

January 25, 1985

*Correction to AmBk 104
15 DPR-31*

Docket Nos. 50-250
and 50-251
Mr. J. W. Williams, Jr., Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

<u>Distribution</u>	Docket file
NRC PDR	LPDR
ORB#1 RDG	Gray file (4)
DEisenhut	CParrish
DMcDonald	OELD
ACRS (10)	EJordan
PMcKee	JPartlow
RDiggs	LHarmon
TBarnhart (8)	WJones
DBrinkman	OPA, CMiles
RBallard	

Dear Mr. Williams:

Reference: TAC Numbers 10057 and 10058

SUBJECT: ADMINISTRATIVE ERROR - TECHNICAL SPECIFICATION TABLE 3.5-2

On August 14, 1984, the Commission issued Amendment Nos. 104 and 98 to Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Units 3 and 4, respectively. These amendments provided requirements in the Technical Specifications for protection of safety-related equipment subjected to sustained degraded voltage conditions at the offsite power source and interaction between the offsite and onsite source.

Your staff has brought to our attention an administrative error on Table 3.5-2 of the Technical Specifications which resulted in a footnote being left off which identifies an operating bypass condition for Item 1.5 of Table 3.5.2.

Enclosed is the corrected Table 3.5.2. We have varified that the error does not affect the amendments, the supporting Safety Evaluation and the corrections are consistent with the amendments. The footnote was initially added when Amendment Nos. 102 and 96 were issued on April 13, 1984. Please use the enclosed instruction sheet and replace the pages identified in Appendix A of Facility Operating License Nos. DPR-31 and DPR-41 with the corrected pages.

Please accept our apologies for any inconvenience these errors may have caused. As stated previously, we will work closely with your staff to eliminate adminstrative errors prior to the issuance of future amendments.

Sincerely,

/s/DGMcDonald

Daniel G. McDonald, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page
ORB#1:DL
CParrish
01/24/85

ORB#1:DL
DMcDonald/ts
01/25/85

BC-ORB#1:DL
SVarga
01/25/85

8502110125 850125
PDR ADOCK 05000250
PDR



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 25, 1985

Docket Nos. 50-250
and 50-251

Mr. J. W. Williams, Jr., Vice President
Nuclear Energy Department
Florida Power and Light Company
Post Office Box 14000
Juno Beach, Florida 33408

Dear Mr. Williams:

Reference: TAC Numbers 10057 and 10058

SUBJECT: ADMINISTRATIVE ERROR - TECHNICAL SPECIFICATION TABLE 3.5-2

On August 14, 1984, the Commission issued Amendment Nos. 104 and 98 to Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Units 3 and 4, respectively. These amendments provided requirements in the Technical Specifications for protection of safety-related equipment subjected to sustained degraded voltage conditions at the offsite power source and interaction between the offsite and onsite source.

Your staff has brought to our attention an administrative error on Table 3.5-2 of the Technical Specifications which resulted in a footnote being left off which identifies an operating bypass condition for Item 1.5 of Table 3.5.2.

Enclosed is the corrected Table 3.5.2. We have verified that the error does not affect the amendments, the supporting Safety Evaluation and the corrections are consistent with the amendments. The footnote was initially added when Amendment Nos. 102 and 96 were issued on April 13, 1984. Please use the enclosed instruction sheet and replace the pages identified in Appendix A of Facility Operating License Nos. DPR-31 and DPR-41 with the corrected pages.

Please accept our apologies for any inconvenience these errors may have caused. As stated previously, we will work closely with your staff to eliminate administrative errors prior to the issuance of future amendments.

Sincerely,

A handwritten signature in black ink, appearing to read "Daniel G. McDonald".

Daniel G. McDonald, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosure:
As stated

cc w/enclosure:
See next page

J. W. Williams, Jr.
Florida Power and Light Company

Turkey Point Plants
Units 3 and 4

cc: Harold F. Reis, Esquire
Newman and Holtziner P.C.
1615 L Street, N.W.
Washington, DC 10036

Administrator
Department of Environmental
Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 33130

James P. O'Reilly
Regional Administrator, Region II
U.S Nuclear Regulatory Commission
Suite 2900
101 Marietta Street
Atlanta, GA 30303

Norman A. Coll, Esquire
Steel, Hector and Davis
4000 Southeast Financial
Center
Miami, Florida 33131-2398

Martin H. Hodder, Esquire
1131 N.E. 86th Street
Miami, Florida 33138

Mr. Ken N. Harris, Vice President
Turkey Point Nuclear Plant
Florida Power and Light Company
P.O. Box 013100
Miami, Florida 33101

Joette Lorion
7269 SW 54 Avenue
Miami, Florida 33143

Mr. M. R. Stierheim
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Mr. Chris J. Baker, Plant Manager
Turkey Point Nuclear Plant
Florida Power and Light Company
P.O. Box 013100
Miami, Florida 33101

Resident Inspector
Turkey Point Nuclear Generating Station
U.S. Nuclear Regulatory Commission
Post Office Box 57-1185
Miami, Florida 33257-1185

Attorney General
Department of Legal Affairs
The Capitol
Tallahassee, Florida 32304

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.W.
Atlanta, GA 30308

Mr. Allan Schubert, Manager
Public Health Physicist
Department of Health and
Rehabilitative Services
1323 Winewood Blvd.
Tallahassee, Florida 32301

Mr. Jack Shreve
Office of the Public Counsel
Room 4, Holland Building
Tallahassee, Florida 32304

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 104 FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 98 FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NO. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages

Table 3.5-2
Table 3.5-2 (Cont'd.)

Insert Pages

Table 3.5-2
Table 3.5-2 (Cont'd.)

TABLE 3.5-2

ENGINEERED SAFETY FEATURES ACTUATION

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>MIN.</u> <u>OPERABLE</u> <u>CHANNELS</u>	<u>2</u> <u>MIN.</u> <u>DEGREE</u> <u>OF</u> <u>REDUN-</u> <u>DANCY</u>	<u>3</u> <u>OPERATOR ACTION</u> <u>IF CONDITIONS OF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>
1.	SAFETY INJECTION			
1.1	Manual	1	0	Cold Shutdown
1.2	High Containment Pressure	2	1	Cold Shutdown
1.3	High Differential Pressure between any Steam Line and the Steam Line Header*	2	1	Cold Shutdown
1.4	Pressurizer Low Pressure*	2	1	Cold Shutdown
1.5	High Steam Flow in 2/3 Steam Lines with Low T _{avg} or Low Steam Line Pressure**	1/line in each of 2 lines	1	Cold Shutdown
2.	CONTAINMENT SPRAY			
2.1	High Containment Pressure and High-High Containment Pressure (coincident)	2 per set	1/set	Cold Shutdown
3.	AUXILIARY FEEDWATER			
3.1	Low-Low Steam Generator Level	2	1	Hot Shutdown
3.2	Loss of Power			
a.	4.16kV Emergency Bus undervoltage (Loss of Voltage)	2	0	Cold Shutdown
b.	480V Load Centers (2 instantaneous relays per load center)***	1****	0	Cold Shutdown
c.	480V Load Centers (2 inverse time relays per load center)***	1****	0	Cold Shutdown

8502110137 850125
PDR ADOCK 05000250
P PDR

TABLE 3.5-2 (Cont'd.)

ENGINEERED SAFETY FEATURES ACTUATION

<u>NO.</u>	<u>FUNCTIONAL UNIT</u>	<u>1</u> <u>MIN.</u> <u>OPERABLE</u> <u>CHANNELS</u>	<u>2</u> <u>MIN.</u> <u>DEGREE</u> <u>OF</u> <u>REDUN-</u> <u>DANCY</u>	<u>3</u> <u>OPERATOR ACTION</u> <u>IF CONDITIONS OF</u> <u>COLUMN 1 OR 2</u> <u>CANNOT BE MET</u>
3.3	Safety Injection		(---See 1 above---)	
3.4	Trip of both Main Feedwater Pump Breakers	2	0	Cold Shutdown

* This signal may be manually bypassed, when the reactor is shutdown and pressure is below 2000 psig.

** This signal may be manually bypassed, when cooling down the reactor and T_{AVG} is below 543°F.

*** These items do not apply on Unit 3 until after implementation of PC/M 79-116 and on Unit 4 until after implementation of PC/M 80-44.

**** Operation or start-up may continue with only one channel operable only if the inoperable channel is placed in the trip condition.