action is taken before the time expires. This specification applies only to specifications 3.3.1, 3.4.1.b, 3.4.2.b, 3.4.3.b, 3.4.4.b, 3.4.5.b, 3.5, 3.6.d, 3.7.2 and 3.13.3.
- 3.0.2 For purposes of determining if a component is operable for LCO considerations, the component need not be considered inoperable due to inoperability of its normal or emergency power supply if all of its redundant components are operable with their normal or emergency power supplies operable.

reactivity insertion upon injection greater than 0.3 $\Delta k/k$ at rated power. Inoperable rod worth shall be determined within 4 weeks.

b. A control rod shall be considered inoperable if

- (1) the rod cannot be moved by the CRDM, or
- (2) the rod is misaligned from its bank by more than 12 steps, or
- (3) the rod drop time is not met.

c. If a control rod cannot be moved by the drive mechanism, shutdown margin shall be increased by boron addition to compensate for the withdrawn worth of the inoperable rod.

5. CONTROL ROD POSITION INDICATION

If either the power range channel deviation alarm or the rod deviation monitor alarm are not operable, rod positions shall be logged once per shift and after a load change greater than 10% of rated power. If both alarms are inoperable for two hours or more, the nuclear overpower trip shall be reset to 93% of rated power.

6. POWER DISTRIBUTION LIMITS

a. Hot channel factors:

With steam generator tube plugging $\leq 28\%$, the hot channel factors (defined in the basis) must meet the following limits at all times except during low power physics tests:

$$F_q(Z) \leq (2.125/P) \times K(Z), \text{ for } P > .5$$

$$F_q(Z) \leq (4.25) \times K(Z), \text{ for } P \leq .5$$

$$F_{\Delta H}^{N} \leq 1.55 [1. + 0.2 (1-P)]$$

Where P is the fraction of rated power at which the core is operating; K(Z) is the function given in Figure 3.2-3; Z is the core height location of F_q .

If F_q , as predicted by approved physics calculations, exceeds 2.125 the power will be limited to the rated power multiplied by the ratio of 2.125 divided by the predicted F_q , or augmented surveillance of hot channel factors shall be implemented.

3.3 CONTAINMENT

Applicability: Applies to the integrity of the containment.

Objective: To define the operating status of the containment

Specification: 1. CONTAINMENT INTEGRITY

- a. The containment integrity (as defined in 1.5) shall not be violated unless the reactor is in the cold shutdown condition. Specification 3.0.1 applies to 3.3.1.
- b. The containment integrity shall not be violated when the reactor vessel head is removed unless the reactor is in the refueling shutdown condition.

2. INTERNAL PRESSURE

If the internal pressure exceeds 3 psig or the internal vacuum exceeds 2 psig, the condition shall be corrected within 8 hours or the reactor shall be brought to hot shutdown.

3. CONTAINMENT ISOLATION VALVES

With $K_{eff} \geq 0.99$, % thermal power excluding decay heat ≥ 0 , and an average coolant temperature $T_{avg} \geq 200F^{\circ}$, the following conditions shall be met:

The containment isolation valves for Phase containment isolation, Phase B containment isolation, and Containment Ventilation Isolation shall be operable with the isolation times of each power operated or automatic valve within the limits established for testing in accordance with Section XI of ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50.55a(g)(6)(i), or the valve is closed.

3.5 INSTRUMENTATION

Applicability: Applies to reactor safety and features and accident monitoring instrumentation systems.

Objective: To delineate the conditions of the instrumentation and safety circuits necessary to ensure reactor safety.

- Specification:
1. Tables 3.5-1 through 3.5-5 state the minimum instrumentation operation conditions. Specification 3.0.1 applies to Tables 3.5-1 through 3.5-3.
 2. With the number of OPERABLE accident monitoring instrumentation channel(s) less than the Total Number of Channels shown in Table 3.5-5, either restore the inoperable channel(s) to OPERABLE status within 7 days, or be in a condition with $K_{eff} < 0.99$, % thermal power excluding decay heat equal to zero, and an average coolant temperature $T_{avg} < 350^{\circ}\text{F}$ within the next 12 hours.
 3. With the number of OPERABLE accident monitoring instrumentation channels less than the MINIMUM CHANNELS OPERABLE requirements of Table 3.5-5, either restore the inoperable channel(s) to OPERABLE status within 48 hours or be in a condition with $K_{eff} < 0.99$, % thermal power excluding decay heat equal to zero, and an average coolant temperature $T_{avg} < 350^{\circ}\text{F}$ within the next 12 hours.