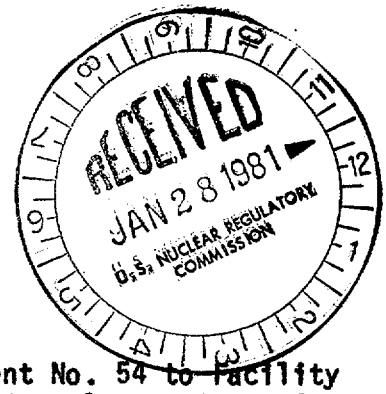
Docket File

January 15, 1981

Docket No. 50-251



Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
P.O. Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit No.4. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 18, 1980.

The amendment permits continued operation of Unit 4 for six equivalent months of operation from January 13, 1981 at which time the steam generator for Unit 4 shall be inspected. This action is subject to your submittal for staff review, information concerning the tube wastage predicted to occur during the latter half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by February 28, 1981.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Enclosures:
1. Admendment No. 54 to DPR-41
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

Docket File 50-251
NRC PDR
Local PDR
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TERA

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C 20555

January 15, 1981

Docket No. 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
P.O. Box 529100
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Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit No.4. The amendment consists of changes to the Technical Specifications in response to your application transmitted by letter dated December 18, 1980.

The amendment permits continued operation of Unit 4 for six equivalent months of operation from January 13, 1981 at which time the steam generator for Unit 4 shall be inspected. This action is subject to your submittal for staff review, information concerning the tube wastage predicted to occur during the latter half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by February 28, 1981.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Steven A. Varga

Steven A. Varga, Chief
Operating Reactors Branch #1

Enclosures:
1. Admendment No. 54 to DPR-41
2. Safety Evaluation
3. Notice of Issuance

cc w/enclosures:
See next page

Robert E. Uhrig
Florida Power and Light Company

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Florida Power and Light Company
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Honorable Dewey Knight
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Bureau of Intergovernmental Relations
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Resident Inspector
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U. S. Nuclear Regulatory Commission
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Miami, Florida 33197

Director, Criteria and Standards Division
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Washington, D. C. 20460

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Atlanta, Georgia 30308

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT, UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated December 18, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, paragraph 3.B and 3.D.1 of Facility Operating License No. DPR-41 are hereby amended and a new paragraph 3.D.7 is added to read as follows:

   (B) Technical Specifications
   The Technical Specification contained in Appendices A and B, as revised through Amendment No. 54 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

   (D) Steam Generator Operation
   (1) After operation in Cycle 7 or six equivalent full power months from January 13, 1981, Turkey Point Unit 4 shall be brought to the cold shutdown condition and the steam generators shall be

inspected unless: (1) an inspection of the steam generators is performed within this period as a result of the requirements in 2, 3 and 4 below, or (2) an acceptable analysis of the susceptibility for stress corrosion cracking of tubing is submitted to explicitly justify continued operation of Unit No. 4 beyond the authorized period of operation. Any analysis justifying continued operation must be submitted at least 45 days prior to the expiration date of the authorized period of operation. For the purpose of this requirement, equivalent operation is defined as operation with the reactor coolant at a temperature greater than 350° F. Nuclear Regulatory Commission (NRC) approval shall be obtained before resuming power operation following this inspection.

(7) The licensee shall provide for staff review information concerning the tube wastage predicted to occur during the latter half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by February 28, 1981.

3. This license amendment is effective as of January 13, 1981

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: January 15, 1981

ATTACHMENT TO LICENSE AMENDMENT
AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-41
DOCKET NO. 50-251

Replace the following page of the Facility Operating License No. DPR-41 with the attached page as indicated. The changed areas in the license is indicated by a marginal line.

Remove Page	Insert Pages
4	4
5	5

B. Technical Specifications

The Technical Specifications contained in Appendices A and B as revised through Amendment No. 54 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

C. This license is subject to the following conditions for the protection of the environment:

(1) The applicant shall pursue evaluations of alternatives to the proposed cooling channel system during construction, interim operation, and evaluation of the channel system. These evaluations shall include at least the following:

(a) Study of availability of groundwater or other alternative sources of surface water to use in the cooling system.

(b) Study of applicability of mechanical cooling devices, including powered spray modules and cooling towers.

(c) Study of marine environmental impacts of once-through cooling alternatives (described in Section X of the AEC Final Environmental Statement on Turkey Point Units 3 and 4, July 1972).

(2) The applicant shall take appropriate corrective action on any adverse effects determined as a result of monitoring and study programs. To the fullest extent practicable, the applicant shall utilize results of study programs in improving and modifying the operation of the facility and its cooling system so as to achieve a minimal adverse environmental impact.

D. Steam Generator Operation

(1) After operation in Cycle 7 of six equivalent full power months from Janaury 13, 1981, Turkey Point Unit 4 shall be brought to the cold shutdown condition and the steam generators shall be inspected unless: (1) an inspection of the steam generators is performed within this period as a result of the requirements in 2, 3 and 4 below, or (2) an acceptable analysis of the susceptibility for stress corrosion cracking of tubing is submitted to explicitly justify continued operation of Unit No. 4 beyond the authorized period of operation. Any analysis justifying continued operation must be submitted at least 45 days prior to the expiration date of the authorized period of operation. For the purpose of this requirement, equivalent operation is defined as operation with the reactor coolant at a temperature greater than 350°F. Nuclear Regulatory Commission (NRC) approval shall be obtained before resuming power operation following this inspection.

(2) Reactor coolant to secondary leakage through the steam generator tubes shall be limited to 0.3 gpm per steam generator. With a steam generator tube leakage greater than this limit, the reactor shall be brought to the cold shutdown condition within 24 hours. A full steam generator inspection shall be performed and NRC approval shall be obtained before resuming power operation following this inspection.

(3) The concentration of radioiodine in the reactor coolant shall be limited to 1.0 microcurie/gram during normal operation and to 30 microcuries/gram during power transients.

(4) Reactor operation shall be terminated and NRC approval shall be obtained prior to resuming operation if primary to secondary leakage attributable to the denting phenomena is detected in 2 or more tubes during any 20 day period.

(5) The Metal Impact Monitoring System (MIMS) shall be contained in operation with the capability of detecting losse objects. If the MIMS is out of service in other than cold shutdown or refueling mode of operation, this fact shall be reported to the NRC. Any abnormal indications from the MIMS shall also be reported to the NRC by telephone by the next working day and by a written evaluation within two weeks.

(6) Following each startup from below 350°F, core barrel movement shall be evaluated using neutron noise techniques.

(7) The licensee shall provide for staff review information concerning the tube wastage predicted to occur during the latter half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by February 28, 1981.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT PLANT UNIT NO. 4

DOCKET NO. 50-251

INTRODUCTION

By letter dated December 18, 1980 Florida Power and Light Company (the licensee) submitted the results of the most recent steam generator inspection at Turkey Point Unit 4 and requested authorization to operate Unit 4 for six equivalent months from the time of the inspection. Equivalent operation is defined as operation with the reactor coolant greater than 350°F. The implemented steam generator inspection and preventive plugging program are similar to those performed previously at this and other similarly degraded units and have been determined adequate by the NRC to support six (6) equivalent months of operation, subject to the licensee submittal for staff review, information concerning the tube wastage predicted to occur during the later half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by Fedrauary 28, 1981.

DISCUSSION

The steam generator tube inspection performed at Turkey Point Unit 4 during November 1980 included programs to assess tube degradation associated with both the denting and wastage phenomena. For denting, tube gauging was performed in all three steam generators using 0.650, 0.610, and 0.5 inches (diameter) eddy current probes. The implemented gauging program was similar to those implemented previously at this and other similarly degraded units and included the gauging of all unplugged tubes within areas (tubelane, periphery, wedge, and patch plate regions of the hot leg, and tubelane region of the cold leg) where significant denting activity had been observed previously. Significant denting, in this context, is considered to include (in addition to leakers) tubes restricting passage of a .610 inch probe (or less) and tubes at the periphery of the hot leg wedge location and on either side of the patch plate boundary which restrict passage of a .650 inch probe (or less), since these tubes are the most likely candidates to develop inservice leaks.

In previous inspections of the tubelane region, finite element analysis had been used to determine the progression of significant tube restriction activity for purposes of defining the boundary for the tubelane gauging inspection. However, the 17.5% tube hoop strain contour which realistically bounded the significant tube restriction activity in the tubelane following the previous inspection is now predicted to cover most of the support plate. Thus, the licensee elected to gauge the tubelane tubes within a boundary incorporating regions of previously observed activity (i.e. tubes restricting passage of 0.650 inch probes), plus several rows of tubes beyond.

With regard to the defined regions (discussed above) within which all tubes were gauged, if a restricted tube (tube restricting a 0.650 inch probe) was found close to the inspection boundary, the inspection was expanded in that area. In addition, a sample population of tubes in the central bundle region, located outside these defined regions, was tested with a 0.700 inch probe in the hot and cold legs, respectively, as part of the Regulatory Guide 1.83 eddy current inspection (to be discussed). These latter inspections provide an early indication of any new deformation which exists away from the regions usually regarded as active (i.e. the tubelane, patch plate, wedges and periphery). Tubes restricted in previous sample inspections, but not adjacent to the areas of predominant activity were also incorporated into the inspection.

Measurements of the visible support plate flow slots in all steam generators were made to assess the condition of the support plates and to provide a gross measure of the continuation of denting.

Eddy current inspection for wastage was conducted in accordance with Regulatory Guide 1.83 in all of the steam generators. The U-bends of the unplugged tubes in rows three through five in steam generator C were also eddy current inspected.

The following tabulation summarizes the number of tubes included in the gauging and eddy current inspections:

	A Hot Leg	A Cold Leg	B Hot Leg	B Cold Leg	C Hot Leg	C Cold Leg
Gauging	1184	154	1159	117	1324	161
U-Bend Rows 2-5	-	-	-	-	-	79
R. G. 1.83	397	317	383	527	268	461

INSPECTION RESULTS

The results of the gauging inspection in terms of the number of tubes restricting passage of a given size probe of 0.65 inch or less are summarized below:

	Tubelane		Periphery & Wedge		Patch Plate
	Hot Leg	Cold Leg	Hot Leg	Cold Leg	Hot Leg
SG A					
.650"	29	0	25	0	3
.610"	7	0	3	0	1
.540"	0	0	1	0	0

SG B					
.650"	32	0	42	0	3
.610"	4	3	8	2	0
.540"	0	0	0	0	0
SG C					
.650"	34	0	15	0	13
.610"	5	1	5	3	1
.540"	1	0	1	0	0

Tubes in the tubelane region that restrict a 0.650 inch probe or less are located adjacent to the areas in which such restrictions have been observed during previous inspections. In general, tube restriction activity observed in the periphery, wedge, and patch plate areas appears consistent with previous experience at this and other similarly degraded units. In Steam Generator B, four tube restrictions occurred away from the pattern of prior results. Two tubes, one restricts a 0.610 inch probe and the other restricts a 0.650 inch probe, were located adjacent to a tube which had been pulled in September 1975 as part of the earliest field investigations of denting. The other two tubes, which restrict passage of 0.650 inch probes, were located adjacent to the patch plate inspection area. In Steam Generator C, one tube restricting a 0.650 inch probe was located adjacent to the patch plate inspection area. The level of denting activity in the cold leg remains low compared to the hot leg activity.

The eddy current inspection performed in accordance with Regulatory Guide 1.83 identified 25 tubes that required plugging due to thinning indications. The following summarizes the results of the Regulatory Guide 1.83 inspection:

Size of Indication (% Wall Penetration)	SG A		SG B		SG C	
	Inlet	Outlet	Inlet	Outlet	Inlet	Outlet
<20	54	76	50	125	4	132
20-29	40	174	30	133	4	194
30-39	20	71	21	83	1	57
40-49	1	0	0	13	1	2
50-59	1	0	1	4	1	0
60-69	1	0	0	0	0	0

It should be noted from the above tables that the results for the cold leg side of Steam Generator A include four (4) indications which were picked up during gauging.

The results of the Regulatory Guide 1.83 inspection for wastage indicate an apparent increase in thinning activity relative to previous inspections, particularly in the cold leg side of steam generator B where 17 pluggable indications were found at the top of the tubesheet elevation. The licensee reports an average increase in thinning during the period April 1979 to

November 1980 of 9.9% in steam generator B and approximately 1% in steam generators A and C. Data provided by phone on January 6, 1981 indicates that during the period May 1980 to November 1980 thinning increased by an average of 13% for the 17 tubes in the cold leg of steam generator B which were found to contain pluggable indications.

The licensee believes that while some additional thinning may have occurred since the previous inspections, the overall increase is likely to be smaller than that indicated by the reported data. Comparison of the affected tubes in steam generator B with corresponding signals from previous inspections shows the presence of an increased denting component. The dent signal influences the thinning signal, causing slight phase rotation and overestimations of the depth of penetration. Westinghouse is currently reevaluating the eddy current signals from both the November and May 1980 inspections in an attempt to determine a more realistic value of thinning increase during this period.

No eddy current indications were identified in the U-bends of the unplugged tubes in Rows 3 through 5.

The measurements of the support plate flow slots indicated no deviations from anticipated conditions.

TUBE PLUGGING PROGRAM

The plugging criteria implemented for dented tubes during the November 1980 steam generator inspection are the same as those implemented previously at this and other similarly degraded units. These criteria include the plugging of two tubes beyond any 0.540 inch restricted tube found in the tubelane region. This latter criterion was based in the past upon finite element predictions regarding the progression of tube denting during the next operating interval. Based upon their experience, the licensee believes that the finite element model no longer provides an appropriate basis for tube plugging. Thus, they have discontinued the practice of updating the analyses with each succeeding inspection. The criterion for plugging two tubes beyond 0.540 inch restricted tubes, which was previously developed using the finite element model, has been retained by the licensee since its application has generally been effective in reducing the frequency of tube leakage events resulting from denting. The licensee notes that only one 0.540 inch restricted tube in the tubelane region, where this criterion is applicable, was found during this inspection.

Tubes with greater than 40% through wall eddy current indications were plugged.

Implementation of the plugging criteria resulted in 24, 37, and 44 tubes being plugged for denting and 3, 18, and 4 tubes being plugged for wastage in steam generators A, B, and C, respectively. Total steam generator tube plugging in all three steam generators is approximately 23.8% which is conservatively bounded by the 25% tube plugging assumption ECCS analysis, approved on May 15, 1980.

EVALUATION

The November 1980 gauging and preventive plugging program at Turkey Point Unit 4 is similar to previous programs conducted in at this and other similarly degraded units. This inspection included the gauging of all tubes within areas (tubelane, periphery, wedge, and patch plate regions) were significant denting activity, has been observed previously. In addition, a sample population of tubes in the central bundle region were gauged as part of the Regulatory Guide 1.83 inspection for wastage.

Based upon our review of the gauging results, we find that the observed denting activity is generally consistent with previous experience at this and other similarly degraded units. Although denting in four (4) tubes in Steam Generators B and one (1) tube in Steam Generator C occurred away from the pattern of prior results, we find significant tube restriction activity remains confined to areas immediately adjacent to previous activity. The implemented gauging program was sufficient to adequately determine the condition of the steam generator from a denting standpoint.

The preventative plugging criteria implemented during this and previous inspections have proven successful in removing from service those tubes which are the most likely candidates to develop inservice leaks. The inspection data and recent operating experience provide adequate justification for the implemented criterion of plugging two tubes beyond 0.540 inch restricted tubes in the tubelane. We find that the implemented gauging program and preventive plugging criteria provide reasonable assurance that the vast majority of tubes most likely to develop inservice leaks have been identified and removed from service. No forced shutdowns because of steam generator tube leakage occurred during the 4.75 months of operation since the previous inspection in May 1980. The 0.3 gmp leak rate limit in the license provides adequate assurance that even if through wall cracks and leaks occur, they will be detected and appropriate corrective action taken before excessive leakage can occur from tube degradation during normal operating, transient, or accident conditions.

With regard to the wastage phenomenon, the November 1980 eddy current inspection (in accordance with Regulatory Guide 1.83) and 40% plugging limit are similar to those implemented in previous inspections. The results, however, indicate an apparent increase in thinning activity relative to previous inspections, particularly in the cold leg side of steam generator B where 17 pluggable indications were found. A comparison of the November 1980 eddy current results with the previous May 1980 results indicates an apparent average increase of 13% in wall thinning penetration during this period for the 17 tubes in the cold leg of steam generator B containing the plugging indications. The licensee believes that the increase in thinning is likely to be less than that indicated by the reported data due to the confounding effect of increased denting component on the eddy current signal. They have not submitted adequate comfirmatory data to support this belief.

In view of data indicating a possibly high rate of wastage degradation in the Turkey Point Unit 4 steam generator, we have performed an evaluation to determine if the implemented 40% plugging limit provides adequate assurance that unacceptable wastage degradation of the tube walls will not occur during the six months preceeding the next steam generator inspection. We have used Regulatory Guide 1.121 (issued for comment in August 1976) as guidance in evaluating the reported data. Regulatory Guide 1.121 defines the basis for plugging degraded steam generator tubes. The Regulatory Guide specifies, in part, that degraded steam generator tubes should meet the following criteria:

(1) A margin of safety against tube failure during postulated accidents consistant with ASME code requirements.

(2) A factor of safety against tube burst of at least 3 under normal operating conditions, and

(3) Loadings during normal operating conditions should not produce a primary membrane stress in excess of the material yield stress at operating temperature.

In meeting these criteria, the Regulatory Guide states that allowance is to be made for future degradation. On the basis of wastage degradation at the average rate of 13% (through wall) during the previous 4.75 equivalent full power months of operation, we estimate that the above criteria can only be complied with during the first 3.5 months of operation from the time of the November 1980 inspection. Based on information supplied by the licensee in telephone conservations, we have performed simplified calculations that indicate that acceptable margins against tube failure during postulated accidents will exist throughout a six month operating period. However, we estimate that during normal operating conditions the minimum margin to burst would be approximately 2.5 rather than 3 and the material yield strength would be exceeded by approximately 10% at the end of the six month operating period.

Although compliance with Regulatory Guide Criteria pertaining to normal operating pressure cannot be shown to exist beyond 3.5 months, we believe that the unit can be safely operated for six months for the following reasons:

(1) Margin with respect to tube failure under postulated accident conditions provides reasonable assurance against the occurrance of multiple tube failures and excessive leakage.

(2) Margin of approximately 2.5 with respect ot tube burst exists under normal operating conditions and provides assurance that multiple tube ruptures will not occur.

(3) In degraded tubes where stresses exceed yield under normal operating conditions, the consequences would likely be limited to small leaks of individual tubes. The 0.3 gpm primary to secondary

leak rate limit in the Plant Licensee provides assurance that in the event that a leak occurs in service, appropriate and timely corrective action will be taken.

From a denting standpoint, we find that the gauging inspection results, implemented plugging and existing leak rate limits adequately support six equilvalent months of operation from the time of the November 1980 inspection. By Febrauary 28, 1981, confirmatory data and analysis which demonstrates that unacceptable wastage degradation of the tube walls will not occur prior to performing the next steam generator inspection will be submitted.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: January 15, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 54 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of the Turkey Point Plant, Unit No. 4 (the facility) located in Dade County, Florida. The amendment is effective January 13, 1981.

The amendment permits continued operation of Unit No. 4 for six equivalent months of operation from January 13, 1981, at which time the steam generators for Unit No. 4 shall be inspected. This action is subject to licensee submittal for staff review, information concerning the tube wastage predicted to occur during the latter half of the operating period which begins January 13, 1981 and extends for six equivalent months of operation. This information is to be supplied by February 28, 1981.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since this amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursant to 10 CFR §51-5(d) (4) an environmental impact statement or negative declaration and

environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated December 18, 1980, (2) Amendment No. 54 to License No. DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 15th day of January 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing