

December 18, 2001

Mr. Oliver D. Kingsley, President
Exelon Nuclear
Exelon Generation Company, LLC
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: ISSUANCE OF AMENDMENTS - REACTOR VESSEL LOW WATER LEVEL
SETPOINTS FOR DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3,
AND QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
(TAC NOS. MB1272, MB1273, MB1270, AND MB1271)

Dear Mr. Kingsley:

The Commission has issued the enclosed Amendment No. 190 to Facility Operating License No. DPR-19 and Amendment No. 184 to Facility Operating License No. DPR-25 for Dresden, Units 2 and 3, respectively, and Amendment No. 200 to Facility Operating License No. DPR-29 and Amendment No. 196 to Facility Operating License No. DPR-30 for the Quad Cities Nuclear Power Station, Units 1 and 2, respectively. The amendments are in response to your application dated February 22, 2001, as supplemented by your letters dated May 4, and September 13, 2001.

The amendments revise reactor vessel water level - low scram and isolation setpoints in order to minimize unnecessary reactor scrams that might result from events involving a temporary reduction in feedwater flow. The revision to these setpoints was originally requested as part of the power uprate licensing amendment. However, since the setpoint reduction will provide a similar benefit when operating at the current power level, you requested that the changes be reviewed separately from the power uprate. This will allow earlier implementation of the change and improve reliability at the plants.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,
/RA/
Mahesh Chawla, Project Manager, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-237, 50-249, 50-254, 50-265

Enclosures: 1. Amendment No. 190 to DPR-19
2. Amendment No. 184 to DPR-25
3. Amendment No. 200 to DPR-29
4. Amendment No. 196 to DPR-30
5. Safety Evaluation

cc w/encls: See next page

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DISTRIBUTION

- Enclosures: 1. Amendment No. 190 to DPR-19
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- 3. Amendment No. 200 to DPR-29
- 4. Amendment No. 196 to DPR-30
- 5. Safety Evaluation

- PUBLIC G. Hill (8)
- PD3-2 r/f ACRS
- A. Mendiola OGC
- L. Rossbach W. Beckner
- S. Bailey M. Ring, RIII
- C. Rosenberg R. Caruso
- M. Chawla E. Marinos

cc w/encls: See next page

ADAMS Accession Number: ML013380352 *See Previous Concurrence Sheet

OFFICE	PM:LPD3-2	PM:LPD3-2	LA:LPD3-2	SC:EEIB	SC:SRXB
NAME	MChawla*	LRossbach	THarris for* CRosenberg	HGarg for* EMarinos	RCaruso*
DATE	12/11/01	12/18/01	12/5/01	12/06/01	12/11/01

OFFICE	PM:LPD3-2	OGC	SC:LPD3-2
NAME	SBailey	RHoefling*	AMendiola
DATE	12/17/01	12/14/01	12/18/01

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EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-237

DRESDEN NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 190
License No. DPR-19

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated February 22, 2001, as supplemented by letters dated May 4, and September 13, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-19 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 190 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 2001

EXELON GENERATION COMPANY, LLC

DOCKET NO. 50-249

DRESDEN NUCLEAR POWER STATION, UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 184
License No. DPR-25

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Exelon Generation Company, LLC (the licensee) dated February 22, 2001, as supplemented by letters dated May 4, and September 13, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 3.B. of Facility Operating License No. DPR-25 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 184 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 2001

ATTACHMENT TO LICENSE AMENDMENT NOS. 190 AND 184

FACILITY OPERATING LICENSE NOS. DPR-19 AND DPR-25

DOCKET NOS. 50-237 AND 50-249

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.3.1.1-9
3.3.6.1-5
3.3.6.1-7
3.3.6.2-4

INSERT

3.3.1.1-9
3.3.6.1-5
3.3.6.1-7
3.3.6.2-4

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-254

QUAD CITIES NUCLEAR POWER STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 200
License No. DPR-29

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated February 22, 2001, as supplemented by letters dated May 4, and September 13, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-29 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 200 are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 2001

EXELON GENERATION COMPANY, LLC

AND

MIDAMERICAN ENERGY COMPANY

DOCKET NO. 50-265

QUAD CITIES NUCLEAR POWER STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 196
License No. DPR-30

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Exelon Generation Company, LLC (the licensee) dated February 22, 2001, as supplemented by letters dated May 4, and September 13, 2001, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-30 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 196 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 30 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/RA/

Anthony J. Mendiola, Chief, Section 2
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 18, 2001

ATTACHMENT TO LICENSE AMENDMENT NOS. 200 AND 196

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

DOCKET NOS. 50-254 AND 50-265

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the attached pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3.3.1.1-8
3.3.6.1-5
3.3.6.1-7
3.3.6.2-4
3.3.7.1-4

INSERT

3.3.1.1-8
3.3.6.1-5
3.3.6.1-7
3.3.6.2-4
3.3.7.1-4

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 190 TO FACILITY OPERATING LICENSE NO. DPR-19,
AMENDMENT NO. 184 FACILITY OPERATING LICENSE NO. DPR-25,
AMENDMENT NO. 200 TO FACILITY OPERATING LICENSE NO. DPR-29
AND AMENDMENT NO. 196 TO FACILITY OPERATING LICENSE NO. DPR-30
EXELON GENERATION COMPANY, LLC
AND
MIDAMERICAN ENERGY COMPANY
DRESDEN NUCLEAR POWER STATION, UNITS 2 AND 3, AND
QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
DOCKET NOS. 50-237, 50-249, 50-254 AND 50-265

1.0 INTRODUCTION

By letter dated February 22, 2001, as supplemented by letters dated May 4, and September 13, 2001, Exelon Generation Company, LLC (Exelon, the licensee) submitted proposed Technical Specification (TS) changes for Dresden Nuclear Power Station, Units 2 and 3 (Dresden), and Quad Cities Nuclear Power Station, Units 1 and 2 (Quad Cities). The proposed amendments would allow a reduction in the reactor vessel low water level scram and isolation setpoints. The licensee previously submitted this change with its power uprate application, but then decided to request this change separately from the power uprate. This change reduces unnecessary reactor scrams and associated engineered safety feature actuations that might result from events involving a temporary reduction in feedwater flow.

The intent of the proposed TS changes is to increase the operating range between the normal reactor vessel water level and the reactor vessel water level-low trip function. The increased range will allow operators or automatic features additional time to respond to recoverable transients and thus avert unnecessary transients.

Industry studies have identified low water level scrams as being initiators of a significant number of plant trips. The Boiling Water Reactor Owners Group Scram Frequency Reduction Committee identified some of these scrams as unnecessary since the reactor water level would have stabilized above the top of the active fuel and recovered to normal water level without the scram.

The supplemental letters contained clarifying information and did not change the initial no significant hazards consideration determination and did not expand the scope of the original *Federal Register* notice.

2.0 BACKGROUND INFORMATION

During normal power operation, significant changes in reactor vessel water level can occur due to pressure transients that cause shrinking or swelling of the steam within the reactor, or due to excessive removal of coolant from the vessel as a result of a feedwater trip. For Dresden and Quad Cities, there is currently a 22-inch difference in elevation between the normal water level and the scram setpoint. Process control systems are designed so that the reactor can automatically recover from many transients such as a feedwater pump trip, which might cause a significant change in the reactor water level. However, in some cases, with this narrow water level range, reactor scrams may result that would have been avoidable if plant control systems or operators had slightly more time to take control. In addition to scrambling the reactor, a drop in reactor water level initiates primary containment isolation, isolation of reactor water clean up system (RWCU), and residual heat removal shutdown suction valves closure.

A low reactor pressure vessel (RPV) water level indicates that the capability to cool the fuel may be threatened. Should the RPV water level decrease too far, fuel damage could result. The current Reactor Vessel Water Level-Low allowable value is ≥ 10.24 inches for Dresden ≥ 11.8 inches for Quad Cities. The affected functions for Dresden and Quad Cities are listed below:

1. Reactor Protection Instrumentation, TS Table 3.3.1.1-1 Function 4 - A reactor scram is initiated at a low water level to substantially reduce the heat generated in the fuel from fission.
2. Primary Containment Isolation Instrumentation, TS Table 3.3.6.1. Function 2.a - The isolation of the primary containment on low RPV level supports actions to ensure that the offsite dose limits of 10 CFR Part 100 are not exceeded.
3. Primary Containment Isolation Instrumentation, TS Table 3.3.6.1. Function 5.b - The isolation of the RWCU system on low RPV water level supports actions to ensure that the fuel peak cladding temperature remains below the limits of 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water nuclear power reactors."
4. Primary Containment Isolation Instrumentation, TS Table 3.3.6.1. Function 6.b - The isolation of shutdown cooling system is not directly assumed in the updated safety analysis report safety analyses because a break of the shutdown cooling system is bounded by breaks of the recirculation and main steam lines.
5. Secondary Containment Isolation Instrumentation, TS Table 3.3.6.2-1, Function 1 - An isolation of the secondary containment and actuation of the standby gas treatment system are initiated in order to minimize the potential of an offsite release.

6. Control Room Emergency Ventilation (CREV) System Isolation Instrumentation, TS Table 3.3.7.1-1, Function 1 (Quad Cities only) - An isolation of the CREV system occurs since this could be a precursor to a potential radiation release and subsequent radiation exposure to control room personnel.

3.0 EVALUATION

The proposed change will give both facilities an additional 8 inches of operating range between the normal reactor vessel water level and the vessel water level used as the setpoint for initiation of the above functions.

The licensee's proposed changes and the staff's evaluation of the changes are discussed below:

1. Proposed Change: TS Table 3.3.1.1-1 Function 4 — The allowable value for this function (reactor scram on reactor vessel low water level) will change from greater than or equal to 10.24 inches to greater than or equal to 2.65 inches for Dresden, and from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for Quad Cities.

Evaluation: The primary purpose of the proposed change is to increase the operating range between the normal reactor vessel water level and the trip setpoint. The increased range will provide additional time for operators or automatic features to respond to recoverable transients and thus avoid unnecessary reactor scrams. The allowable value for normal plant operation is based on the requirement that the seal skirts of the separators and dryers remain covered with water. This requirement does not affect the plant's licensing and safety basis.

The allowable value is based on the requirement that there be sufficient quantity of coolant following a reactor scram on reactor vessel low water level signal during transients involving loss of normal feedwater flow. Thus, the only transient affected by the lowering of the reactor vessel low water level scram setpoint is the loss of feedwater flow (LOFW) transient. During the LOFW event, the reactor water level continues to drop following the scram until the low-low water level is reached. At that time, for Dresden, high-pressure coolant injection (HPCI) is started to maintain the reactor water level and the isolation condenser system provides the core cooling. In the case of Quad Cities, both HPCI and the reactor core isolation cooling system initiate to maintain the reactor water level.

The low-low setpoint also actuates the closure of the main steam isolation valves (MSIVs) to limit the amount of water inventory leaving the vessel. Lowering the water level setpoint by 8 inches will delay the scram by a few seconds, but since the setpoint for the initiation of HPCI and MSIVs at the low-low water level signal is not changed, the change will not affect the ability of these systems to maintain vessel inventory and therefore will not affect the thermal margin. The postulated loss-of-coolant accident (LOCA) inside the containment is the most limiting accident for the peak cladding temperature (PCT) because the break outside the containment is isolated before the loss of vessel inventory can uncover the core in the vessel.

For both large- and small-break LOCAs, the licensee has determined that the reactor time of scram will not change because the high drywell pressure will precede the low reactor water level scram signal. Lowering of the water level signal setpoint will not significantly affect an emergency core cooling system initiation time or the PCT.

The licensee has used the staff-approved setpoint calculation methodology to determine the allowable value of the reactor vessel low water level setpoint. On this basis, the staff finds the proposed change acceptable.

2. Proposed Change: TS Table 3.3.6.1-1 Function 2.a — The allowable value for this function (primary containment isolation on reactor vessel low water level) will be changed from greater than or equal to 10.24 inches to greater than or equal to 2.65 inches for Dresden and from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for Quad Cities.

Evaluation: The purpose of containment isolation on reactor vessel low water level signal is to minimize potential inventory loss across the containment boundary and to prevent offsite radiation doses from exceeding 10 CFR Part 100 limits during a postulated LOCA. For LOCAs inside containment, the high-drywell pressure signal will isolate the primary containment before the reactor vessel low water level signal and since the radiological fission product are assumed to be immediately released following a LOCA, the reactor vessel low water level signal does not affect the amount of fission product released or the time at which containment isolation occurs.

For LOCAs outside containment, the main steamline break is the limiting event. This event is mitigated by the containment isolation, which occurs on high steam flow or low steam line pressure. Therefore, this change does not affect the mitigation of this event. For small steamline breaks outside containment, the containment isolation occurs on the reactor vessel low water level signal. However, lowering reactor vessel low water level setpoint will not cause a larger mass release from the small steamline break than from the large steamline break. Therefore, a few seconds delay will not affect the ability of the containment isolation valves to perform their intended function. On this basis, the staff finds the proposed change acceptable.

3. Proposed Change: TS Table 3.3.6.1-1 Function 5.b — The allowable value for this function (reactor water cleanup system isolation on reactor vessel low water level signal) will be changed from greater than or equal to 10.24 inches to greater than or equal to 2.65 inches for Dresden and from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for Quad Cities.

Evaluation: The purpose of this function is to isolate the RWCU on the reactor vessel low water level signal. However, no credit is taken for RWCU system isolation in the accident analysis, since breaks in this system are bounded by the breaks in larger systems. Therefore, the delay of a few seconds in isolating valves on the reactor vessel low water level signal will not affect the valves' ability to perform their intended function. On this basis, the staff finds the proposed change acceptable.

4. Proposed Change: TS Table 3.3.6.1-1 Function 6.b — The allowable value for this function (shutdown cooling system isolation on reactor vessel low water level signal) will

be changed from greater than or equal to 10.24 inches to greater than or equal to 2.65 inches for Dresden and from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for the Quad Cities.

Evaluation: This function is required to be operable only during modes 3, 4, and 5. This function is not directly assumed in the safety analysis because a break in the shutdown cooling system is bounded by the break in the recirculation and main steam lines. Therefore, the delay of a few seconds in isolating the shutdown cooling system for the reactor vessel low water level signal will not affect the ability of the isolation valves to perform their intended function. On this basis, the staff finds the proposed change acceptable.

5. Proposed Change: TS Table 3.3.6.2-1, Function 1 - The allowable value for this function (secondary containment isolation instrumentation on reactor vessel low water level signal) will be changed from greater than or equal to 10.24 inches to greater than or equal to 2.65 inches for Dresden and from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for the Quad Cities.

Evaluation: The associated secondary containment isolation is initiated in order to minimize the potential of an offsite release. The licensee has elected to use the same allowable value as the allowable value for the reactor protection system and therefore has not analyzed separately.

Since the proposed change in the reactor scram setpoint does not result in a change to the current safety analyses, the change in the allowable value for the secondary containment isolation function continues to ensure that any offsite releases are within the limits calculated in the safety analyses. On this basis, the staff finds the proposed change acceptable.

6. Proposed Change: TS Table 3.3.7.1-1, Function 1 (Quad Cities only) - The allowable value for this function (CREV System isolation instrumentation on reactor vessel low water level signal) will be changed from greater than or equal to 11.8 inches to greater than or equal to 3.8 inches for the Quad Cities.

Evaluation: The CREV system isolation is initiated in order to minimize the potential dose to the control room operators. The allowable value for the CREV system isolation function is selected to be the same as the allowable value for the reactor protection system setpoint and is not analyzed separately. The proposed change in the reactor scram setpoint does not result in a change to the current safety analyses. Therefore, for the CREV system isolation function, the change in allowable value continues to ensure that the radiation exposure of control room personnel, as result of a LOCA, does not exceed the limits set by general design criteria (GDC) 19 "Control Room," of 10 CFR Part 50, Appendix A. On this basis, the staff finds the proposed change acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Illinois State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (66 FR 30490). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: H. Garg, G. Thomas

Date: December 18, 2001