December 28, 1994

DISTRIBUTION
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Mr. J. H. Goldberg
President-Nuclear Division
Florida Power and Light Company
P.O. Box 14000
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS RE: SCHEDULE FOR THE WITHDRAWAL OF REACTOR VESSEL MATERIAL SPECIMENS FROM TECHNICAL SPECIFICATIONS (TAC NOS. M90681 AND M90682)

Dear Mr. Goldberg:

The Commission has issued the enclosed Amendment No. 170 to Facility Operating License No. DPR-31 and Amendment No. 164 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated October 20, 1994, relating to removal of the schedule for the withdrawal of reactor vessel material specimens from TS as discussed in Generic Letter 91-01.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

(Original Signed By)

Richard P. Croteau, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-250
and 50-251

Enclosures:
1. Amendment No.170 to DPR-31
2. Amendment No.164 to DPR-41
3. Safety Evaluation

cc w/enclosures: See next page

12/28/94

Document Name: G:\TP90681.AMD

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NAME	Dunnington	Croteau	Thadani	MZOBLER	JStrosnider
DATE	11/ / /94	11/2 /94	11/29/94	11/21/94	11/ 8 /94
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DATED: December 28, 1994

AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-31-TURKEY POINT UNIT 3
AMENDMENT NO. 164 TO FACILITY OPERATING LICENSE NO. DPR-41-TURKEY POINT UNIT 4

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 170
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated October 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

   (B) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 170, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

DBMatthews

David B. Matthews, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1994

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 164
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated October 20, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

   (B) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A, as revised through Amendment No. 164, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

David B. Matthews, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 28, 1994

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 170 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 164 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove pages	Insert pages
viii	viii
3/4 4-30	3/4 4-30
3/4 4-34	3/4 4-34
B 3/4 4-8	B 3/4 4-8
B 3/4 4-9	B 3/4 4-9
B 3/4 4-15	B 3/4 4-15

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

REACTOR COOLANT SYSTEM

3/4.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

LIMITING CONDITION FOR OPERATION

3.4.9.1 The Reactor Coolant System (except the pressurizer) temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4-2 and 3.4-3 and 3.4-4 during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:

a. A maximum heatup of 100°F in any 1-hour period,

b. A maximum cooldown of 100°F in any 1-hour period, and

c. A maximum temperature change of less than or equal to 5°F in any 1-hour period during inservice hydrostatic and leak testing operations above the heatup and cooldown limit curves.

APPLICABILITY: At all times.

ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limit within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operation or be in at least HOT STANDBY within the next 6 hours and reduce the RCS T_{avg} and pressure to less than 200°F and 500 psig, respectively, within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.9.1.1 The Reactor Coolant System temperature and pressure shall be determined to be within the limits at least once per 30 minutes during system heatup, cooldown, and inservice leak and hydrostatic testing operations.

4.4.9.1.2 The reactor vessel material irradiation surveillance specimens shall be removed and examined, to determine changes in material properties, as required by 10 CFR Part 50, Appendix H. The results of these examinations shall be used to update Figures 3.4-2, 3.4-3 and 3.4-4.

TABLE 4.4-5

REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM - WITHDRAWAL SCHEDULE

(Deleted)

PRESSURE/TEMPERATURE LIMITS (Continued)

1. The reactor coolant temperature and pressure and system heatup and cooldown rates (with the exception of the pressurizer) shall be limited in accordance with Figures 3.4-2 to 3.4-4 for the service period specified thereon:

   a. Allowable combinations of pressure and temperature for specific temperature change rates are below and to the right of the limit lines shown. Limit lines for cooldown rates between those presented may be obtained by interpolation; and

   b. Figures 3.4-2 to 3.4-4 define limits to assure prevention of non-ductile failure only. For normal operation, other inherent plant characteristics, e.g., pump heat addition and pressurizer heater capacity, may limit the heatup and cooldown rates that can be achieved over certain pressure-temperature ranges.

2. These limit lines shall be calculated periodically using methods provided below,

3. The secondary side of the steam generator must not be pressurized above 200 psig if the temperature of the steam generator is below 70°F,

4. The pressurizer heatup and cooldown rates shall not exceed 100°F/h and 200°F/h, respectively. The spray shall not be used if the temperature difference between the pressurizer and the spray fluid is greater than 320°F, and

5. System preservice hydrotests and inservice leak and hydrotests shall be performed at pressures in accordance with the requirements of ASME Boiler and Pressure Vessel Code, Section XI.

The fracture toughness properties of the ferritic materials in the reactor vessel are determined in accordance with the NRC Standard Review Plan, the version of the ASTM E185 standard required by 10 CFR 50 Appendix H, and in accordance with additional reactor vessel requirements.

The properties are then evaluated in accordance with Appendix G of the 1983 Edition of Section III of the ASME Boiler and Pressure Vessel Code and the additional requirements of 10 CFR 50, Appendix G and the calculation methods described in Westinghouse Report GTSD-A-1.12, "Procedure for Developing Heatup and Cooldown Curves."

Heatup and cooldown limit curves are calculated using the most limiting value of the nil-ductility reference temperature, RT_{NDT}, at the end of 20 effective full power years (EFPY) of service life. The 20 EFPY service life period is chosen such that the limiting RT_{NDT} at the 1/4T location in

PRESSURE/TEMPERATURE LIMITS (Continued)

the core region is greater than the RT_{NDT} of the limiting unirradiated material. The selection of such a limiting RT_{NDT} assures that all components in the Reactor Coolant System will be operated conservatively in accordance with applicable Code requirements.

The heatup and cooldown limit curves, Figures 3.4-2, 3.4-3 and 3.4-4 are composite curves prepared by determining the most conservative case with either the inside or outside wall controlling, for any heatup rate up to 100 degrees F per hour and cooldown rates of up to 100 degrees F per hour. The heatup and cooldown curves were prepared based upon the most limiting value of predicted adjusted reference temperature at the end of the applicable service period (20 EFPY).

The reactor vessel materials have been tested to determine their initial RT_{NDT}; the results of these tests are shown in Tables B 3/4.4-1 and B 3/4.4-2. Reactor operation and resultant fast neutron (E greater than 1 MeV) irradiation can cause an increase in the RT_{NDT}. Therefore, an adjusted reference temperature, based upon the fluence and chemistry factors of the material has been predicted using Regulatory Guide 1.99, Revision 2, dated May 1988, "Radiation Embrittlement of Reactor Vessel Materials." The heatup and cooldown limit curves of Figures 3.4-2, 3.4-3, and 3.4-4 include predicted adjustments for this shift in RT_{NDT} at the end of the applicable service period.

The actual shifts in RT_{NDT} of the vessel materials will be established periodically during operation by removing and evaluating, in accordance with the version of the ASTM E185 standard required by 10 CFR 50 Appendix H, reactor vessel material irradiation surveillance specimens installed near the inside wall of the reactor vessel in the core area. Since the neutron spectra at the irradiation samples and vessel inside radius are essentially identical, the measured transition shift for a sample can be applied with confidence to the adjacent section of the reactor vessel.

Since the limiting beltline materials (Intermediate to Lower Shell Circumferential Weld) in Units 3 and 4 are identical, the RV surveillance program was integrated and the results from capsule testing is applied to both Units. The surveillance capsule "T" results from Unit 3 (WCAP 8631) and Unit 4 (SWRI 02-4221) and the capsule "V" results from Unit 3 (SWRI 06-8576) were used with the methodology in Regulatory Guide 1.99, Revision 2, to provide

PRESSURE/TEMPERATURE LIMITS (Continued)

Finally, the 10 CFR 50 Appendix G rule which addresses the metal temperature of the closure head flange and vessel flange regions is considered. The rule states that the minimum metal temperature for the flange regions should be at least 120 F higher than the limiting RT_{NDT} for these regions when the pressure exceeds 20 percent of the preservice hydrostatic test pressure (621 psig). Since the limiting RT_{NDT} for the flange regions for Turkey Point Units 3 and 4 is 44 F, the minimum temperature required for pressure of 621 psig and greater based on the Appendix G rule is 164 F. The heatup and cooldown curves as shown in Figures 3.4-2 to 3.4-4 clearly satisfy the above requirement by ample margins.

Finally, the composite curves for the heatup rate data and the cooldown rate data are adjusted for possible errors in the pressure and temperature sensing instruments by the values indicated on the respective curves.

The limitations imposed on the pressurizer heatup and cooldown rates and spray water temperature differential are provided to assure that the pressurizer is operated within the design criteria assumed for the fatigue analysis performed in accordance with the ASME Code requirements.

OVERPRESSURE MITIGATING SYSTEM

The Technical Specifications provide requirements to isolate High Pressure Safety Injection from the RCS and to prevent the start of an idle RCP if secondary temperature is more than 50°F above the RCS cold leg temperatures. These requirements are designed to ensure that mass and heat input transients more severe than those assumed in the low temperature overpressurization protection analysis cannot occur.

The OPERABILITY of two PORVs or an RCS vent opening of at least 2.20 square inches ensures that the RCS will be protected from pressure transients which could exceed the limits of Appendix G to 10 CFR Part 50 when one or more of the RCS cold legs are less than or equal to 275°F. Either PORV has adequate relieving capability to protect the RCS from overpressurization when the transient is limited to either: (1) the start of an idle RCP with the secondary water temperature of the steam generator less than or equal to 50°F above the RCS cold leg temperatures including margin for instrument error, or (2) the start of a HPSI pump and its injection into a water-solid RCS. When the PORVs or 2.2 square inch area vent is used to mitigate a plant transient, a Special Report is submitted. However, minor increases in pressure resulting from planned plant actions, which are relieved by designated openings in the system, need not be reported.

REACTOR MATERIAL SURVEILLANCE PROGRAM

Each Type I capsule contains 28 V-notch specimens, ten Charpy specimens machined from each of the two shell forgings. The remaining eight Charpy specimens are machined from correlated monitor material. In addition, each

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 170 TO FACILITY OPERATING LICENSE NO. DPR-31

AND AMENDMENT NO.164 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By letter dated October 20, 1994, Florida Power and Light Company (FPL or the licensee) proposed a change to the Technical Specifications (TS) for Turkey Point Units 3 and 4. The proposed change removes TS Table 4.4-5 which provides the schedule for reactor vessel material specimen withdrawal. Guidance on the proposed TS change was provided by Generic Letter (GL) 91-01, of January 4, 1990, to all holders of operating licenses or construction permits for nuclear power reactors.

2.0 BACKGROUND

Ferritic materials, such as those found in the beltline region of the reactor vessel, become more susceptible to brittle fracture with exposure to neutron flux. An operating reactor creates neutron flux levels which reduce the fracture toughness of the ferritic materials. To prevent fracture of the vessel, stress on the material is limited by controlling the operating pressures, temperatures, and rate of change of these parameters during heatup and cooldown. Tighter controls on these parameters are necessary to maintain adequate levels of safety with exposure to neutron flux during vessel life.

The reactor vessel material surveillance program involves placing test specimens of ferritic materials either the same or similar to those used in the construction of the reactor vessel inside the reactor vessel. The test specimens are irradiated with neutron flux equivalent to the material of the reactor vessel itself. By removing and testing the fracture toughness of these specimens, the condition of the reactor vessel can be determined. The operating pressures, temperatures, and other pertinent parameters are then modified, based on the tests, to maintain adequate levels of safety.

GL 91-01 provided guidance for removal of the schedule for the withdrawal of the reactor vessel material specimens from the TS since 10 CFR 50 Appendix H requires that the proposed withdrawal schedule be submitted and approved by the NRC prior to implementation. The GL also stated that reference to the use of the examinations to update the TS figures for the pressure and temperature operating limits should be retained in the TS. Also, a copy of the NRC-approved version of the specimen withdrawal schedule should be included in the next revision to the FSAR to provide a readily available copy.

3.0 EVALUATION

3.1 Relation to Pressure and Temperature Limits

The Turkey Point TS include pressure and temperature limits for the reactor coolant system (RCS). TS 3/4.4.9.1, "Pressure/Temperature Limits," limits the RCS temperature and pressure and their rate of change to values consistent with the fracture toughness requirements of Appendix G of the 1983 Edition of Section III of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code and the additional requirements of Appendix G to 10 CFR Part 50. The limits are shown in TS Figures 3.4-2, 3.4-3, and 3.4-4 and provide an acceptable range of operating temperatures and pressures for heatup, cooldown, criticality, and inservice leak and hydrostatic testing. The current limits in these TS figures are valid up to 20 Effective Full-Power Years (EFPY) of operation. Periodic updates of these limits are necessary as previously discussed in Section 2.0. This assists in fulfilling the requirements of Appendix H to 10 CFR Part 50 to prevent brittle fracture of the reactor vessel.

Current TS also include Surveillance Requirement 4.4.9.1.2, which requires the removal and examination of reactor vessel material irradiation surveillance specimens to determine changes in material properties in accordance with the schedule in TS Table 4.4-5 and the requirements of 10 CFR Part 50, Appendix H. The results of these examinations are used to update TS Figures 3.4-2, 3.4-3, and 3.4-4.

The BASES state that the heatup and cooldown curves are recalculated when data from the surveillance specimens indicate a change in material properties that exceeds the limiting value of those properties that were used to develop the existing pressure and temperature limits. TS BASES also provide background information on the use of the data obtained from material specimens. This background information clearly defines the purpose and relationship of this information to the requirements included in the regulations and the ASME Code. Therefore, the removal of the schedule for specimen withdrawal from the TS will not result in any loss of clarity related to regulatory requirements of Appendix H to 10 CFR Part 50.

3.2 Control of Changes to Specimen Withdrawal Schedule

The Turkey Point Facility Operating License states that the license shall be deemed to contain and is subject to the conditions specified in all applicable provisions of the rules, regulations and orders of the Commission. Section II.B.3 of Appendix H to 10 CFR Part 50 requires the submittal of a proposed withdrawal schedule for material specimens to the NRC and approval by the NRC before implementation but does not specifically state that this applies to changes to the withdrawal schedule. ASTM E 185 was incorporated by reference in Appendix H, and it is intended that licensee withdrawal schedules are consistent with the schedule criteria contained in ASTM E 185-79 or 185-82. After a licensee has removed its withdrawal schedule from its TS, it may proceed to make changes to its schedule which are consistent with ASTM E 185-79 or 185-82 without prior NRC approval and report those changes in a manner consistent with 10 CFR 50.59, provided the licensee determines that an unreviewed safety question does not exist. If the changes to the withdrawal schedule are not consistent with ASTM E 185 revisions referenced in Appendix H, the changes would likely be deemed to involve an unreviewed safety question and would require prior NRC approval in the form of an amendment to the licensee, as provided in 10 CFR 50.59(c). These regulatory controls are adequate to control changes to this schedule without the necessity of including the schedule in TS.

In accordance with GL 91-01, the licensee committed to include a copy of the NRC-approved version of the specimen withdrawal schedule in the next revision to the FSAR to provide a readily available copy. Also, reference to the use of the examinations to update the TS figures for the pressure and temperature operating limits are retained in the TS.

In addition to the implementation of the line-item improvements proposed in GL 91-01, FPL proposed to reference 10 CFR 50, Appendix H, rather than list the specific revision of ASTM E 185 in the TS BASES. This is acceptable since Appendix H controls the revision to be used as previously discussed and inclusion of the revision in the Bases is not necessary.

The staff finds these changes acceptable since inclusion of the withdrawal schedules in the TS (a) is not required by 10 CFR 50.36, other regulations, or the four criteria from the NRC's Policy Statement on TS improvements (58 FR 39132), (b) is not required to avert an immediate threat to the public health and safety, and (c) is not necessary since Appendix H provides an adequate means of controlling proposed changes to withdrawal schedules. 10 CFR 50, Appendix H, also controls the revision to ASTM E 185 that must be used. These changes are consistent with GL 91-01.

As a consequence of deletions and previous typographical errors, administrative changes have been proposed. The staff finds these changes acceptable as they are editorial in nature.

4.0 CONCLUSION

We find the requested changes acceptable since 10 CFR 50, Appendix H, provides an adequate means of controlling proposed changes to withdrawal schedules and inclusion in TS in not necessary. The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of these amendments will not be inimical to the common defense and security or the health and safety of the public.

5.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

6.0 ENVIRONMENTAL CONSIDERATION

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 60381). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

Principal Contributor: T. Dunning, R. Croteau

Date: December 28, 1994