



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

February 25, 1994

Docket Nos. 50-250  
and 50-251

Mr. J. H. Goldberg  
President - Nuclear Division  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS RE:  
FIRE PROTECTION PROGRAM (TAC NOS. M87314 AND M87315)

The Commission has issued the enclosed Amendment No. 159 to Facility Operating License No. DPR-31 and Amendment No. 153 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Units Nos. 3 and 4, respectively. These amendments consist of changes to licenses DPR-31 and DPR-41 and to the Technical Specifications (TS) in response to your application dated August 17, 1993, to relocate fire protection requirements from the TS to the Final Safety Analysis Report in accordance with Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements."

A copy of the Safety Evaluation (SE) is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Please note that your submittal does not explicitly address TS for implementing alternative safe shutdown equipment, consistent with GL 81-12. Our SE is independent of this alternative safe shutdown equipment TS which will be a subject of our future discussion.

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P PDR

DFOI

Also, by letter dated January 12, 1994, you submitted to the staff a proposed performance-based approach for resolving the Thermo-Lag fire barrier issues at the Turkey Point and St. Lucie plants. We have not made any technical determination on the acceptability of your proposed approach. Pending this determination, you should not use the fire protection license condition approved in this amendment to implement your proposed approach relating to Thermo-Lag without prior NRC review.

If you have any questions please call me at (301) 504-1471.

Sincerely,



L. Raghavan, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 159 to DPR-31
- 2. Amendment No. 153 to DPR-41
- 3. Safety Evaluation

cc w/enclosures:  
See next page

Document Name - TP87314.AMD

Distribution

See next page

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Mr. J. H. Goldberg  
Florida Power and Light Company

Turkey Point Plant

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DATED: February 25, 1994

AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-31-TURKEY POINT UNIT 3  
AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NO. DPR-41-TURKEY POINT UNIT 4

~~Docket File~~

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 159  
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated August 17, 1993, as supplemented January 14, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 159, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. Also, the license is amended by replacing Paragraph 3.G. on page 5 of Facility Operating License DPR-31 with the following:\*

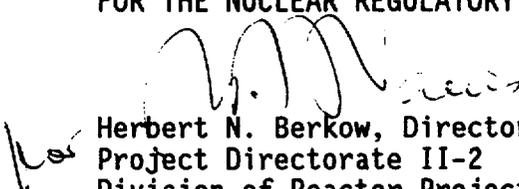
(G) Fire Protection

FPL shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Final Safety Analysis Report (UFSAR) for Turkey Point Units 3 and 4 and as approved in the Safety Evaluation Report (SER) dated March 21, 1979 and supplemented by NRC letters dated April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and by Safety Evaluation dated February 25, 1994, subject to the following provision.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

4. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachments:

1. Page 5 of License DPR-31
2. Changes to the Technical Specifications

Date of Issuance: February 25, 1994

\*Page 5 is attached, for convenience, for the composite license to reflect this change.

G. Fire Protection

FPL shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Final Safety Analysis Report (UFSAR) for Turkey Point Units 3 and 4 and as approved in the Safety Evaluation Report (SER) dated March 21, 1979 and supplemented by NRC letters dated April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and by Safety Evaluation dated February 25, 1994, subject to the following provision.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

H. Safeguards Contingency Plan

DELETED (per Amendment No. 125, 7-5-88)

I. Steam Generator Repair Program

- (1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 3 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.
- (2) During the repair program the following temporary license conditions\* will be imposed:

\*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153  
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated August 17, 1993, as supplemented January 14, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No.153, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. Also, the license is amended by replacing Paragraph 3.F. on page 6 of Facility Operating License DPR-41 with the following:\*

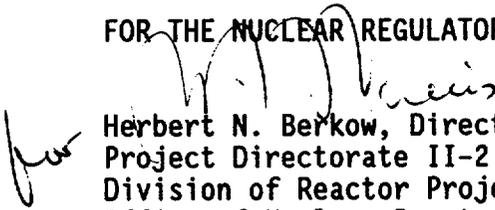
(F) Fire Protection

FPL shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Final Safety Analysis Report (UFSAR) for Turkey Point Units 3 and 4 and as approved in the Safety Evaluation Report (SER) dated March 21, 1979 and supplemented by NRC letters dated April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and by Safety Evaluation dated February 25, 1994, subject to the following provision.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

4. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachments:

1. Page 6 of License DPR-41
2. Changes to the Technical Specifications

Date of Issuance: February 25, 1994

\*Page 6 is attached, for convenience, for the composite license to reflect this change.

E. DELETED (per Amendment No. 125, 7-5-88)

F. Fire Protection

FPL shall implement and maintain in effect all provisions of the approved Fire Protection Program as described in the Updated Final Safety Analysis Report (UFSAR) for Turkey Point Units 3 and 4 and as approved in the Safety Evaluation Report (SER) dated March 21, 1979 and supplemented by NRC letters dated April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and by Safety Evaluation dated February 25, 1994, subject to the following provision.

The licensee may make changes to the approved Fire Protection Program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

G. Safeguards Contingency Plan

DELETED (per Amendment No. 125, 7-5-88)

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 159 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 153 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

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**INSTRUMENTATION**

**FIRE DETECTION INSTRUMENTATION**

**LIMITING CONDITION FOR OPERATION**

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3/4.3.3.4 - (Deleted)

**TABLE 3.3-6**  
**FIRE DETECTION INSTRUMENTS**  
**FOR ESSENTIAL EQUIPMENT**

**(Deleted)**

TABLE 3.3-6 (Continued)  
FIRE DETECTION INSTRUMENTS  
FOR ESSENTIAL EQUIPMENT

(Deleted)

## ADMINISTRATIVE CONTROLS

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### PLANT STAFF

6.2.2 The plant organization shall be subject to the following:

- a. Each on-duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1;
- b. At least one licensed Operator shall be in the control room when fuel is in either reactor.
- c. At least two licensed Operators shall be present in the control room during reactor startup, scheduled reactor shutdown and during recovery from reactor trips. In addition, while either unit is in MODE 1, 2, 3, or 4, at least one licensed Senior Operator shall be in the control room;
- d. A Health Physics Technician\* shall be on site when fuel is in the reactor;
- e. All CORE ALTERATIONS shall be observed and directly supervised by either a licensed Senior Operator or licensed Senior Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation; and
- f. Administrative procedures shall be developed and implemented to limit the working hours of plant staff who perform safety-related functions (e.g., licensed Senior Operators, licensed Operators, health physicists, auxiliary operators, and key maintenance personnel).

Adequate shift coverage shall be maintained without routine heavy use of overtime. The objective shall be to have operating personnel work a normal 8-hour day, 40-hour week while the plant is operating. However, in the event that unforeseen problems require substantial amounts of overtime to be used, or during extended periods of shutdown for refueling, major maintenance, or major plant modification, on a temporary basis the following guidelines shall be followed:

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\* The Health Physics Technician composition may be less than the minimum requirements for a period of time not to exceed 2 hours, in order to accommodate unexpected absence, provided immediate action is taken to fill the required positions.

## ADMINISTRATIVE CONTROLS

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### RESPONSIBILITIES (Continued)

- e. Investigation of all violations of the Technical Specifications, including the preparation and forwarding of reports covering evaluation and recommendations to prevent recurrence, to the President-Nuclear Division and to the Chairman of the Company Nuclear Review Board;
- f. Review of all REPORTABLE EVENTS;
- g. Review of reports of significant operating abnormalities or deviations from normal and expected performance of plant equipment or systems that affect nuclear safety.
- h. Performance of special reviews, investigations, or analyses and reports thereon as requested by the Plant General Manager or the Chairman of the Company Nuclear Review Board;
- i. Review of the Emergency Plan and implementing procedures and submittal of recommended changes to the Chairman of the Company Nuclear Review Board;
- j. Review of changes to the PROCESS CONTROL PROGRAM and the OFFSITE DOSE CALCULATION MANUAL;
- k. Review of any accidental, unplanned, or uncontrolled radioactive release including the preparation of reports covering evaluation, recommendations, and disposition of the corrective action to prevent recurrence and the forwarding of these reports to the President-Nuclear Division and to the Chairman of the Company Nuclear Review Board.
- l. Review of the Fire Protection Program and implementing procedures and the submittal of recommended changes to the Company Nuclear Review Board.

#### 6.5.1.7 The PNSC shall:

- a. Recommend in writing to the Plant General Manager approval or disapproval of items considered under Specification 6.5.1.6a. through d. prior to their implementation and items considered under Specification 6.5.1.6i through l.
- b. Provide written notification within 24 hours to the Plant General Manager, President-Nuclear Division and the Company Nuclear Review Board of disagreement between the PNSC and the Plant General Manager; however, the Plant General Manager shall have responsibility for resolution of such disagreements pursuant to Specification 6.1.1.

## INSTRUMENTATION

### BASES

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#### 3/4.3.3 MONITORING INSTRUMENTATION

##### 3/4.3.3.1 RADIATION MONITORING FOR PLANT OPERATIONS

The OPERABILITY of the radiation monitoring instrumentation for plant operations ensures that conditions indicative of potential uncontrolled radioactive releases are monitored and that appropriate actions will be automatically or manually initiated when the radiation level monitored by each channel reaches its alarm or trip setpoint.

##### 3/4.3.3.2 MOVABLE INCORE DETECTORS

The OPERABILITY of the movable incore detectors with the specified minimum complement of equipment ensures that the measurements obtained from use of this system accurately represent the spatial neutron flux distribution of the core. The OPERABILITY of this system is demonstrated by irradiating each detector used and determining the acceptability of its voltage curve.

For the purpose of measuring  $F_Q(Z)$  or  $F_{\Delta H}^N$  a full incore flux map is used. Quarter-core flux maps, as defined in WCAP-8648, June 1976, may be used in recalibration of the Excore Neutron Flux Detection System, and full incore flux maps or symmetric incore thimbles may be used for monitoring the QUADRANT POWER TILT RATIO when one Power Range channel is inoperable.

##### 3/4.3.3.3 ACCIDENT MONITORING INSTRUMENTATION

The OPERABILITY of the accident monitoring instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables following an accident. This capability is consistent with the recommendations of Regulatory Guide 1.97, Revision 3, "Instrumentation for Light-Water-Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," May 1983 and NUREG-0737, "Clarification of TMI Action Plan Requirements," November 1980.

##### 3/4.3.3.4 FIRE DETECTION INSTRUMENTATION - (Deleted)

## BASES

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### 3/4.3.3.5 RADIOACTIVE LIQUID EFFLUENT MONITORING INSTRUMENTATION

The radioactive liquid effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in liquid effluents during actual or potential releases of liquid effluents. The Alarm/Trip Setpoints for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50.

### 3/4.3.3.6 RADIOACTIVE GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The radioactive gaseous effluent instrumentation is provided to monitor and control, as applicable, the releases of radioactive materials in gaseous effluents during actual or potential releases of gaseous effluents. The Alarm/Trip Setpoints for these instruments shall be calculated and adjusted in accordance with the methodology and parameters in the ODCM to ensure that the alarm/trip will occur prior to exceeding the limits of 10 CFR Part 20. This instrumentation also includes provisions for monitoring (and controlling) the concentrations of potentially explosive gas mixtures in the GAS DECAY TANK SYSTEM. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria 60, 63, and 64 of Appendix A to 10 CFR Part 50. The sensitivity of any noble gas activity monitors used to show compliance with the gaseous effluent release requirements of Specification 3.11.2.2 shall be such that concentrations as low as  $1 \times 10^{-6}$   $\mu\text{Ci/ml}$  are measurable.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 159 TO FACILITY OPERATING LICENSE NO. DPR-31  
AND AMENDMENT NO. 153 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

Generic Letter (GL) 86-10, "Implementation of Fire Protection Requirements," dated April 24, 1986, recommended that licensees remove the fire protection requirements, including the fire hazards analysis and major commitments that form the basis for the Fire Protection Program, from the facility Technical Specifications (TS) and incorporate them in their Final Safety Analysis Reports (FSARs). GL 88-12, "Removal of Fire Protection Requirements from Technical Specifications," dated August 2, 1988, provided guidance for a license amendment submittal to implement GL 86-10.

By letter dated August 17, 1993, Florida Power and Light Company (FPL or the licensee) proposed changes to the TS for the Turkey Point Plant, Units 3 and 4, to implement GL 86-10. By letter dated January 14, 1994, the licensee submitted clarifying information which did not change the staff's initial no significant hazards determination.

2.0 DISCUSSION

The TS changes proposed by the licensee are as follows:

1. Delete TS 3.3.3.4 (Fire Detection Instrumentation), TS 3.7.8.1 (Fire Suppression Water System), TS 3.7.8.2 (Spray and/or Sprinkler Systems), TS 3.7.8.3 (Fire Hose Stations), TS 3.7.8.4, (Fire Hydrants and Hydrant Hose Houses), and TS 3.7.9 (Fire Rated Assemblies) and their associated bases and surveillances, and incorporate them into the FSAR.
2. Delete TS 6.2.2.f for site fire brigade staffing and incorporate into the FSAR.
3. Add TS 6.5.1.7.1 to include the review of the fire protection program and implementing procedures as an additional responsibility of the Company Nuclear Review Board (CNRB).

The licensee also proposed the following fire protection license condition:

Florida Power and Light Company shall implement and maintain in effect all provisions of the approved fire protection program as described in the Final Safety Analysis Report for the facility and as approved in the SERs dated March 21, 1979, April 3, 1980, July 9, 1980, December 8, 1980, January 26, 1981, May 10, 1982, March 27, 1984, April 16, 1984, August 12, 1987, and February 25, 1994, respectively, subject to the following provision:

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

Other editorial changes to the TS, to reflect the proposed changes, would also be made.

### 3.0 EVALUATION

GL 86-10 recommended removing fire protection requirements from the TS. Although a comprehensive Fire Protection Program is essential to plant safety, the basis for the recommendation to relocate these requirements from the TS is that many details of this program that are currently addressed in TS do not constitute performance requirements necessary to ensure safe operation of the facility and, therefore, can be modified without affecting nuclear safety. Such modifications can be made provided that there are suitable administrative controls over these changes to ensure that changes to the Fire Protection Program would be based on careful review and analysis by competent individuals such that nuclear safety is not adversely affected. These controls relating to the Fire Protection Program include: (1) the TS administrative controls; (2) the license condition on implementation of and subsequent changes; and (3) the 10 CFR 50.59 criteria for evaluating changes.

The NRC staff reviewed the license amendment request against the guidance provided in GLs 86-10 and 88-12. GL 86-10 requested that the licensee incorporate the NRC-approved fire protection program in its FSAR for the facility and specified a standard fire protection license condition. GL 88-12 addressed the elements a licensee should include in a license amendment request to remove the fire protection requirements from the plant TS. These elements are: (1) the NRC-approved fire protection program must be incorporated into the FSAR; (2) the Limiting Conditions of Operations and Surveillance Requirements associated with fire detection systems, fire suppression systems, fire barriers, and the administrative controls that address fire brigade staffing would be deleted from the TS (the existing administrative controls related to fire protection audits would be retained in the TS); (3) all operational conditions, remedial actions, and test requirements presently included in the TS for these systems, as well as the fire brigade staffing requirements, shall be incorporated into the fire protection program; (4) the standard fire protection license condition specified in GL 86-10 must be included in the facility operating license; (5) the Unit Review Group (Onsite Review Group) shall be given responsibility for the review of the fire protection program and implementing procedures and for the submittal

of recommended changes to the Company Nuclear Review and Audit group (Off-site or Corporate Review Group); and (6) fire protection program implementation shall be added to the list of elements for which written procedures shall be established, implemented, and maintained.

The licensee incorporated the NRC-approved fire protection program for the facilities into its FSAR in July 1987. The licensee has, therefore, satisfied Element 1 of GL 88-12.

The licensee will incorporate the current TS operability and surveillance requirements for the fire detection systems, fire suppression systems, fire rated assemblies, and the TS requirements related to fire brigade staffing into the FSAR. Therefore, the licensee has satisfied Elements 1, 2, and 3 of GL 88-12.

The licensee proposed the standard fire protection license condition specified in GL 86-10 and, therefore, satisfied Element 4 of GL 88-12.

To satisfy Elements 5 and 6 of GL 88-12, the licensee will add TS 6.5.3.8.1 to include the review of the fire protection program and implementing procedures as an additional responsibility of the CNRB. Element 6 of GL 88-12 specified that the licensee add fire protection program implementation to the list of elements for which written procedures shall be established, implemented, and maintained. TS 6.8.1.h currently addresses the fire protection program; therefore, no changes are required. The licensee has, therefore, satisfied Elements 5 and 6 of GL 88-12.

The licensee also proposed to maintain TS 6.9.2, which relates to the requirement of special reports of fire detection instrumentation, fire suppression systems and fire rated assemblies.

Based on the above discussion, the licensee's proposed TS amendment is in accordance with NRC staff guidance provided in GLs 86-10 and 88-12 and therefore, acceptable.

#### 5.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

#### 4.0 ENVIRONMENTAL CONSIDERATION

These amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration and there has been no public comment on such finding (58 FR 50967). Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be

prepared in connection with the issuance of these amendments.

#### 6.0 CONCLUSION

Based on its evaluation in Section 2.0 above, the staff concludes that the proposed TS changes are acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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