

DEC 5 1972

Docket Nos. 50-250  
and 50-251

Florida Power & Light Company  
ATTN: Dr. James Coughlin  
P. O. Box 3100  
Miami, Florida 33101

Change No. 2  
License No. DPR-31

Gentlemen:

Your letter dated September 20, 1972, proposed seven revisions to the Technical Specifications attached as Appendix A to Facility Operating License No. DPR-31. These proposals have been designated Change No. 2. We have dealt with them as follows:

1. The proposal to change the underfrequency trip of the reactor coolant pump motors from 55.6 Hz to 56.1 Hz is approved. Since equipment tests have indicated a somewhat longer circuit delay time than previously assumed, this change represents a proper, conservative adjustment to the trip set point.
2. The proposal to provide additional relief from minimum shutdown margin requirements is unwarranted and, therefore, disapproved. In our view, the current specifications afford you adequate flexibility to perform all necessary control rod worth tests.
3. The proposal to substitute certain of the horizontal tendons included in the containment surveillance program, which have proven inaccessible, with other equally representative tendons is approved.
- 4., 5., and 6.

The corrections of the printing and spelling errors are approved.

7. The notification of Regulatory Operations by telephone or telegraph within 24 hours of an abnormal occurrence is approved. Also, the appropriate AEC organizational name changes are authorized.

*lw*

OFFICE ▶						
SURNAME ▶						
DATE ▶						

We have concluded that the aforesaid approved changes do not involve significant hazard considerations not described or implicit in the Final Safety Analysis Report and that there is reasonable assurance that the health and safety of the public will not be endangered. Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, these changes are hereby authorized.

The Technical Specifications of Facility Operating License No. DPR-31 are changed as set forth in the revised pages 2.3-3, 4.4-4, 2 and 3 of Table 4.12-1, 6.1-14, and 6.6-9 which are enclosed.

Sincerely,

*original signed by*  
*Karl Goller*  
 for R. C. DeYoung, Assistant Director  
 for Pressurized Water Reactors  
 Directorate of Licensing

Enclosure:  
 Revised pages

cc: Mr. Jack Newman  
 Lowenstein, Newman & Reis  
 1100 Connecticut Avenue NW  
 Washington, D. C. 20036

DISTRIBUTION:

AEC PDR	MJinks (w/4 enclosures)
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DATE ▶	12/5/72	12/5/72	12/5/72		

$$\text{Overpower } \Delta T \leq \Delta T_o \left[ 1.11 - K_1 \frac{dT}{dt} - K_2 (T - T') - f(\Delta I) \right]$$

$\Delta T_o$  = Indicated  $\Delta T$  at rated power, F

T = Average temperature, F

T' = Indicated average temperature at nominal conditions and rated power, F

$K_1$  = 0 for decreasing average temperature,  
0.2 sec./F for increasing average temperature

$K_2$  = 0.00068 for T equal to or more than T';  
0 for T less than T'

$\frac{dT}{dt}$  = Rate of change of temperature, F/sec

f( $\Delta I$ ) = As defined above

#### Pressurizer

Low Pressurizer pressure - equal to or greater than 1835 psig.

High Pressurizer pressure - equal to or less than 2385 psig.

High Pressurizer water level - equal to or less than 92% of full scale.

#### Reactor Coolant Flow

Low reactor coolant flow - equal to or greater than 90% of normal indicated flow

Low reactor coolant pump motor frequency - equal to or greater than 56.1 Hz

Under voltage on reactor coolant pump motor bus - equal to or greater than 70% of normal voltage

#### Steam Generators

Low-low steam generator water level - equal to or greater than 5% of narrow range instrument scale

#### 4.4.6 TENDON SURVEILLANCE

##### Lift-off

Lift-off readings will be taken for the following nine (9) tendons available for inspection:

<u>Unit 3</u>	<u>Unit 4</u>
Horizontal 62H18,42H70,64H50	13H15,51H50,35H70
Vertical 23V1,45V7,61V1	23V1,45V7,61V1
Dome 1D27,2D28,3D28	1D28,2D28,3D28

##### Wire Inspection

One horizontal, one vertical and one dome tendon will be relaxed and one wire will be removed from each as a sample. (At subsequent inspections different tendons will be used for the sample). Wires will be visually inspected for corrosion and pitting. Tensile tests will be performed on three (3) samples cut from each wire (one from each end and one from the middle) of a length equal to the maximum length acceptable for the test apparatus to be used.

After samples are taken, tendons will be re-tensioned and final lift-off readings will be taken.

##### Test Frequency

Lift-off readings and wire inspection will take place at the end of the first, third and every fifth year thereafter from the date of the structural integrity test. Tendon surveillance may be conducted during reactor operation.

TABLE 4.12-1 (continued)

OPERATIONAL ENVIRONMENTAL RADIOLOGICAL SURVEILLANCE PROGRAM

	<u>Criteria and Sampling Locations</u>	<u>Collection Frequency</u>	<u>Analysis/Counting</u>
B. Ground Water	Dolan Farm	Quarterly	Same as II.A.2
C. Potable Water	City of Homestead, drinking water supply Naranja Water Company, drinking water supply	Quarterly	Same as II.A.2
D. Bottom Sediment			
1. Canal	Upper Discharge Canal (2 locations)	Quarterly	Gamma spectral analysis Sr-90
2. Bay	Homestead Bayfront Park Girl Scout Bathing Area Mouth of Discharge Canal - Biscayne Bay Card Sound North of Causeway Mouth of Model Land Canal Mouth of Discharge Canal - Card Sound Card Sound - North Boundary	Quarterly (all locations)	Same as II.D.1

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TABLE 4.12-1 (continued)

OPERATIONAL ENVIRONMENTAL RADIOLOGICAL SURVEILLANCE PROGRAM

	<u>Criteria and Sampling Locations</u>	<u>Collection Frequency</u>	<u>Analysis/Counting</u>
II. <u>WATER</u> (cont'd)			
E. Aquatic Biota			
1.	Crustacea	Quarterly	Gamma spectral analysis Sr-90
a.	Lobster, crab &/or shrimp		
	Mouth of Discharge Canal - Biscayne Bay Card Sound North of Causeway Bay side of Caesar Creek Bay side of Ragged Keys Mouth of Model Land Canal Mouth of Discharge Canal - Card Sound Card Sound - North Boundary		
2.	Fish (vertebrates)	Quarterly	Same as II.E.1
a.	Carnivores		
	Barracuda or Mangrove Snapper		
b.	Herbivores	Quarterly	Same as II.E.1
	Mullet (mugil cephalus)		
3.	Other	Semi-annually	Gamma spectral analysis Sr-90
a.	Manatee Grass		
	&/or Turtle Grass		
b.	Algae	Semi-annually	Same as II.E.3a
c.	Sponges (porifera)	Quarterly	Same as II.E.3a

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environment

other appropriate fields required by the  
unique characteristics of the nuclear units  
involved.

When the nature of a particular situation dictates,  
special consultants will be utilized to provide  
expert advice to board members upon request of any  
two board members.

3. If sufficient expertise in the specialty in e.2 above  
is not available from within the board, staff specialists  
and/or outside consultants shall be used to supplement  
review and audit functions of the board. Personnel in  
this category shall be competent in technical matters  
related to nuclear unit safety and other engineering  
and scientific support aspects.

f. Meeting frequency:

Meetings shall be no less frequent than semiannually, or  
as required by circumstances.

g. Quorum:

1. No less than a majority of the board voting member-  
ship shall constitute a quorum.
2. Either the Chairman or Vice Chairman shall be present.
3. No more than a minority of the quorum shall have line  
responsibility for nuclear unit operation.

h. Minutes:

Meeting minutes shall be prepared, formally approved, re-  
tained and also promptly distributed to appropriate members  
of management having responsibility in the area reviewed.

(h) Occupational Personnel Radiation Exposure

A tabulation in 0.15 rem increments of personnel exposures between 0 - 1.25 rem by work groups.

6.6.2 NON-ROUTINE REPORTS

a. Reporting of Abnormal Events

Abnormal Occurrence Reports

Notification shall be made within 24 hours by telephone or telegraph to the Director of the Region II Field Office of the Directorate of Regulatory Operations followed by a written report within 10 days to the Director, Directorate of Licensing (cc. to the Director of the Region II Field Office, Directorate of Regulatory Operations) in the event of the abnormal occurrences as defined in Section 1.13.

The written report on these abnormal occurrences, and to the extent possible, the preliminary telephone or telegraph notification, shall: (a) describe, analyze and evaluate safety implications, (b) outline the measures taken to assure that the cause of the condition is determined, and (c) indicate the corrective action (including any changes made to the procedures and to the quality assurance program) taken to prevent repetition of the occurrence and of similar occurrences involving similar components or systems.

In addition, the written report shall relate any failures or degraded performance of systems and components for the incident to similar equipment failures that may have previously occurred at the plant. The evaluation of the safety implications of the incident should consider the cumulative experience obtained from the record of previous failures and malfunctions of the affected systems and components or of similar equipment.