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DBrinkman
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Docket Nos. 50-250
and 50-251

Posted
Amdt. 75
to DPR-41

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 81 to Facility Operating License No. DPR-31 and Amendment No. 75 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated December 18, 1981. We have found it necessary to modify parts of the amendment request. We have discussed the modifications with your staff. They have found the modifications acceptable and they have been incorporated.

These amendments incorporate steam generator inservice inspection requirements into the plant Technical Specifications. These specifications replace the Facility Operating License No. DPR-31 condition 3.E for Unit 3, effective as of the date of this amendment; and the Facility Operating License No. DPR-41 condition 3.D for Unit 4 effective as of the date of completion of the steam generator replacement in Unit 4.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

ORIGINAL SIGNED

Marshall Grotenhuis, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 81 to DPR-31
2. Amendment No. 75 to DPR-41
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated December 18, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

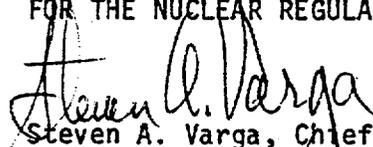
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Paragraph 3.E is hereby deleted in its entirety.
4. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 24, 1982



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 75
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated December 18, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 75, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. Paragraph 3.D is deleted in its entirety.
4. This license amendment is effective as of the date that the steam generator repair is completed.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 24, 1982

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
3.1-4	3.1-4
3.1-5	3.1-5
-----	4.2-3
-----	4.2-4
-----	4.2-5
-----	4.2-6
-----	Table 4.2-2
-----	Table 4.2-3
B4.2-13	B4.2-13
-----	B4.2-14

3. LEAKAGE

- a. Any reactor coolant system leakage indication in excess of 1 gpm shall be the subject of an investigation and evaluation initiated within 4 hours of the indication (ex. water inventory changes, radiation level increases, visual or audible indication). A leak shall be assumed to exist until it is determined that no unsafe condition exists and that the indicated leak cannot be substantiated. Leakage of reactor coolant through reactor pump seals and system valves to connecting closed systems from which coolant can be returned to the reactor coolant system shall not be considered as leakage except that such losses shall not exceed 30 gpm.
- b. If a reactor coolant system leakage indication is proven real, and is not evaluated as safe, or exceeds 10 gpm, reactor shutdown shall be initiated within 24 hours of the initial indication, except as noted in Section 3.1.3.g.
- c. If reactor coolant leakage exists through a fault in the system boundary that cannot be isolated (ex. vessels, piping, valve bodies) the reactor shall be shutdown, and cool down to cold shutdown shall be initiated within 24 hours.
- d. The safety evaluation shall consider the source and magnitude of the leak, rates of change of detection variables, and if shutdown is required this evaluation shall be used to determine shutdown rates and conditions. A written log of the action taken shall be made as soon as practicable. The evaluation shall assure that no potential gross leak is developing and that potential release of activity will be within the guidelines of 10CFR20.

- e. After shutdown, corrective action shall be taken before operation is resumed.
- f. Above 2% of rated power, two leak detection systems of different principles shall be operable, one of which is sensitive to radioactivity. The latter may be out of service for 48 hours provided two other systems are operable.
- g. Reactor Coolant System leakage shall be limited to 1 gpm total primary-to-secondary leakage through all steam generators not isolated from the Reactor Coolant System and 500 gallons per day through any one steam generator not isolated from the Reactor Coolant System.

4. MAXIMUM REACTOR COOLANT ACTIVITY

The total specific activity of the reactor coolant due to nuclides with half-lives of more than 30 minutes, excluding tritium, shall not exceed $135/\bar{E}^*$ $\mu\text{Ci/cc}$ whenever the reactor is critical or average reactor coolant temperature is greater than 500F. The concentration of radioiodine in the reactor coolant shall be limited to 1.0 microcurie/gram during normal operation and to 30 microcuries/gram during power transients.

If the limit above is not satisfied, the reactor shall be shutdown and cooled to 500F or less within 6 hours.

* \bar{E} is the average of beta and gamma energy (Mev) per disintegration of the specific activity.

4.2.5 STEAM GENERATOR INSPECTION

4.2.5.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.2-2.

4.2.5.2 Steam Generator Tube Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.2-3. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.2.5.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.2.5.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators; the tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.
- b. The sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
 1. All nonplugged tubes that previously had detectable wall penetrations (greater than 20%), and
 2. Tubes in those areas where experience has indicated potential problems.
 3. A tube inspection (pursuant to Specification 4.2.5.4.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.
- c. The tubes selected as the second and third samples in the inservice inspection may be less than a full tube inspection by concentrating (selecting at least 50% of the tubes to be inspected) the inspection on those areas of the tube sheet array and on those portions of tubes where tubes with imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three categories:

Category

Inspection Results

C-1

Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.

Catetory

Inspection Results

C-2

One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.

C-3

More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

NOTE: In all inspections, previously degraded tubes must exhibit significant (greater than 10%) further wall penetrations to be included in the above percentage calculations.

4.2.5.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after six effective full power months of operation but within 24 calendar months following replacement of steam generators. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following preservice under AVT conditions, not including the results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the inservice inspection of a steam generator conducted in accordance with Table 4.2-3 requires a third sample inspection whose results fall in Category C-3, the inspection frequency shall be reduced to at least once per 20 months. The reduction in inspection frequency shall apply until a subsequent inspection demonstrates that a third sample inspection is not required.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.2-3 during the shutdown subsequent to any of the following conditions.

1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.1.3.g.
2. A seismic occurrence greater than the operating Basis Earthquake (OBE).
3. A loss-of-coolant accident resulting in rapid depressurization of the primary system, or
4. A main steam line or feedwater line break resulting in rapid depressurization of the affected steam generator.

4.2.5.4 Acceptance Criteria

a. As used in this Specification:

1. Imperfection means an exception to the dimensions, finish or contour of a tube from that required by fabrication drawings or specifications. Eddy-current testing, indications below 20% of the nominal tube wall thickness, if detectable may be considered as imperfections.
2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube.
3. Degraded Tube means a tube containing imperfections greater than or equal to 20% of the nominal wall thickness caused by degradation.
4. % Degradation means the percentage of the tube wall thickness affected or removed by degradation.
5. Defect means an imperfection of such severity that it exceeds the plugging limit. A tube containing a defect is defective. Any tube which does not permit the passage of the eddy-current inspection probe shall be deemed a defective tube.
6. Plugging Limit means the imperfection depth at or beyond which the tube shall be removed from service because it may become unserviceable prior to the next inspection and is equal to 40% of the nominal tube wall thickness.
7. Unserviceable describes the condition of a tube if it leaks or contains a defect large enough to affect its structural integrity in the event of OBE, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.2.5.3.c, above.

8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg.
 9. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service to establish a baseline condition of the tubing.
- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug all tubes exceeding the plugging limit and all tubes containing through-wall cracks) required by Table 4.2-3.

4.2.5.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged in each steam generator shall be reported to the Commission within 15 days.
- b. The complete results of the steam generator tube inservice inspection shall be included in the Annual Changes, Tests and Experiment Reports for the period in which this inspection was completed. This report shall include:
 1. Number and extent of tubes inspected.
 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 3. Identification of tubes plugged.
- c. Results of steam generator tube inspections which fall into Category C-3 and require prompt notification of the Commission shall be reported pursuant to Specification 6.9^a prior to resumption of plant operation. The written followup of this report shall provide a description of investigations conducted to determine the cause of the tube degradation and corrective measures taken to prevent recurrence.

TABLE 4.2-2

MINIMUM NUMBER OF STEAM GENERATORS TO BE
INSPECTED DURING INSERVICE INSPECTION

Preservice Inspection	No			Yes		
	Two	Three		Two	Three	
No. of Steam Generators per Unit						
First Inservice Inspection	All			One	Two	
Second & Subsequent Inservice Inspections	One ¹			One ¹	One ²	

Table Notation:

1. The inservice inspection may be limited to one steam generator on a rotating schedule encompassing 3 N % of the tubes (where N is the number of steam generators in the plant) if the results of the first or previous inspections indicate that all steam generators are performing in a like manner. Note that under some circumstances, the operating conditions in one or more steam generators may be found to be more severe than those in other steam generators. Under such circumstances the sample sequence shall be modified to inspect the most severe conditions.
2. The other steam generator not inspected during the first inservice inspection shall be inspected. The third and subsequent inspections should follow the instructions described in 1 above.

TABLE 4.2-3
STEAM GENERATOR TUBE INSPECTION

1ST SAMPLE INSPECTION			2ND SAMPLE INSPECTION		3RD SAMPLE INSPECTION	
Sample Size	Result	Action Required	Result	Action Required	Result	Action Required
A minimum of S Tubes per S. G.	C-1	None	N/A	N/A	N/A	N/A
	C-2	Plug defective tubes and inspect additional 2S tubes in this S. G.	C-1	None	N/A	N/A
			C-2	Plug defective tubes and inspect additional 2S tubes in this S. G.	C-1	None
			C-3	Perform action for C-3 result of first sample	C-2	Plug defective tubes
			C-3	Perform action for C-3 result of first sample	C-3	Perform action for C-3 result of first sample
	C-3	Inspect all tubes in this S. G., plug de- fective tubes and inspect 2S tubes in each other S. G. Prompt notification to NRC pursuant to specification 6.9.2. a.	All other S. G.s are C-1	None	N/A	N/A
			Some S. G.s C-2 but no additional S. G. are C-3	Perform action for C-2 result of second sample	N/A	N/A
			Additional S. G. is C-3	Inspect all tubes in each S. G. and plug defective tubes. Prompt notification to NRC pursuant to specification 6.9.2. a.	N/A	N/A

$S = 3 \frac{N}{n} \%$ Where N is the number of steam generators in the unit, and n is the number of steam generators inspected during an inspection

by a factor of 2.1. Thus, this capsule provides information for approximately a four-year exposure to the vessel.

Capsule No. 2 is scheduled to be removed at the fourth region replacement. This capsule leads the vessel maximum exposure by a factor of 0.8 and thus will provide data for a four-year exposure to the vessel. This sample also contains weld metal which is not present in Capsule No. 1.

Capsule No. 3 leads the vessel maximum exposure by a factor of 2.2 and is scheduled to be removed after twenty years. Thus, sample No. 3 will provide data for an exposure to the vessel of approximately forty years.

Capsules No. 4 and 5 lead the maximum vessel exposure by factors of 0.7 and 0.5, respectively. Thus, Capsule No. 4, which is scheduled to be removed after thirty years, provides data for a vessel exposure of twenty-one years and Capsule No. 5, which is scheduled to be removed at forty years, provides data for a vessel exposure of twenty years.

In addition to the capsules discussed above, there are three spares.

Item 7.3 Steam Generator Tube Inspection

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those parameter limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these parameter limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system (primary-to-secondary leakage = 1 gallon per minute, total). Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operating plants have demonstrated that primary-to-secondary leakage of 1 gallon per minute can readily be detected by radiation monitors of steam generator blowdown. Leakage in excess of this limit will require plant shutdown and an unscheduled inspection, during which the leaking tubes will be located and plugged.

Wastage-type defects are unlikely with the all volatile treatment (AVT) of secondary coolant. However, even if a defect of similar type should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging will be required of all tubes with imperfections exceeding the plugging limit which, by the definition of Specification 4.2.5.4.a is 40% of the tube nominal wall thickness. Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be promptly reported to the Commission pursuant to Specification 6.9.2a prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 75 TO FACILITY OPERATING LICENSE NO. DPR-41
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT PLANT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

Introduction

By letter dated December 18, 1981, the Florida Power and Light Company (the licensee) requested amendments to Facility Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The amendments update the Technical Specifications to include steam generator inspection requirements. These inspection requirements will replace the Facility Operating License No. DPR-31 conditions; paragraph 3.E.1 through 3.E.7 for Unit 3, effective with this amendment and Facility Operating License No. DPR-41 condition; paragraph 3.D.1 through 3.D.9 for Unit 4, effective as of the date of replacement of the steam generators of Unit 4.

We found it necessary to modify as follows the amendment request:

1. The first tube sample selection needed clarification (4.2.5.2.b).
2. A tube inspection requirements was inadvertently omitted (4.2.5.b.3).
3. The second and third tube sample selection needed clarification (4.2.5.2.c).
4. The first inspection interval needed clarification (4.2.5.3.a).

We discussed these modifications with the licensee staff and with their agreement the modifications have been incorporated.

Evaluation

The amendment request was evaluated by Idaho Engineering Laboratory, under a contract to NRC. Their Technical Evaluation Report (TER) noted modifications 1 and 4 above. We accept the evaluation performed by the Idaho Engineering Laboratory and incorporated it into this Safety Evaluation Report by reference. In addition, we noted modifications 2 and 3 above.

Item 1 is a clarification of T/S 4.2.5.2.b which indicates that the requirement involves the first "sample of tubes selected" rather than "the first inservice inspection."

Item 2 inserts T/S 4.2.5.2.b.3 which was inadvertently omitted.

Item 3 is a clarification of T/S 4.2.5.2.c which indicates that the requirement involves the second and third "sample selection" rather than the "inservice inspection."

Item 4 is a clarification of T/S 4.2.5.3.a which indicates the inspections required following completion of the steam generator repair program.

We have reviewed these Technical Specifications and find that they meet the requirements for steam generator inservice inspection as expressed by the Standard Technical Specifications (NUREG-0452 Revision 4 Fall 1981). This completes the requirements for steam generator inservice inspection which was requested by the NRC letter dated September 27, 1974. This requirement was satisfied during the interim period by special license requirements when the denting of the Turkey Point steam generators was discovered.

Based on our evaluation, which incorporates the TER prepared by the Idaho Engineering Laboratory, and the modifications listed above, we conclude that the proposed amendments are acceptable.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant decrease in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is a reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 24, 1982

Principal Contributors:

E. Murphy

M. Grotenhuis

INTERIM REPORT

Accession No. _____
Report No. EGG-EA-5743

Contract Program or Project Title:
Selected Operating Reactor Issues Program (III)

Subject of this Document:
Technical Specifications for the Inspection of Steam Generators,
Turkey Point, Unit Nos. 3 and 4

Type of Document:
Technical Evaluation Report

Author(s):
J. A. Steverson

Date of Document:
January 1982

Responsible NRC Individual and NRC Office or Division:
J. N. Donohew, Division of Licensing

This document was prepared primarily for preliminary or internal use. It has not received full review and approval. Since there may be substantive changes, this document should not be considered final.

EG&G Idaho, Inc.
Idaho Falls, Idaho 83415

Prepared for the
U.S. Nuclear Regulatory Commission
Washington, D.C.
Under DOE Contract No. DE-AC07-76ID01570
NRC FIN No. A6429

INTERIM REPORT

TECHNICAL SPECIFICATIONS FOR THE INSPECTION OF STEAM GENERATORS

TURKEY POINT, UNIT NOS. 3 AND 4

Docket Nos. 50-250, 50-251

January 1982

by

J. A. Steverson
Reliability and Statistics Branch
Engineering Analysis Division
EG&G Idaho, Inc.

ABSTRACT

This report reviews the Turkey Point, Unit Nos. 3 and 4 technical specifications for steam generator inspection.

FOREWORD

This report is supplied as part of the "Selected Operating Reactor Issues Program (III)" being conducted for the U.S. Nuclear Regulatory Commission, Office of Nuclear Reactor Regulation, Division of Licensing, by EG&G Idaho, Inc., Reliability and Statistics Branch.

The U.S. Nuclear Regulatory Commission funded the work under the authorization, B&R 20 19 02 06, FIN No. A6429.

TECHNICAL EVALUATION REPORT

TECHNICAL SPECIFICATIONS FOR THE INSPECTION OF STEAM GENERATORS TURKEY POINT, UNIT NOS. 3 AND 4

1.0 INTRODUCTION

Turkey Point steam generators have been inspected according to special license conditions imposed originally by Amendment 20, dated December 3, 1976, for Unit 4 and by Amendment 22, dated January 14, 1977, for Unit 3. Thus, technical specifications for the inservice inspection of the steam generator tubes were not required. By letter, dated July 1, 1981,¹ the NRC requested Turkey Point 3 and 4 review their submittals and appropriately modify them, using as a guide the technical specifications for steam generator inspection provided by the NRC.

On December 18, 1981, Florida Power and Light (FP&L) submitted proposed changes to their technical specifications (TS).² The following discussion evaluates the proposed Turkey Point TS and notes any differences existing between them and the NRC model TS.

2.0 DISCUSSION AND EVALUATION

Turkey Point, Units 3 and 4 are three loop Westinghouse PWR Plants. The specific sections of the Westinghouse Standard Technical Specifications³ that apply to this task are as follows:

3/4.4 REACTOR COOLANT SYSTEM

3/4.4.6 STEAM GENERATORS

3/4.4.7 REACTOR COOLANT SYSTEM LEAKAGE

The licensee proposed the following changes and incorporations to their technical specifications. First, Specification 3.1.3.6 was revised and 3.1.3.g was added to incorporate limiting conditions for operation regarding primary-to-secondary leakage through steam generator tubes. This satisfies requirements 3.4.7.2.c and 3.4.7.2.d in the NRC standard TS. Secondly, a

steam generator inspection program was added to the licensee's section on Reactor Coolant System Inservice Inspection. The program is in Section 4.2.5, Steam Generator Inspection, and in Tables 4.2-2 and 4.2-3 of their proposed TS and coincides with Section 3/4.4.6, Steam Generators, and with Tables 4.4-1 and 4.4-2 of the Westinghouse Standard TS.

Also in the steam generator inspection program, the Westinghouse Standard TS in Section 4.4.6.3.a, Inspection Frequencies, require the first inservice inspection to be performed after six effective full power months but within 24 calendar months of initial criticality. The licensee's corresponding requirement in Section 4.2.5.3.a, requires the first inservice inspection to be performed after the first refueling following initial criticality or replacement of steam generators (whichever is later).

3.0 CONCLUSION

The proposed TS submitted by FP&L for Turkey Point, Units 3 and 4 included revisions and/or additions regarding (1) primary-to-secondary leakage through steam generator tubes, and (2) a steam generator inspection program. A basis for the steam generator inspection was also added to the specification.

The TS proposed by Turkey Point are in agreement with the Westinghouse Standard TS with two exceptions. The preceding evaluation noted (1) an omission in the Turkey Point TS concerning the inspection of the first sample of steam generator tubes, and (2) a difference in the timing of the first inservice inspection of a steam generator.

4.0 REFERENCES

1. NRC letter, S. A. Varga, to FP&L, R. E. Uhrig, dated July 1, 1981.
2. FP&L letter, R. E. Uhrig, to NRC, D. G. Eisenhut, dated December 18, 1981.
3. Standard Technical Specifications for Westinghouse PWRs, NUREG-0452, Rev. 3, Fall 1980.

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-250 AND 50-251FLORIDA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 81 to Facility Operating License No. DPR-31, and Amendment No. 75 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of Turkey Point Plant, Unit Nos. 3 and 4 (the facilities) located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments incorporate the steam generator inservice inspection requirements into the Technical Specifications. These specifications replace Facility Operating License No. DPR-31 condition 3.E for Unit 3 effective as of the date of this amendment and Facility Operating License No. DPR-41 condition 3.D for Unit 4 effective as of the date of completion of the steam generator replacement in Unit 4.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments has not required since the amendments do not involve a significant hazards consideration.

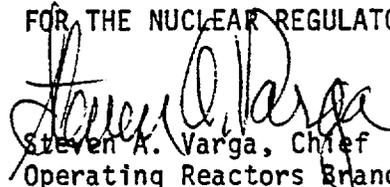
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The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated December 18, 1981, (2) Amendment Nos. 81 and 75 to License Nos. DPR-31 and DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 24th day of March, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing