

December 13, 1984

Correction to Amdt 108
to DPR-31

Docket Nos. 50-250
and 50-251

Mr. J. W. Williams, Jr., Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

Dear Mr. Williams:

During the period of August 27, 1984 through November 21, 1984, the Commission has issued six amendments to Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Units 3 and 4. Your staff has brought to our attention administrative errors on some of the Technical Specification pages. These typographical errors, inadvertent omissions or other errors were the result of issuing the amendments in a different sequence than they were requested. It is our understanding that a system has been initiated by your staff to track all active amendment requests to assure that administrative errors will be identified prior to issuance of amendments regardless of the request or issuance sequence. We will work closely with your staff to eliminate administrative errors prior to the issuance of future amendments.

Administrative errors are identified in the Technical Specification Table of Content pages i and iv, Technical Specification pages 1-9 and 3.1-4, Table 4.1-1 Sheets 3 and 4 and the Bases page B2.3-3. Amendments 110/104 left out the identification of Section 1.35 on page i and Amendments 111/105 left out the identification of B4.18 and B4.19 on page iv. Amendments 108/102 resulted in an incorrect sequence of numbers and Item 1.35 was omitted from page 1-9. Amendments 110/104 had a typographical error which resulted in the last sentence in paragraph f being partially omitted. Amendments 110/104 omitted Item 31.b and Item 33 on Table 4.1-1 Sheet 3 and an incorrect numbering sequence on Sheet 4. Amendments 106/100 resulted in incorrect wording of the second paragraph on page B2.3-3.

Enclosed are the corrected pages i, iv, 1-9, 3.1-4, Table 4.1-1 Sheets 3 and 4, and B2.3-3. We have verified that the errors do not affect the amendments, the supporting Safety Evaluations and the corrections are consistent with their applicable amendments. Please use the enclosed instruction sheet and replace the pages identified in Appendix A of Facility Operating License Nos. DPR-31 and DPR-41 with the corrected pages and Tables.

Distribution

Docket file

NRC PDR	LPDR
ORB#1 RDG	
Gray file	(4)
DEisenhut	WJones
CParrish	DBrinkman
DMcDonald	RDiggs
OELD	OPA, CMiles
EJordan	
PMcKee	
JPartlow	
ACRS (10)	
LHarmon	
TBarnhart	(8)

8412260299 841213
PDR ADDCK 05000250
P PDR

Mr. Williams

-2-

December 13, 1984

Please accept our apologies for any inconvenience these errors may have caused. As stated above, we will work closely with your staff to eliminate administrative errors in issuance of future amendments.

Sincerely,

/s/DGMcDonald

Daniel G. McDonald, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:
As stated

cc w/enclosures:
See next page

ORB#1:DL
CParrish
12/12/84

ORB#1:DL
DMcDonald/ts
12/12/84

C-ORB#1:DL
Svarga
12/12/84

J. W. Williams, Jr.
Florida Power and Light Company

Turkey Point Plants
Units 3 and 4

cc: Harold F. Reis, Esquire
Newman and Holtziner P.C.
1615 L Street, N.W.
Washington, DC 10036

Administrator
Department of Environmental
Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 33130

James P. O'Reilly
Regional Administrator, Region II
U.S Nuclear Regulatory Commission
Suite 2900
101 Marietta Street
Atlanta, GA 30303

Norman A. Coll, Esquire
Steel, Hector and Davis
4000 Southeast Financial
Center
Miami, Florida 33131-2398

Martin H. Hodder, Esquire
1131 N.E. 86th Street
Miami, Florida 33138

Mr. Ken N. Harris, Vice President
Turkey Point Nuclear Plant
Florida Power and Light Company
P.O. Box 013100
Miami, Florida 33101

Joette Lorion
7269 SW 54 Avenue
Miami, Florida 33143

Mr. M. R. Stierheim
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Mr. Chris J. Baker, Plant Manager
Turkey Point Nuclear Plant
Florida Power and Light Company
P.O. Box 013100
Miami, Florida 33101

Resident Inspector
Turkey Point Nuclear Generating Station
U.S. Nuclear Regulatory Commission
Post Office Box 57-1185
Miami, Florida 33257-1185

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.W.
Atlanta, GA 30308

Mr. Allan Schubert, Manager
Public Health Physicist
Department of Health and
Rehabilitative Services
1323 Winewood Blvd.
Tallahassee, Florida 32301

Mr. Jack Shreve
Office of the Public Counsel
Room 4, Holland Building
Tallahassee, Florida 32304

AMENDMENT CORRECTIONS
FACILITY LICENSE NOS. DPR-31 AND DPR-41
DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
i Amendment Nos. 110 and 104	i Amendment Nos. 110 and 104
iv Amendment Nos. 111 and 105	iv Amendment Nos. 111 and 105
1-9 Amendment Nos. 108 and 102	1-9 Amendment Nos. 108 and 102
3.1-4 Amendment Nos. 110 and 104	3.1-4 Amendment Nos. 110 and 104
Table 4.1-1 Sh 3 & Sh 4 Amendment Nos. 110 and 104	Table 4.1-1 Sh 3 & Sh 4 Amendment Nos. 110 and 104
B 2.3-3 Amendment Nos. 106 and 100	B 2.3-3 Amendment Nos. 106 and 100

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
TECHNICAL SPECIFICATIONS		
1.0	DEFINITIONS	1-1
1.1	Safety Limits	1-1
1.2	Limiting Safety System Settings	1-1
1.3	Limiting Conditions for Operation	1-1
1.4	Operable	1-1
1.5	Containment Integrity	1-2
1.6	Protective Instrumentation Logic	1-2
1.7	Instrumentation Surveillance	1-3
1.8	Shutdown	1-3
1.9	Power Operation	1-4
1.10	Refueling Operation	1-4
1.11	Rated Power	1-4
1.12	Thermal Power	1-4
1.13	Design Power	1-4
1.14	Dose Equivalent I-131	1-5
1.15	Power Tilt	1-5
1.16	Interim Limits	1-6
1.17	Low Power Physics Tests	1-6
1.18	Engineered Safety Features	1-6
1.19	Reactor Protection System	1-6
1.20	Safety Related Systems and Components	1-6
1.21	Per Annum	1-6
1.22	Reactor Coolant System Pressure Boundary Integrity	1-6
1.23	Coolant Loop	1-7
1.24	\bar{E} -Average Disintegration Energy	1-7
1.25	Gas Decay Tank System	1-8
1.26	Ventilation Exhaust Treatment System	1-8
1.27	Process Control Program (PCP)	1-8
1.28	Offsite Dose Calculation Manual (ODCM)	1-8
1.29	Dose Equivalent I-131	1-8
1.30	Purge-Purging	1-9
1.31	Venting	1-9
1.32	Site Boundary	1-9
1.33	Unrestricted Area	1-9
1.34	Member(s) of the Public	1-9
1.35	Heavy Loads	1-9
2.0	SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS	2.1-1
2.1	Safety Limit, Reactor Core	2.1-1
2.2	Safety Limit, Reactor Coolant System Pressure	2.2-1
2.3	Limiting Safety System Setting, Protective Instrumentation	2.3-1
3.0	LIMITING CONDITIONS FOR OPERATION	3.0-1
3.1	Reactor Coolant System	3.1-1
	Operational Components	3.1-1
	Pressure-Temperature Limits	3.1-2
	Leakage	3.1-3
	Maximum Reactor Coolant Activity	3.1-4
	Reactor Coolant Chemistry	3.1-6
	DNB Parameters	3.1-7

TABLE OF CONTENTS (Continued)

<u>Section</u>	<u>Title</u>	<u>Page</u>
B3.5	Bases for Limiting Conditions for Operation, Instrumentation	B3.5-1
B3.6	Bases for Limiting Conditions for Operation, Chemical and Volume Control System	B3.6-1
B3.7	Bases for Limiting Conditions for Operation, Electrical Systems	B3.7-1
B3.8	Bases for Limiting Conditions for Operation, Steam and Power Conversion Systems	B3.8-1
B3.9	Bases for Limiting Conditions for Operation, Radioactive Materials Release	B3.9-1
B3.10	Bases for Limiting Conditions for Operation, Refueling	B3.10-1
B3.11	Bases for Limiting Conditions for Operation, Miscellaneous Radioactive Material Sources	B3.11-1
B3.12	Bases for Limiting Conditions for Operation, Cask Handling	B3.12-1
B3.13	Bases for Limiting Conditions for Operation, Snubbers	B3.13-1
B3.14	Bases for Fire Protection System	B3.14-1
B3.15	Bases for Limiting Conditions of Operation, Overpressure Mitigating System	B3.15-1
B3.17	Bases for Limiting Conditions for Operation, Spent Fuel Storage	B3.17-1
B4.1	Bases for Operational Safety Review •	B4.1-1
B4.2	Bases for Reactor Coolant System In-Service Inspection	B4.2-1
B4.3	Bases for Reactor Coolant System Integrity	B4.3-1
B4.4	Bases for Containment Tests	B4.4-1
B4.5	Bases for Safety Injection Tests	B4.5-1
B4.6	Bases for Emergency Containment Cooling System Tests	B4.6-1
B4.7	Bases for Emergency Containment Filtering and Post Accident Containment Venting Systems Tests	B4.7-1
B4.8	Bases for Emergency Power System Periodic Tests	B4.8-1
B4.9	Bases for Main Steam Isolation Valve Tests	B4.9-1
B4.10	Bases for Auxiliary Feedwater System Tests	B4.10-1
B4.11	Bases for Reactivity Anomalies	B4.11-1
B4.12	Bases for Environmental Radiation Survey	B4.12-1
B4.13	Bases for Fire Protection Systems	B4.13-1
B4.14	Bases for Snubbers	B4.14-1
B4.15	Bases for Surveillance Requirements, Overpressure Mitigating System	B4.15-1
B4.18	Bases for System Flow Path Verifications	B4.18-1
B4.19	Bases for Reactor Coolant Vent System	B4.19-1

1.30 **PURGE - PURGING**

PURGE or PURGING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

1.31 **VENTING**

VENTING is the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is not provided or required during VENTING. Vent, used in system names, does not imply a VENTING process.

1.32 **SITE BOUNDARY**

The SITE BOUNDARY shall be that line beyond which the land is neither owned, leased nor otherwise controlled by the licensee.

1.33 **UNRESTRICTED AREA**

An UNRESTRICTED AREA shall be any area at or beyond the SITE BOUNDARY access to which is not controlled by the licensee for purposes of protection of individuals from exposure to radiation from radioactive materials, or any area within the SITE BOUNDARY used for residential quarters or for industrial, commercial, institutional and/or recreational purposes.

1.34 **MEMBER(S) OF THE PUBLIC**

MEMBER(S) OF THE PUBLIC shall include all persons who are not occupationally associated with the plant. This category does not include employees of the licensee, its contractors, vendors or members of the Armed Forces using property located within the SITE BOUNDARY. Also excluded from this category are persons who enter the site to service equipment or to make deliveries. This category does include persons who use portions of the site for recreational, occupational or other purposes not associated with the plant.

1.35 **HEAVY LOADS**

Any load in excess of the nominal weight of a fuel and control rod assembly and associated handling tool. For the purpose of this specification, HEAVY LOADS will be defined as loads in excess of 2000 pounds.

- e. After shutdown, corrective action shall be taken before operation is resumed.
- f. Above 2% of rated power, two leak detection systems of different principles shall be operable, one of which is sensitive to radioactivity. The latter may be out of service for 48 hours provided 1) Two other systems are operable and 2) containment purge valves are maintained closed.
- g. Reactor Coolant System leakage shall be limited to 1 gpm total primary-to-secondary leakage through all steam generators not isolated from the Reactor Coolant System and 500 gallons per day through any one steam generator not isolated from the Reactor Coolant System.

4. MAXIMUM REACTOR COOLANT ACTIVITY

The specific activity of the primary coolant shall be limited to:

- a. Less than or equal to 1.0 microcurie per gram DOSE EQUIVALENT I-131, and
- b. Less than or equal to $100/\bar{E}$ microcuries per gram.

With the above limits being exceeded, the following actions shall be taken:

- 1. When the reactor is critical or average reactor coolant temperature is greater than 500 F:
 - a. With the specific activity of the primary coolant greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 but within the allowable limit (below and to the left of the line) shown on Figure 3.1-1, operation may continue for up to 48 hours provided that the cumulative operating time under these circumstances does not exceed 800 hours in any consecutive 12 month period. With the total cumulative operating time at a primary coolant specific activity greater than 1.0 microcurie per gram DOSE EQUIVALENT I-131 exceeding 500 hours in

TABLE 4.1-1-SHEET 2

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
23. Logic Channels	N.A.	N.A.	M†	
24. Emer. Portable Survey Instruments	N.A.	A	M	
25. Seismograph	N.A.	N.A.	Q	Make trace. Test battery (change semi-annually)
26. Auxillary Feedwater Flow Rate	M†	R	N.A.	
27. RCS Subcooling Margin Monitor	M†	R	N.A.	
28. PORV Position Indicator (Primary Detector)	M†	N.A.	R	} Check consists of monitoring indicated position and verifying by observation of related parameters.
29. PORV Block Valve Position Indicator	M†	N.A.	R	
30. Safety Valve Position Indicator	M†	R	N.A.	
31. a. Loss of Voltage (both 4kv busses)	N.A.	N.A.	R	For AFW actuation at power only
b. Undervoltage (both 4KV busses and 480 volt Load Centers)**	St	R	M†	
32. Trip of both Main Feedwater Pump Breakers	N.A.	N.A.	R	For AFW actuation at power only
33. Turbine Trip (Auto-Stop Oil Pressure Switches)	N.A.	R.	N.A.	

** This item does not apply on Unit 3 until after implementation of PC/M 79-116 and on Unit 4 until after implementation of PC/M 80-44.

TABLE 4.1-1 SHEET 4

<u>Channel Description</u>	<u>Check</u>	<u>Calibrate</u>	<u>Test</u>	<u>Remarks</u>
34. Containment Water Level (Narrow Range)	M++	R	N.A.	
35. Containment Water Level (Wide Range)	M++	R	N.A.	
36. Containment High Range Area Radiation	S++	R(Note 1)	M++	
37. Containment Hydrogen Monitors	S+	Q(1)	M+	(1) Channel calibration using sample gas containing: a. One volume percent hydrogen, balance nitrogen. b. Four volume percent hydrogen, balance nitrogen.
38. High Range Noble Gas Effluent Monitors				
a. Plant Vent Exhaust	S	R	M	
b. Unit 3 Spent Fuel Pit Exhaust	S	R	M	
c. Condenser Air Ejectors	S+	R	M+	
d. Main Steam Lines	S+	R	M+	
39. Incore Thermocouples (Core Exit Thermocouples)	M++	R	N.A.(See Note 2)	

Reactor Trip Interlocks

Specified reactor trips are by passed at low power where they are not required for protection and would otherwise interfere with normal operation. The prescribed set points above which these trips are made functional assures their availability in the power range where needed.

An automatic reactor trip will occur if any pump is lost above 55% power which will prevent the minimum value of the DNBR from going below the applicable design limit during normal and anticipated transient operations when only two loops are in service,⁽⁹⁾ and the overtemperature ΔT trip setpoint is adjusted to the value specified for three loop operation.

A turbine trip initiates a reactor trip. On decreasing power, the turbine trip is automatically blocked by P-7; and on increasing power reinstated automatically by P-7.

Reset of reactor trip interlocks will be done under strict administrative control.

References

- (1) FSAR 14.1.1
- (2) FSAR 14.1.2
- (3) FSAR 14.1
- (4) FSAR 7.2, 7.3
- (5) FSAR 3.2.1
- (6) FSAR 14.3.1
- (7) FSAR 14 (page 14-3 and 14.1.9)
- (8) FSAR 14.1.11
- (9) FSAR 14.1.9
- (10) WCAP-8074