

May 20, 1994

Docket Nos. 50-250
and 50-251

Mr. J. H. Goldberg
President - Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS RE: ELIMINATION
OF CRANKING DIESEL GENERATORS (TAC NOS. M87662 AND M87663)

The Commission has issued the enclosed Amendment No. 164 to Facility Operating License No. DPR-31 and Amendment No. 158 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Units Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated September 3, 1993, concerning the Standby Feedwater System and the associated bases. Specifically, you proposed to eliminate the reliance on the five "cranking" diesel generators by replacing the motor driver on one of the standby steam generator feedwater pumps with a dedicated diesel driver.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)
Richard P. Croteau, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 164 to DPR-31
2. Amendment No. 158 to DPR-41
3. Safety Evaluation

cc w/enclosures:
See next page

Distribution - See next page

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DATED: May 20, 1994

AMENDMENT NO. 164 TO FACILITY OPERATING LICENSE NO. DPR-31-TURKEY POINT UNIT 3
AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-41-TURKEY POINT UNIT 4

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Docket File

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 164
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 3, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

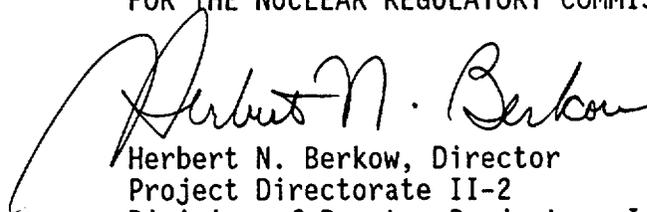
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.164, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of completion of the modifications specified in the application for amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 20, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 158
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 3, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 158, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days of completion of the modifications specified in the application for amendment.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 20, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 164 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 158 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove pages

x

3/4 7-11

B 3/4 7-4

B 3/4 7-5

Insert pages

x

3/4 7-11

3/4 7-11a

B 3/4 7-4

B 3/4 7-5

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PLANT SYSTEMS

STANDBY STEAM GENERATOR FEEDWATER SYSTEM

LIMITING CONDITION FOR OPERATION

3.7.1.6 Two Standby Steam Generator Feedwater Pumps shall be OPERABLE* and at least 60,000 gallons of water (available volume), shall be in the Demineralized Water Storage Tank**.

APPLICABILITY: MODES 1, 2 and 3

ACTION:

- a. With one Standby Steam Generator Feedwater Pump inoperable, restore the inoperable pump to available status within 30 days or submit a SPECIAL REPORT per 3.7.1.6d.
- b. With both Standby Steam Generator Feedwater Pumps inoperable, restore at least one pump to OPERABLE status within 24 hours, or:
 1. Notify the NRC within the following 4 hours, and provide cause for the inoperability and plans to restore pump(s) to OPERABLE status and,
 2. Submit a SPECIAL REPORT per 3.7.1.6d.
- c. With less than 60,000 gallons of water in the Demineralized Water Storage Tank restore the available volume to at least 60,000 gallons within 24 hours or submit a SPECIAL REPORT per 3.7.1.6d.
- d. If a SPECIAL REPORT is required per the above specifications submit a report describing the cause of the inoperability, action taken and a schedule for restoration within 30 days in accordance with 6.9.2.

SURVEILLANCE REQUIREMENTS

4.7.1.6.1 The Demineralized Water Storage tank water volume shall be determined to be within limits at least once per 24 hours.

4.7.1.6.2 At least monthly verify the Standby Steam Generator Feedwater Pumps are OPERABLE by testing in recirculation on a STAGGERED TEST BASIS.

4.7.1.6.3 At least once per 18 months, verify operability of the respective standby steam generator feedwater pump by starting each pump and providing feedwater to the steam generators.

*These pumps do not require plant safety related emergency power sources for operability and the flowpath is normally isolated.

**The Demineralized Water Storage Tank is non-safety grade.

PLANT SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

4.7.1.6.4 The diesel engine for the diesel-driven Standby Steam Generator Feedwater Pump shall be demonstrated OPERABLE:

- a. At least once every 31 days, by testing with the associated standby steam generator feedwater pump in recirculation.
- b. At least once per 18 months, by subjecting the diesel to an inspection in accordance with procedures prepared in conjunction with its manufacturer's recommendations for the class of service.

PLANT SYSTEMS

BASES

3/4.7.1.6 STANDBY STEAM GENERATOR FEEDWATER SYSTEM

The purpose of this specification and the supporting surveillance requirements is to assure operability of the non-safety grade Standby Steam Generator Feedwater System. The Standby Steam Generator Feedwater System consists of commercial grade components designed and constructed to industry and FPL standards of this class of equipment located in the outdoor plant environment typical of FPL facilities system wide. The system is expected to perform with high reliability, i.e., comparable to that typically achieved with this class of equipment. FPL intends to maintain the system in good operating condition with regard to appearance, structures, supports, component maintenance, calibrations, etc.

The function of the Standby Steam Generator Feedwater System for OPERABILITY determinations is that it can be used as a backup to the Auxiliary Feedwater (AFW) System in the event the AFW System does not function properly. The system would be manually started, aligned and controlled by the operator when needed.

The A pump is electric-driven and is powered from the non-safety related C bus. In the event of a coincident loss of offsite power, the B pump is diesel driven and can be started and operated independent of the availability of on-site or offsite power.

A supply of 60,000 gallons from the Demineralized Water Storage Tank for the Standby Steam Generator Feedwater Pumps is sufficient water to remove decay heat from the reactor for six (6) hours for a single unit or two (2) hours for two units. This was the basis used for requiring 60,000 gallons of water in the non-safety grade Demineralized Water Storage Tank and is judged to provide sufficient time for restoring the AFW System or establishing make-up to the Demineralized Water Storage Tank.

The Standby Steam Generator Feedwater Pumps are not designed to NRC requirements applicable to Auxiliary Feedwater Systems and are not required to satisfy design basis events requirements. These pumps may be out of service for up to 24 hours before initiating formal notification because of the extremely low probability of a demand for their operation.

The guidelines for NRC notification in case of both pumps being out of service for longer than 24 hours are provided in applicable plant procedures, as a voluntary 4-hour notification.

Adequate demineralized water for the Standby Steam Generator Feedwater system will be verified once per 24 hours. The Demineralized Water Storage Tank provides a source of water to several systems and therefore, requires daily verification.

The Standby Steam Generator Feedwater Pumps will be verified OPERABLE monthly on a STAGGERED TEST BASIS by starting and operating them in the recirculation mode. Also, during each unit's refueling outage, each Standby Steam Generator Feedwater Pump will be started and aligned to provide flow to the nuclear unit's steam generators.

PLANT SYSTEMS

BASES

STANDBY STEAM GENERATOR FEEDWATER SYSTEM (Continued)

This surveillance regimen will thus demonstrate operability of the entire flow path and the pump drivers at least once each refueling outage. The pump and pump drivers would typically be demonstrated by operation of the pumps in the recirculation mode monthly on a staggered test basis.

The diesel engine driver for the B Standby Steam Generator Feedwater Pump will be verified operable once every 31 days on a staggered test basis performed on the B Standby Steam Generator Feedwater Pump. In addition, an inspection will be performed on the diesel at least once every 18 months in accordance with procedures prepared in conjunction with its manufacture's recommendations for the diesel's class of service. This inspection will ensure that the diesel driver is maintained in good operating condition consistent with FPL's overall objectives for system reliability.

3/4.7.2 COMPONENT COOLING WATER SYSTEM

The OPERABILITY of the Component Cooling Water System ensures that sufficient cooling capacity is available for continued operation of safety-related equipment during normal and accident conditions. The redundant cooling capacity of this system, assuming a single active failure, is consistent with the assumptions used in the safety analyses. One pump and two heat exchangers provide the heat removal capability for accidents that have been analyzed.

3/4.7.3 INTAKE COOLING WATER SYSTEM

The OPERABILITY of the Intake Cooling Water System ensures that sufficient cooling capacity is available for continued operation of safety-related equipment during normal and accident conditions. The design and operation of this system, assuming a single active failure, ensures cooling capacity consistent with the assumptions used in the safety analyses.

3/4.7.4 ULTIMATE HEAT SINK

The limitations on the ultimate heat sink temperature ensure that sufficient cooling capacity is available either: (1) to provide normal cooldown of the facility or (2) to mitigate the effects of accident conditions within acceptable limits.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 164 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 158 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT UNITS 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By letter dated September 3, 1993, Florida Power and Light Company (the licensee or FPL) proposed to revise Technical Specification (TS) 3/4.7.1.6, "standby feedwater system," (renamed "standby steam generator feedwater system" by this revision) and the associated bases. The proposed changes relate to elimination of reliance on the five "cranking" diesel generators to power the standby steam generator feedwater (SSGF) pumps. The basis for eliminating this reliance is a plant modification which will replace the motor driver for one of the SSGF pumps with a diesel driver. The cranking diesel generators are relied upon in the event of a fire affecting the auxiliary feedwater (AFW) system (they are not relied upon to meet the station blackout rule.) In the event of a loss of AFW due to a fire, the motor-driven SSGF system pumps are used for decay heat removal via the steam generators. Power to the SSGF system from the cranking diesel generators is attained through manual operator actions taken locally. These actions take about 20 minutes to perform. The system modifications and proposed TS changes were undertaken to eliminate the high maintenance costs and remove the excessive operational burden of maintaining the operability of the non-safety-related cranking diesel generators.

With the replacement of the motor driver with a diesel driver on one of the SSGF pumps, the diesel-driven pump will be relied upon for decay heat removal in the event of a fire which disables the AFW system coincident with a loss of offsite power. The proposed revision to TS 3/4.7.1.6 takes this into account by adding surveillance requirements for the diesel-driven SSGF pump.

2.0 EVALUATION

The safety-related auxiliary feedwater system for Turkey Point 3 and 4 contains three shared turbine-driven AFW pumps. The non-safety-related SSGF system provides added reliability and diversity as a back-up to the AFW system during events involving a loss of main feedwater. The SSGF presently uses two shared motor-driven pumps, either of which can supply either or both units. Credit is taken for the SSGF to meet the requirements of 10 CFR Part 50, Appendix R, in the event of a fire in two fire zones containing AFW equipment. Because of this credit it was necessary to require operability of at least two

of the five cranking diesel generators in order to provide power to the 3C and 4C 4.16 Kv switchgear in the event of a loss of offsite power during a fire disabling the AFW system. The cranking diesel generators are capable of providing a backup power source to Units 3 and 4. Manual operator actions taken locally are required to connect this backup power source and the SSGF can be made operable via the cranking diesel generators within 20 minutes.

In order to eliminate the high maintenance costs associated with the cranking diesel generators and remove the potentially high operational burden from nuclear plant personnel, the licensee proposed replacing the electric motor for one SSGF pump (Pump "B") with a dedicated diesel driver, thereby eliminating reliance on the cranking diesel generators. In the event of a fire which disables the AFW system, coupled with a loss of offsite power, the diesel-driven SSGF pump will be relied upon to deliver the required feedwater flow to the steam generators of both units. One SSGF pump, either the motor or diesel driven pump, is adequate to meet the decay heat removal requirements of both units assuming a loss of offsite power and the unavailability of the AFW system. Both SSGF pumps will still act as diverse backups to the AFW system under all circumstances, but only the diesel-driven pump will be given credit for operating in the event of an AFW disabling fire coincident with a loss of offsite power. Therefore, the diesel-driven pump is necessary to meet the requirements of 10 CFR Part 50, Appendix R with respect to safe shutdown following a fire which disables the AFW system.

TS 3/4.7.1.6 presently allows one SSGF pump to be out of service for up to 30 days before a special report is required to be issued. With both SSGF pumps inoperable, the present TS requires that, within 24 hours, the NRC is notified and a cause for inoperability and a plan to restore a pump to operable status must be provided. In addition, a special report must be submitted to the NRC. With the proposed revision, these actions will basically remain the same except that with both pumps inoperable, the licensee must restore a pump to operable status within 24 hours or notify the NRC within the next 4 hours. The staff concurs with this change which allows the licensee to restore the pumps to an operable status. This change is also consistent with the intent of the staff's August 13, 1986, Safety Evaluation (License Amendments 118 and 112) for the SSGF system TS where the staff stated that with both pumps unavailable the problems would be corrected within 24 hours or a Special Report would be submitted. As proposed, the revised specification could result in the diesel-driven pump being inoperable for up to 30 days without any specific action taken. Although the motor-driven pump would still be available, its onsite power supply is no longer required to be operable by the TS. However, a 30-day outage time is consistent with other fire protection safe shutdown equipment such as the remote shutdown panels. The 30-day outage time is acceptable because the equipment is only relied upon in the event of a disabling fire in a specific plant area.

As a result of the design changes, Surveillance Requirements (SR) 4.7.1.6.3 and 4.7.1.6.4 have been revised and added, respectively. The proposed changes to the SR remove the reference to the cranking diesel generators and provide specific requirements for the diesel-driven SSGF pump. The proposed SR

changes are necessary to be consistent with the design changes and reflect the new reliance on the diesel-driven pump to meet the requirements of 10 CFR 50, Appendix R. The proposed changes are, therefore, acceptable.

The Bases section for TS 3/4.7.1.6 has also been revised to remove references to the cranking diesel generators and a description of the role of the diesel-driven pump has been added. The Bases section changes adequately reflect the SSGF design changes and are necessary for consistency with the revised specifications, and are, therefore, acceptable.

Based on its review and evaluation of the licensee's proposed changes to TS 3/4.7.1.6, as described above, the staff concludes that the proposed changes are necessary to be consistent with the proposed system design changes, satisfy the requirements of 10 CFR 50, Appendix R with respect to safe shutdown in the event of a fire, and maintain the SSGF system licensing commitments associated with AFW diversity and reliability, i.e., Generic letter 83-37, "NUREG-0737 TECHNICAL SPECIFICATIONS." The staff, therefore, concludes that the proposed changes are acceptable and should be approved.

3.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (58 FR 52985). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

Based on the staff evaluation in Section 2.0 above, the staff concludes that the proposed Technical Specifications changes are acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such

activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: W. LeFave

Date: May 20, 1994