

November 12, 1993

Docket Nos. 50-250  
and 50-251

DISTRIBUTION  
See attached sheet

Mr. J. H. Goldberg  
President-Nuclear Division  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

Dear Mr. Goldberg:

SUBJECT: TURKEY POINT UNITS 3 AND 4 - ISSUANCE OF AMENDMENTS RE: RELAXED AXIAL OFFSET CONTROL (RAOC) AND CORE OPERATING LIMITS REPORT (TAC NOS. M86297 AND M86298)

The Commission has issued the enclosed Amendment No. 156 to Facility Operating License No. DPR-31 and Amendment No. 150 to Facility Operating License No. DPR-41 for the Turkey Point Plant, Units Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application dated April 23, 1993 to implement a Relaxed Axial Offset Control (RAOC) methodology for axial flux difference control and to relocate cycle-specific parameter limits from the Technical Specifications to a Core Operating Limits Report.

A copy of the Safety Evaluation is also enclosed. The Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,



L. Raghavan, Project Manager  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 156 to DPR-31
2. Amendment No. 150 to DPR-41
3. Safety Evaluation

cc w/enclosures:  
See next page

OFFICE	LA:PDII-2	PM:PDII-2	D:PDII-2	OGC/LLY	<del>SRXB</del>
NAME	ETana ETT	LRaghavan LL	HBERKOW	M/Pring	
DATE	09/29/93	10/15/93	10/18/93	11/24	11/1

160018

9311170388 931112  
PDR ADDCK 05000250  
P PDR

**NRC FILE CENTER COPY**

DF01/11

Mr. J. H. Goldberg  
Florida Power and Light Company

Turkey Point Plant

cc:

Harold F. Reis, Esquire  
Newman and Holtzinger, P.C.  
1615 L Street, N.W.  
Washington, DC 20036

Regional Administrator, RII  
U. S. Nuclear Regulatory Commission  
101 Marietta Street, N.W. Suite 2900  
Atlanta, Georgia 30323

Jack Shreve, Public Counsel  
Office of the Public Counsel  
c/o The Florida Legislature  
111 West Madison Avenue, Room 812  
Tallahassee, Florida 32399-1400

Attorney General  
Department of Legal Affairs  
The Capitol  
Tallahassee, Florida 32304

John T. Butler, Esquire  
Steel, Hector and Davis  
4000 Southeast Financial Center  
Miami, Florida 33131-2398

Plant Manager  
Turkey Point Nuclear Plant  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

Mr. Thomas F. Plunkett, Site  
Vice President  
Turkey Point Nuclear Plant  
Florida Power and Light Company  
P.O. Box 029100  
Miami, Florida 33102

Mr. H. N. Paduano  
Acting Director, Nuclear Licensing  
Florida Power and Light Company  
P.O. Box 14000  
Juno Beach, Florida 33408-0420

Joaquin Avino  
County Manager of Metropolitan  
Dade County  
111 NW 1st Street, 29th Floor  
Miami, Florida 33128

Senior Resident Inspector  
Turkey Point Nuclear Generating  
Station  
U.S. Nuclear Regulatory Commission  
P.O. Box 1448  
Homestead, Florida 33090

Mr. Bill Passeti  
Office of Radiation Control  
Department of Health and  
Rehabilitative Services  
1317 Winewood Blvd.  
Tallahassee, Florida 32399-0700

Mr. Joe Myers, Director  
Division of Emergency Preparedness  
Department of Community Affairs  
2740 Centerview Drive  
Tallahassee, Florida 32399-2100



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 156  
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated April 23, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and;
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

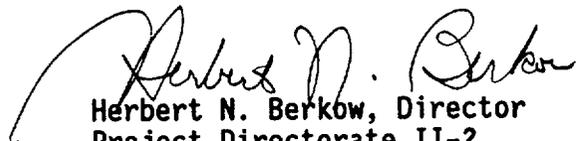
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 156, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 12, 1993



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150  
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated April 23, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

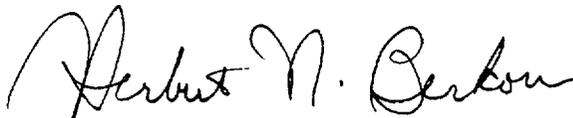
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 150, are hereby incorporated in the license. The Environmental Protection Plan contained in Appendix B is hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director  
Project Directorate II-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: November 12, 1993

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 156 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 150 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove pages

ii  
v  
xxiii  
-  
3/4 2-1  
3/4 2-2  
3/4 2-3  
B 3/4 2-2  
B 3/4 2-3  
6-20  
-

Insert pages

ii  
v  
xxiii  
1-6a  
3/4 2-1  
3/4 2-2  
3/4 2-3  
B 3/4 2-2  
B 3/4 2-3  
6-20  
6-20a

INDEX

DEFINITIONS

---

<u>SECTION</u>	<u>PAGE</u>
1.28 SOLIDIFICATION.....	1-5
1.29 SOURCE CHECK.....	1-5
1.30 STAGGERED TEST BASIS.....	1-5
1.31 THERMAL POWER.....	1-6
1.32 TRIP ACTUATING DEVICE OPERATIONAL TEST.....	1-6
1.33 UNIDENTIFIED LEAKAGE.....	1-6
1.34 UNRESTRICTED AREA.....	1-6
1.35 VENTILATION EXHAUST TREATMENT SYSTEM.....	1-6
1.36 VENTING.....	1-6
1.37 DIGITAL CHANNEL OPERATIONAL TEST.....	1-6
1.38 CORE OPERATING LIMITS REPORT.....	1-6a
TABLE 1.1 FREQUENCY NOTATION.....	1-7
TABLE 1.2 OPERATIONAL MODES.....	1-8

INDEX

LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS

---

<u>SECTION</u>	<u>PAGE</u>
<u>3/4.2 POWER DISTRIBUTION LIMITS</u>	
3/4.2.1 AXIAL FLUX DIFFERENCE.....	3/4 2-1
3/4.2.2 HEAT FLUX HOT CHANNEL FACTOR.....	3/4 2-4
FIGURE 3.2-2 $K(Z)$ - NORMALIZED $F_Q(Z)$ AS A FUNCTION OF CORE HEIGHT..	3/4 2-5
3/4.2.3 NUCLEAR ENTHALPY RISE HOT CHANNEL FACTOR.....	3/4 2-11
3/4.2.4 QUADRANT POWER TILT RATIO.....	3/4 2-13
3/4.2.5 DNB PARAMETERS.....	3/4 2-16
 <u>3/4.3 INSTRUMENTATION</u>	
3/4.3.1 REACTOR TRIP SYSTEM INSTRUMENTATION.....	3/4 3-1
TABLE 3.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION.....	3/4 3-2
TABLE 4.3-1 REACTOR TRIP SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-8
3/4.3.2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-13
TABLE 3.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION.....	3/4 3-14
TABLE 3.3-3 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION TRIP SETPOINTS.....	3/4 3-23
TABLE 4.3-2 ENGINEERED SAFETY FEATURES ACTUATION SYSTEM INSTRUMENTATION SURVEILLANCE REQUIREMENTS.....	3/4 3-29
3/4.3.3 MONITORING INSTRUMENTATION	
Radiation Monitoring For Plant Operations.....	3/4 3-35

INDEX

ADMINISTRATIVE CONTROLS

---

<u>SECTION</u>	<u>PAGE</u>
6.5.2 COMPANY NUCLEAR REVIEW BOARD	
Function.....	6-8
Composition.....	6-8
Alternates.....	6-8
Consultants.....	6-9
Meeting Frequency.....	6-9
Quorum.....	6-9
Review.....	6-9
Audits.....	6-10
Records.....	6-11
6.5.3 TECHNICAL REVIEW AND CONTROL	
Activities.....	6-11
<u>6.6 REPORTABLE EVENT ACTION.....</u>	6-12
<u>6.7 SAFETY LIMIT VIOLATION.....</u>	6-12
<u>6.8 PROCEDURES AND PROGRAMS.....</u>	6-13
<u>6.9 REPORTING REQUIREMENTS.....</u>	6-15
6.9.1 ROUTINE REPORTS.....	6-15
Startup Report.....	6-15
Annual Reports.....	6-16
Annual Radiological Environmental Operating Report.....	6-17
Semiannual Radioactive Effluent Release Report.....	6-18
Monthly Operating Report.....	6-19
Peaking Factor Limit Report.....	6-19
Core Operating Limits Report.....	6-20
6.9.2 SPECIAL REPORTS.....	6-20
<u>6.10 RECORD RETENTION.....</u>	6-20

## DEFINITIONS

---

### CORE OPERATING LIMITS REPORT

1.38 The CORE OPERATING LIMITS REPORT (COLR) is the unit-specific document that provides core operating limits for the current operating reload cycle. These cycle-specific core operating limits shall be determined for each reload cycle in accordance with NRC approved methodology. Unit operation within these operating limits is addressed in individual specifications. The COLR is submitted to the NRC in accordance with the requirements of 6.9.1.7.

### 3/4.2 POWER DISTRIBUTION LIMITS

#### 3/4.2.1 AXIAL FLUX DIFFERENCE

##### LIMITING CONDITION FOR OPERATION

---

3.2.1 The indicated AXIAL FLUX DIFFERENCE (AFD) shall be maintained within:

- a. the allowed Relaxed Axial Offset Control (RAOC) operational space as defined in the CORE OPERATING LIMITS REPORT (COLR), or
- b. within a +/- 2% or +/- 3% target band about the target flux difference during Base Load operation.

APPLICABILITY: MODE 1, above 50% of RATED THERMAL POWER\*.

##### ACTION\*\*\*:

- a. For RAOC operation with the indicated AFD outside of the limits specified in the COLR, either
  1. Restore the indicated AFD to within the RAOC limits within 15 minutes, or
  2. Reduce THERMAL POWER to less than 50% of RATED THERMAL POWER within 30 minutes and reduce the Power Range Neutron Flux - High Trip setpoint to less than or equal to 55% of RATED THERMAL POWER within the next 4 hours.
- b. For Base Load operation above  $P_T^{**}$  with the indicated AFD outside of the applicable target band about the target flux difference, either
  1. Restore the indicated AFD to within the Peaking Factor Limit Report target band limits within 15 minutes, or
  2. Reduce THERMAL POWER to less than  $P_T$  and discontinue Base Load operation within 30 minutes.
- c. THERMAL POWER shall not be increased above 50% of RATED THERMAL POWER unless indicated AFD is within the limits specified in the COLR.

---

\* See Special Test Exceptions Specification 3.10.2.

\*\*  $P_T$  = Reactor Power at which predicted  $F_0$  would exceed its limit (consistent with Specification 4.2.2.1).

\*\*\* The indicated AFD shall be considered outside of its target band when two or more OPERABLE excore channels are indicating the AFD to be outside the target band.

## POWER DISTRIBUTION LIMITS

### SURVEILLANCE REQUIREMENTS

---

4.2.1.1 The indicated AFD shall be determined to be within its limits during POWER OPERATION above 50% of RATED THERMAL POWER by:

- a. Monitoring the indicated AFD for each OPERABLE excore channel:
  - 1) At least once per 7 days when the alarm used to monitor the AFD is OPERABLE, and
  - 2) At least once per hour for the first 6 hours after restoring the alarm used to monitor the AFD to OPERABLE status.\*
- b. Monitoring and logging the indicated AFD for each OPERABLE excore channel at least once per hour for the first 24 hours and at least once per 30 minutes thereafter, when the alarm used to monitor the AFD is inoperable. The logged values of the indicated AFD shall be assumed to exist during the interval preceding each logging.

4.2.1.2 The target flux difference of each OPERABLE excore channel shall be determined by measurement at least once per 92 Effective Full Power Days. The provisions of Specification 4.0.4 are not applicable.

4.2.1.3 The target flux difference shall be updated at least once per 31 Effective Full Power Days by either determining the target flux difference pursuant to Specification 4.2.1.2 above or by linear interpolation between the most recently measured value and the predicted value at the end of the cycle life. The provisions of Specification 4.0.4 are not applicable.

---

\* Performance of a functional test to demonstrate OPERABILITY of the alarm used to monitor the AFD may be substituted for this requirement.

(Deleted)

FIGURE 3.2-1  
AXIAL FLUX DIFFERENCE LIMITS AS A FUNCTION OF  
RATED THERMAL POWER

## POWER DISTRIBUTION LIMITS

### BASES

---

#### AXIAL FLUX DIFFERENCE (Continued)

At power level below  $P_T$ , the limits on AFD are specified in the CORE OPERATING LIMITS REPORT (COLR) for RAOC operation. These limits were calculated in a manner such that expected operational transients, e.g., load follow operations, would not result in the AFD deviating outside of those limits. However, in the event that such a deviation occurs, a 15 minute period of time allowed outside of the AFD limits at reduced power levels will not result in significant xenon redistribution such that the envelope of peaking factors would change sufficiently to prevent operation in the vicinity of the power level.

With  $P_T$  greater than 100%, two modes are permissible: 1) RAOC with fixed AFD limits as a function of reactor power level and 2) Base Load operation which is defined as the maintenance of the AFD within a band about a target value. Both the fixed AFD limits for RAOC operation and the target band for Base Load operation are defined in the COLR and the Peaking Factor Limit Report, respectively. However, it is possible during extended load following maneuvers that the AFD limits may result in restrictions in the maximum allowed power or AFD in order to guarantee operation with  $F_0(Z)$  less than its limiting value. Therefore,  $P_T$  is calculated to be less than 100%. To allow operation at the maximum permissible value above  $P_T$ , Base Load operation restricts the indicated AFD to a relative small target band and power swings. For Base Load operation, it is expected that the plant will operate within the target band. Operation outside of the target band for the short time period allowed (15 minutes) will not result in significant xenon redistribution such that the envelope of peaking factors will change sufficiently to prohibit continued operation in the power region defined above. To assure that there is no residual xenon redistribution impact from past operation on the Base Load operation, a 24-hour waiting period within a defined range of  $P_T$  and AFD allowed by RAOC is necessary. During this period, load changes and rod motion are restricted to that allowed by the Base Load requirement. After the waiting period, extended Base Load operation is permissible.

Provisions for monitoring the AFD on an automatic basis are derived from the plant process computer through the AFD Monitoring Alarm. The computer monitors the OPERABLE excore detector outputs and provides an alarm message immediately if the AFD for two or more OPERABLE excore channels are: 1) outside the acceptable AFD (for RAOC operation), or 2) outside the acceptable AFD target band (for Base Load operation). These alarms are active when power is greater than: 1) 50% of RATED THERMAL POWER (for RAOC operation), or 2)  $P_T$  (Base Load operation). Penalty deviation minutes for Base Load operation are not accumulated based on the short time period during which operation outside of the target band is allowed.

POWER DISTRIBUTION LIMITS

BASES

---

(Deleted)

FIGURE B 3/4 2-1

TYPICAL INDICATED AXIAL FLUX DIFFERENCE VERSUS THERMAL POWER

PEAKING FACTOR LIMIT REPORT (Continued)

Factor Limit Report, the Peaking Factor Limit Report shall be provided to the NRC Document Control desk with copies to the Regional Administrator and the Resident Inspector within 30 days of their implementation, unless otherwise approved by the Commission.

The analytical methods used to generate the Peaking Factor limits shall be those previously reviewed and approved by the NRC. If changes to these methods are deemed necessary they will be evaluated in accordance with 10 CFR 50.59 and submitted to the NRC for review and approval prior to their use if the change is determined to involve an unreviewed safety question or if such a change would require amendment of previously submitted documentation.

CORE OPERATING LIMITS REPORT

6.9.1.7 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT (COLR) before each reload cycle or any remaining part of a reload cycle for the Axial Flux Difference (AFD) Limits. The analytical methods used to determine the AFD limits shall be those previously reviewed and approved by the NRC in:

1. WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL  $F_0$  SURVEILLANCE TECHNICAL SPECIFICATION," June 1983.
2. WCAP-8385, "POWER DISTRIBUTION CONTROL AND LOAD FOLLOWING PROCEDURES - TOPICAL REPORT," September 1974.

The AFD limits shall be determined such that all applicable limits of the safety analyses are met. The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided to the NRC Document Control Desk with copies to the Regional Administrator and the Resident Inspector, within 30 days after their implementation unless otherwise approved by the Commission.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the Regional Administrator of the Regional Office of the NRC within the time period specified for each report as stated in the Specifications within Sections 3.0, 4.0, or 5.0.

6.10 RECORD RETENTION

6.10.1 In addition to the applicable record retention requirements of Title 10, Code of Federal Regulations, the following records shall be retained for at least the minimum period indicated.

6.10.2 The following records shall be retained for at least 5 years:

- a. Records and logs of unit operation covering time interval at each power level;
- b. Records and logs of principal maintenance activities, inspections, repair, and replacement of principal items of equipment related to nuclear safety;

RECORD RETENTION (Continued)

- c. ALL REPORTABLE EVENTS;
- d. Records of surveillance activities, inspections, and calibrations required by these Technical Specifications;
- e. Records of changes made to the procedures required by Specification 6.8.1;
- f. Records of radioactive shipments;
- g. Records of sealed source and fission detector leak tests and results; and
- h. Records of annual physical inventory of all sealed source material of record.

6.10.3 The following records shall be retained for the duration of the unit Operating License:



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NO. DPR-31  
AND AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

1.0 INTRODUCTION

By letter dated April 23, 1993, Florida Power and Light Company (FPL or the licensee) proposed changes to the Technical Specifications (TS) for the Turkey Point Plant, Units 3 and 4, to implement a Relaxed Axial Offset Control (RAOC) methodology instead of the present Constant Axial Offset Control (CAOC) methodology for the Axial Flux Difference (AFD) control. The proposed RAOC methodology would be in accordance with the Nuclear Regulatory Commission (NRC) approved methodology of WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control  $F_0$  Surveillance Technical Specification." TS 3/4.2.1, Axial Flux Difference, associated surveillance requirements and TS bases would be revised to reflect the proposed AFD control.

In addition, the licensee proposed TS changes to relocate cycle-specific parameter limits to a Core Operating Limits Report (COLR) in accordance with NRC's Generic Letter (GL) 88-16, "Removal of Cycle-specific Parameter Limits from Technical Specifications" dated October 4, 1988. TS 6.0 "Administrative Controls" would be revised to establish and implement COLR. Consistent with the proposed changes, TS would also be revised editorially.

2.0 EVALUATION

The proposed revisions to the TS include (1) relocating the AFD cycle-specific core operating limits from the TS to the COLR, (2) changing the AFD control methodology in TS 3/4.2.1 from CAOC to RAOC, and (3) incorporating editorial changes.

2.1 Core Operating Limits Report

In accordance with the guidance provided by GL 88-16, the licensee proposed TS changes for relocating the AFD cycle-specific core operating limits from the TS to the COLR.

1. TS 1.38 "Core Operating Limits Report" is added to define COLR as a document that provides unit-specific parameter limits for the current operating reload. The licensee would determine cycle/reload-specific

parameter limits on a unit-specific basis in accordance with NRC-approved methodologies. Plant operation within these limits is addressed by individual specifications.

2. TS Figure 3.2-1, which specifies AFD limits as a function of rated thermal power, would be deleted and relocated to the COLR. Also, TS bases of affected specifications would be modified to include appropriate reference to the COLR.
3. TS 6.9.1.7 is added to include the COLR under the reporting requirements of the Administrative Control section of the TS. The report would provide the values of cycle-specific parameter limits applicable for the current fuel cycle. Furthermore, this specification requires determination of AFD limits using the following NRC approved methodologies:
  - (a) WCAP-10216-P-A, "Relaxation of Constant Axial Offset Control  $F_0$  Surveillance Technical Specification," June 1983; and
  - (b) WCAP-8385, "Power Distribution Control and Load Following Procedures - Topical Report," September 1974.

The AFD limits would be determined such that all applicable limits of the safety analysis are met. In addition, TS 6.9.1.7 requires that all changes in cycle-specific parameter limits be documented in the COLR before each reload cycle, or remaining part of a reload cycle, and submitted upon issuance to the NRC prior to operation with the new parameter limits.

On the basis of its review, the NRC staff concludes that the licensee's proposed TS changes to relocate cycle-specific parameter limits in the COLR are consistent with GL 88-16. Because plant operation continues to be limited in accordance with the values of cycle-specific parameter limits that are established using NRC-approved methodologies, the staff finds that the proposed changes are consistent with 10 CFR 50.36 and are acceptable.

## 2.2 Axial Flux Difference Control Methodology

The licensee's proposed TS changes to implement the RAOC methodology instead of the present CAOC methodology for the AFD control are addressed below.

### (1) TS 3.2.1 "Axial Flux Difference"

The Limiting Condition for Operation (LCO) is revised to conform to (a) the RAOC methodology and (b) TS 4.2.2.3 for Base Load operation. This revision also includes increasing the power level from 15% to 50% in the applicability section of the LCO. RAOC is the methodology developed and documented in WCAP-10216-P-A for Westinghouse plants which allows AFD operation over a fixed wide range in AFD between 100% and 50% power with no limits below 50%. If the AFD limits are exceeded, the condition is alarmed, and the AFD must be returned to the allowable range within 15

minutes or power must be reduced to below 50%. The staff has reviewed and found this proposed change to be acceptable since it is performed in accordance with the NRC approved methodology of WCAP-10216-P-A.

(2) TS 4.2.1.1 Surveillance Requirements

This Surveillance Requirement is revised to allow determination of the AFD above 50% power level instead of the present 15% power level. This is acceptable since it is consistent with the NRC approved RAOC methodology.

2.3 Editorial Changes

The staff also reviewed deletion of Surveillance Requirements TS 4.2.1.2 for implementing the RAOC methodology, consequent renumbering of TS 4.2.1.3 and 4.2.1.4, and other minor editorial changes to the Index and Definition sections of the TS. The changes are administrative and, therefore, are acceptable.

Based on its review, the staff finds that relocation of the cycle-specific parameters to the COLR is in accordance with the requirements of GL 88-16, and the change from CAOC to RAOC method is in accordance with the NRC approved methodology. Therefore the proposed TS changes are acceptable.

3.0 STATE CONSULTATION

Based upon the written notice of the proposed amendments, the Florida State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment involves a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The amendment changes the surveillance requirements, record keeping and reporting requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (58 FR 30195). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and (c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

Based on the staff evaluation in Section 2.0 above, the staff concludes that the proposed Technical Specifications changes are acceptable.

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principle Contributor: T. Huang, SRXB, DE

DATE: November 12, 1993

OFFICIAL RECORD COPY - DOCUMENT NAME: C:\AUTOS\WPDOCS\TURKEY\TP86297.AMD

DATED: November 12, 1993

AMENDMENT NO. 156 TO FACILITY OPERATING LICENSE NO. DPR-31-TURKEY POINT UNIT 3  
AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NO. DPR-41-TURKEY POINT UNIT 4

Distribution

Docket File

NRC & Local PDRs

PDII-2 Reading

S. Varga, 14/E/4

H. Berkow

E. Tana

L. Raghavan

OGC

D. Hagan, 3302 MNBB

G. Hill (4)

C. Grimes, 11/F/23

ACRS (10)

OPA

OC/LFDCB

M. Sinkule, R-II

R. Jones

T. Huang