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MAR 0 8 1982

Docket Nos. 50-250
and 50-251

Posted
Amdt. 73
to DPR-41

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

(letter dated 10 [illegible] 82 & proprietary supporting [illegible])

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 79 to Facility Operating License No. DPR-31 and Amendment No. 73 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The Amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated March 10, 1981.

These amendments change the Technical Specifications to define the Reactor Coolant System Pressure Boundary integrity and to provide an alternate means of increasing assurance of proper valve position. In addition, certain administrative corrections have been made to the Technical Specifications.

During our review we found that certain modifications were necessary. We have discussed these modifications with your staff. They agreed to the modifications which have been incorporated in these amendments.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED
Marshall Grotenhuis, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Amendment No. 79 to DPR-31
2. Amendment No. 73 to DPR-41
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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SURNAME	CParrish	MGrotenhuis	SVarga	TNovak			
DATE	02/24/82	02/25/82:ds	02/25/82	02/25/82	03/1/82		

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Robert E. Uhrig
Florida Power and Light Company

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 79
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated March 10, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 79 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 8, 1982

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 73
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated March 10, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 73 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 8, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages	Insert Pages
i	i
1-6	1-6
Table 3.5-2	Table 3.5-2
3.15-1	3.15-1
4.16-1	4.16-1
B3.15-1	B3.15-1

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1.16 REACTOR COOLANT PUMPS

The reactor shall not be operated with less than three reactor coolant pumps in operation.

1.17 LOW POWER PHYSICS TESTS

Low power physics tests are tests below a nominal 5% of rated power which measure fundamental characteristics of the reactor core and related instrumentation.

1.18 ENGINEERED SAFETY FEATURES

Features such as containment, emergency core cooling, and containment atmospheric cleanup systems for mitigating the consequences of postulated accidents.

1.19 REACTOR PROTECTION SYSTEM

Systems provided to act, if needed, to avoid exceeding a safety limit in anticipated transients and to activate appropriate engineered safety features as necessary.

1.20 SAFETY RELATED SYSTEMS AND COMPONENTS

Those plant features necessary to assure the integrity of the reactor coolant pressure boundary, the capability to shutdown the reactor and maintain it in a safe shutdown condition, or the capability to prevent or mitigate the consequences of accidents which could result in off-site exposures comparable to the guideline exposures of 10 CFR 100.

1.21 PER ANNUM

During each calendar year.

1.22 REACTOR COOLANT SYSTEM PRESSURE BOUNDARY INTEGRITY

For purposes of low temperature RCS overpressure protection, the RCS will have pressure boundary integrity UNLESS the RCS is open to containment and the minimum area of the RCS opening is greater than 2.20 square inches.

TABLE 3.5-2

ENGINEERED SAFETY FEATURES ACTUATION

NO.	FUNCTIONAL UNIT	1 MIN. OPERABLE CHANNELS	2 MIN. DEGREE OF REDUNDANCY	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
1.	SAFETY INJECTION			
T.1	Manual	1	0	Cold Shutdown
1.2	High Containment Pressure	2	1	Cold Shutdown
1.3	High Differential Pressure between any Steam Line and the Steam Line Header	2	1	Cold Shutdown
1.4	Pressurizer Low Pressure*	2	1	Cold Shutdown
1.5	High Steam Flow in 2/3 Steam Lines with Low T_{avg} or Low Steam Line Pressure	1/line in each of 2 lines	1	Cold Shutdown
2.	CONTAINMENT SPRAY			
2.1	High Containment Pressure and High-High Containment Pressure (Coincident)	2 per set	1/set	Cold Shutdown
3.	AUXILIARY FEEDWATER			
3.1	Low-Low Steam Generator Level	2	1	Hot Shutdown
3.2	Loss of Voltage (both 4KV busses)	2	0	Cold Shutdown
3.3	Safety Injection	(---See 1 above---)		
3.4	Trip of both Main Feedwater Pump Breakers	2	0	Cold Shutdown

* This signal may be manually bypassed, when the reactor is shut down and pressure is below 2000 psig.

3.15 OVERPRESSURE MITIGATING SYSTEM

Applicability: Establishes operating limitations to assure that the limits of 10 CFR 50, Appendix G, are not exceeded.

Objectives: To minimize the possiblity of an overpressure transient which could exceed the limits of 10 CFR 50, Appendix G.

Specification:

1. At RCS temperature less than or equal to 380°F and with RCS pressure boundary integrity, valves MOV-*-843A, MOV-*-843B and MOV-*-869 shall be closed and their breakers racked out.

2. If any of the valves listed in 3.15.1 are found to be open when required to be closed by 3.15.1, perform at least one of the following within the next 8 hours:

   a. block the corresponding flow path to the reactor vessel, or
   b. close the valve, or
   c. depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or
   d. verify at least one pressurizer power operated relief valve is maintained open.

3. At RCS temperature less than or equal to 275°F with RCS pressure boundary integrity established, two pressurizer power operated relief valves shall be operable with a setpoint of 415 psig $\pm$ 15 psi.

   a. If one power operated relief valve required by 3.15.3 is inoperable, perform at least one of the following within 7 days:

      (1) restore operability of the power operated relief valve, or

      (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or

      (3) verify at least one presssurizer power operated relief valve is maintained open.

   b. If both power operated relief valves required by 3.15.3 are inoperable, perform at least one of the following within the next 24 hours:

      (1) restore operability of at least one power operated relief valve, or

      (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or

      (3) verify at least one pressurizer power operated relief valve is maintained open.

4.16 OVERPRESSURE MITIGATING SYSTEM

Applicability: Applies to periodic surveillance of the Overpressure Mitigating System.

Objective: To demonstrate operability of the Overpressure Mitigating System.

Specification:

1. Within 1 month prior to operation in a condition where the PORV would be required to be operable per 3.15, the pressurizer power operated relief valve actuation circuitry shall be functionally tested. The functional test need not include actual valve operation.

2. While valves MOV-*-843A, MOV-*-843B and MOV-*-869 are required to be closed by Specification 3.15.1, they shall be verified closed daily.

3. While the pressurizer power operated relief valves are required to be operable by Specification 3.15.3, verify weekly that the isolation valve for each operable pressurizer power operated relief valve is open.

4. While the pressurizer power operated relief valves are required to be operable by Specification 3.15.3, the operable pressurizer power operated relief valve actuation circuitry shall be functionally tested monthly. The functional test need not include actual valve operation.

5. While the pressurizer power operated relief valves are required to be operable by specification 3.15.3, the operational readiness of the backup air supply corresponding to the operable pressurizer power operated relief valve shall be verified daily.

6. Testing shall be in accordance with approved plant procedures.

B3.15 BASES FOR LIMITING CONDITIONS OF OPERATION, OVERPRESSURE MITIGATING SYSTEM

The operability of two PORV's or an RCS vent opening of greater than or equal to 2.20 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the RCS cold legs are $\leq$ 275°F. Either PORV has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCP with the secondary water temperature of the steam generator $\leq$ 50°F above the RCS cold leg temperature (includes margin for instrument error) or (2) the start of a HPSI pump and its injection into a water solid RCS.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 79 TO FACILITY OPERATING LICENSE NO. DPR-31

AND AMENDMENT NO. 73 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT PLANT UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

Introduction

By letter dated March 10, 1981, the Florida Power and Light Company (the licensee) requested amendments to Facility Operating License No. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4. The amendments would change the Technical Specification to define the Reactor Coolant System Pressure Boundary and to provide an alternate means of increasing assurance of proper valve position. In addition, administrative errors are corrected in the Table of Contents and Table 3.5-2. Page i had certain items left out and Table 3.5-2 was incorrectly modified in Amendment Nos. 73 and 67, issued November 4, 1981. The pressure in the footnote was inadvertently changed from 2000 psig to 1800.

During our review certain modifications to the amendment request were found necessary. We discussed these modifications with the licensee. He found the modifications acceptable and they have been incorporated in these amendments.

Background

In the letter dated March 10, 1981, the licensee proposed technical specification changes for the overpressure mitigating system. This change addresses the overpressure mitigating system Technical Specification changes proposed by FPL by letter dated October 18, 1977 and included as amendments to Facility Operating Licenses DPR-31 and DPR-41 by Amendment Nos. 55 and 47, respectively, issued on March 14, 1980.

Evaluation

The proposed modifications are discussed below:

(1) T.S. 1.18
This is a definition of the reactor coolant system pressure boundary integrity. This definition is editorial in nature, adds to the clarity of the Technical Specifications, and is acceptable.

(2) T.S. 3.15
This Technical Specification is modified so that MOV-*-869 is closed and it's power removed when RCS temperature is $\leq$ 380°F. Power to this valve is removed in lieu of removing the power from the operators of MOV-*-866 and MOV-*-866B. MOV-*-869 is on the 3" line connecting the 4 High Head Safety Injection pumps to the RCS hot legs. This 3" line forks in two 2" lines leading to RCS loops A and B hot legs. MOV-*866 A and B valves are on the two 2" lines.

This Technical Specification modification proposes not removing the power from the 866 valves' operators, however they will remain closed, and proposes instead removing power from the 869 valve. The reason for this modification is operational convenience, since maintaining the power to the 866 valves' operators will also maintain valve position indications available in the control room for the two valves. This will enable the operator to conduct valve position surveillance without having to enter the containment. We concur with the licensee's argument and conclude that the closure and power removal to the 869 valve on one hand, and closure and control room valve position indication of the parallel valves 866 A & B on the other hand, provide adequate isolation of the hot leg injection paths to the RCS.

(3) T.S. 4.16
This is a new Technical Specification that provides the surveillance specification on the pressurizer PORV's backup air supply.

The licensee proposed to verify the operational readiness of the backup air supply daily. We find this surveillance frequency acceptable.

(4) T.S. B3.15
A typographical error has been corrected.

We note that the staff's evaluation included with above noted license amendments addressed setpoint analysis in Section 3.5 wherein it identified the relief valve setpoint as 415 psig. In Section 3.15, paragraph 3, the specification states that "at RCS temperatures of less than or equal to 275°F with RCS pressure boundary integrity established, two pressurizer power operated relief valves shall be operable at the low setpoint range." We find that this statement is vague with respect to the required setpoint and not consistent with the degree of specificity required in technical specifications. Paragraph 3 of Section 3.15 should be changed to read: "...shall be operable with a setpoint of 415 $\pm$15 psig."

These values were provided in the licensee's analysis of the system which formed the staff acceptance of the design in the SER issued on March 14, 1980.

We conclude that the proposed Technical Specification 3.15.3 is acceptable as modified with the addition of the setpoint value noted above.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 8, 1982

Principal Contributors:
S. Diab
T. Dunning
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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 79 to Facility Operating License No. DPR-31, and Amendment No. 73 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of Turkey Point Plant, Unit Nos. 3 and 4 (the facilities) located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments change the Technical Specifications to define the Reactor Coolant System Pressure Boundary integrity and to provide an alternate means of increasing assurance of proper valve position. In addition, certain administrative corrections have been made to the Technical Specifications.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated March 10, 1981, (2) Amendment Nos. 79 and 73 to License Nos. DPR-31 and DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 8th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing