DISTRIBUTION
Docket Files 50-250 and 50-251
NRC PDRs (2)
Local PDR
TERA
NSIC
NRR Reading
ORB1 Reading
D. Eisenhut
T. Novak
S. Varga
M. Grotenhuis
C. Parrish
I&E (5)
B. Jones (8)
ACRS (16)
B. Scharf (10)
H. Denton
C. Miles
R. Diggs
A-ttorney, O-ELD

MAY 1 1980

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

On March 14, 1980 we issued Amendment Nos. 55 and 47 to Facility Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4 respectively. After the amendments were issued an error was discovered in the numbering of the pages. As a consequence, we are reissuing the "Attachment to License Amendments" with the corrected Technical Specification pages. Pages iii-a, 3.1-1, B3.1-1 and B4.15-1 had no errors but are being reissued for administrative convenience.

Sincerely,

Original Signed By

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As Stated

cc:- w/enclosures
See next page

8005260390

OFFICE	DORORB1	DL:ORB1	DL:ORB1			
SURNAME	MGrotenhuis:jb	CSParrish	SAVarga			
DATE	04/ /80	/ /80	/ /80			

NRC FORM 318 (9-76) NRCM 0240 ★ U.S. GOVERNMENT PRINTING OFFICE: 1976 - 265 - 769

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

MAY 1 1980

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

On March 14, 1980 we issued Amendment Nos. 55 and 47 to Facility Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4 respectively. After the amendments were issued an error was discovered in the numbering of the pages. As a consequence, we are reissuing the "Attachment to License Amendments" with the corrected Technical Specification pages. Pages iii-a, 3.1-1, B3.1-1 and B4.15-1 had no errors but are being reissued for administrative convenience.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As Stated

cc: w/enclosure
See next page

Robert E. Uhrig
Florida Power and Light Company

cc: Mr. Robert Lowenstein, Esquire
Lowenstein, Newman, Reis and Axelrad
1025 Connecticut Avenue, N.W.
Suite 1214
Washington, D. C. 20036

Environmental and Urban Affairs Library
Florida International University
Miami, Florida 33199

Mr. Norman A. Coll, Esquire
Steel, Hector and Davis
1400 Southeast First National
Bank Building
Miami, Florida 33131

Mr. Henry Yaeger, Plant Manager
Turkey Point Plant
Florida Power and Light Company
P. O. Box 013100
Miami, Florida 33101

Honorable Dewey Knight
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

Resident Inspector
Turkey Point Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Post Office Box 971277
Quail Heights Station
Miami, Florida 33197

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U. S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, N.W.
Atlanta, Georgia 30308

Mr. Jack Shreve
Office of the Public Counsel
Room 4, Holland Building
Tallahassee, Florida 32304

Administrator
Department of Environmental
Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 55 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages	Insert Pages
ii	ii
iii	iii-a
-	iii-b
3.1-1	3.1-1
-	3.15-1
-	4.16-1
B3.1-1	B3.1-1
-	B3.15-1
-	B4.15-1

TABLE OF CONTENTS (Continued)

TABLE OF CONTENTS (Continued)

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4

TABLE OF CONTENTS (Continued)

3.0 LIMITING CONDITIONS FOR OPERATION

3.1 REACTOR COOLANT SYSTEM

Applicability: Applies to the operating status of the Reactor Coolant System.

Objective: To specify those limiting conditions for operation of the Reactor Coolant System which must be met to assure safe reactor operation.

Specification:

1. OPERATIONAL COMPONENTS

a. Reactor Coolant Pumps

1. A minimum of ONE pump shall be in operation when the reactor is in power operation, except during low power physics tests.

2. A minimum of ONE pump, or ONE Residual Heat Removal Pump, shall be in operation during reactor coolant boron concentration reduction.

3. Reactor power shall not exceed 10% of rated power unless at least TWO reactor coolant pumps are in operation.

4. Reactor power shall not exceed 45% of rated power with only two pumps in operation unless the overtemperature ΔT trip setpoint, K_1, for two loop operation, has been set at 0.88.

5. A reactor coolant pump shall not be started when cold leg temperature is $\leq$ 275°F unless steam generator secondary water temperature is less than 50°F above the RCS temperature (including instrument error).

b. Steam Generators

1. A minimum of TWO steam generators shall be operable when the average coolant temperature is above 350F.

c. Pressurizer Safety Valves

1. ONE valve shall be operable whenever the head is on the reactor vessel except during hydrostatic tests.

2. THREE valves shall be operable when the reactor coolant average temperature is above 350F or the reactor is critical.

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4

3.15 OVERPRESSURE MITIGATING SYSTEM

Applicability: Establishes operating limitations to assure that the limits of 10 CFR 50, Appendix G, are not exceeded.

Objectives: To minimize the possibility of an overpressure transient which could exceed the limits of 10 CFR 50, Appendix G.

Specification:

1. At RCS temperature less than or equal to 380°F, valves MOV-*-843 A, MOV-*-843 B, MOV-*-866 A, and MOV-*-866 B shall be closed.

2. If any of the valves in 3.14.1 are found to be open while RCS temperature is less than or equal to 380°F, perform at least one of the following within the next 8 hours:

   a. block the corresponding flow path to the reactor vessel,
   b. close the valve, or
   c. depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or
   d. verify at least one pressurizer power operated relief valve is maintained open.

3. At RCS temperature less than or equal to 275°F, two pressurizer power operated relief valves shall be operable at the low setpoint range.

   a. If one power operated relief valve is inoperable with RCS temperature less than or equal to 275°F, perform at least one of the following within 7 days:

      (1) restore operability of the power operated relief valve, or

      (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or

      (3) verify at least one pressurizer power operated relief valve is maintained open.

   b. If both power operated relief valves are inoperable with RCS temperature less than or equal to 275°F, perform at least one of the following within the next 24 hours:

      (1) restore operability of at least one power operated relief valve, or

      (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or

      (3) verify at least one pressurizer power operated relief valve is maintained open.

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4

4.16 OVERPRESSURE MITIGATING SYSTEM

Applicability: Applies to periodic surveillance of the Overpressure Mitigating System.

Objective: To demonstrate operability of the Overpressure Mitigating System.

Specification:

1. Within 1 month prior to operation in a condition where the PORV would be required to be operable, the pressurizer power operated relief valve actuation circuitry shall be functionally tested. The functional test need not include actual valve operation.

2. While RCS temperature is less than or equal to 380°F, verify daily that valves MOV-*-843 A, MOV-*-843 B, MOV-*-866 A, and MOV-*-866 B are closed.

3. While RCS temperature is less than or equal to 275°F, verify weekly that the isolation valve for each operable pressurizer power operated relief valve is open.

4. While RCS temperature is less than or equal to 275°F, the pressurizer power operated relief valve actuation circuitry shall be functionally tested monthly. The functional test need not include actual valve operation.

5. Testing shall be in accordance with approved plant procedures.

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4

B3.1 BASES FOR LIMITING CONDITIONS FOR OPERATION, REACTOR COOLANT SYSTEM

1. Operational Components

The specification requires that a sufficient number of reactor coolant pumps be operating to provide coast down core cooling in the event that a loss of flow occurs. The flow provided will keep DNBR well above 1.30. When the boron concentration of the Reactor Coolant System is to be reduced the process must be uniform to prevent sudden reactivity changes in the reactor. Mixing of the reactor coolant will be sufficient to maintain a uniform boron concentration if at least one reactor coolant pump or one residual heat removal pump is running while the change is taking place. The residual heat removal pump will circulate the reactor coolant system volume in approximately one half hour.

Each of the pressurizer safety valves is designed to relieve 293,330 lbs. per hr. of saturated steam at the valve set point.(1) Below 350 F and 450 psig in the Reactor Coolant System, the Residual Heat Removal System can remove decay heat and thereby control system temperature and pressure. If no residual heat were removed by any of the means available the amount of steam which could be generated at safety valve lifting pressure would be less than the capacity of a single valve. Also, two safety valves have capacity greater than the maximum surge rate resulting from complete loss of load.(2)

The 50°F limit on maximum differential between steam generator secondary water temperature and reactor coolant temperature assures that the pressure transient caused by starting a reactor coolant pump when cold leg temperature is $\leq$ 275°F can be relieved by operation of one Power Operated Relief Valve (PORV). The 50°F limit includes instrument error.

2. Pressure/Temperature Limits

All components in the Reactor Coolant System are designed to withstand the effects of cyclic loads due to system temperature and pressure changes. These cyclic loads are introduced by normal load transients, reactor trips, and startup and shutdown operations. The various categories of load cycles used for design purposes are provided in

B3.15 BASES FOR LIMITING CONDITIONS OF OPERATION, OVERPRESSURE MITIGATING SYSTEM

The operability of two PORV's or an RCS vent opening of greater than or equal to 2.20 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the RCS cold legs are $\leq$ 275°F. Either PORV has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either (1) the start of an idle RCS with the secondary water temperature of the steam generator $\leq$ 50°F above the RCS cold leg temperature (includes margin for instrument error) or (2) the start of a HPSI pump and its injection into a water solid RCS.

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4

B4.15 BASES FOR SURVEILLANCE REQUIREMENTS, OVERPRESSURE MITIGATING SYSTEM

The specified testing of the Overpressure Mitigating System (OMS) will verify its operability. The capacity of one pressurizer Power Operated Relief Valve is sufficient to relieve potential overpressure transients when the RCS is in the low temperature overpressure protection range. Proper functioning of the OMS combined with selected administrative controls will demonstrate the integrity of the system.

Amendment No. 55, Unit 3
Amendment No. 47, Unit 4