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Docket Nos. 50-250
 and 50-251

Dr. Robert E. Uhrig, Vice President
 Advanced Systems and Technology
 Florida Power and Light Company
 Post Office Box 14000
 Juno Beach, Florida 33408

Dear Dr. Uhrig:

By letter dated October 26, 1983, we issued Amendment Nos. 97 and 92 to Facility Operating License Nos. DPR-31 and DPR-41, for the Turkey Point Plant Unit Nos. 3 and 4, respectively.

We inadvertently assigned an incorrect Amendment Number for Facility Operating License No. DPR-41. The correct Amendment Number is 91. Enclosed are the revised Amendment for DPR-41, Technical Specifications for DPR-31 and DPR-41 and the Safety Evaluation reflecting the correct Amendment Numbers 97 and 91.

Please accept our apologies for any inconveniences this administrative error may have caused.

Sincerely,

Daniel G. McDonald, Project Manager
 Operating Reactors Branch No. 1
 Division of Licensing

Enclosures:

1. Amendment No. 91 to DPR-41
2. Revised Technical Specification pages
3. Revised Safety Evaluation

cc w/enclosures:
 See next page

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Robert E. Uhrig
Florida Power and Light Company

cc: Harold F. Reis, Esquire
Lowenstein, Newman, Reis and Axelrad
1025 Connecticut Avenue, N.W.
Suite 1214
Washington, D. C. 20036

Bureau of Intergovernmental Relations
660 Apalachee Parkway
Tallahassee, Florida 33130

Norman A. Coll, Esquire
Steel, Hector and Davis
1400 Southeast First National
Bank Building
Miami, Florida 33131

Mr. Henry Yaeger, Plant Manager
Turkey Point Plant
Florida Power and Light Company
P. O. Box 013100
Miami, Florida 33101

Mr. M. R. Stierheim
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Resident Inspector
Turkey Point Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Post Office Box 1207
Homestead, Florida 33030

Regional Radiation Representative
EPA Region IV
345 Courtland Street, N.W.
Atlanta, Georgia 30308

Mr. Jack Shreve
Office of the Public Counsel
Room 4, Holland Building
Tallahassee, Florida 32304

Administrator
Department of Environmental
Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301

James P. O'Reilly
Regional Administrator - Region II
U. S. Nuclear Regulatory Commission
101 Marietta Street - Suite 3100
Atlanta, Georgia 30303



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. DPR-41

- I. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated May 13, 1983, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 91, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: October 26, 1983

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-41

Revise Appendix A as follows:

Remove Pages

Insert Pages

4.18-1

Table 4.18-1

B 4.18-1 and -2

4.18 SAFETY RELATED SYSTEMS FLOWPATH

Applicability: Applies to the availability of the required flowpaths for the systems specified in Table 4.18-1.

Objective: To verify the availability of an operable flowpath for the systems specified in Table 4.18-1.

Specification: Monthly, perform a system walkdown, as specified in Table 4.18-1, to demonstrate the availability of required flowpaths by:

1. Verifying that each accessible valve (manual, power operated, or automatic) is in its correct position.
2. Verifying the availability of power to those components related to the operability of the designated flowpaths.

TABLE 4.18-1
MINIMUM FREQUENCIES FOR SAFETY RELATED SYSTEMS FLOWPATH VERIFICATIONS

<u>SYSTEM DESCRIPTION</u> (Note 1)	<u>FREQUENCY</u>	<u>APPLICABILITY MODE</u>
1. High Head Safety Injection	M,P	1,2
2. Low Head Safety Injection	M,P	1,2
3. Auxiliary Feedwater	M,P	1,2(Note 2)
4. Containment Spray	M,P	1,2
5. Emergency Diesel Generators	M	1,2(Note2)
6. Component Cooling Water	M,P	1,2
7. Intake Cooling Water	M,P	1,2
8. Boric Acid Flowpath to the Core	M	1,2,3,4
9. Post-accident Containment Ventilation	M,P	1,2,(Note 2)
10. In-plant AC Electrical Distribution	M,P	1,2
11. Post-accident Hydrogen Monitoring	M	1,2,3,4 (Note 2)
12. Post-accident Sampling	M	1,2,3,4 (Note 2)
13. Fire Suppression Water System	M	1,2,3,4 (Note 2)

Frequency:

M - Monthly

P - Within one surveillance Interval prior to criticality.

Operational modes as used for applicability:

MODE	K_{eff}	% RATED THERMAL POWER	T_{avg}
1. Power Operation	>0.99	$>2\%$	$>547^{\circ}F$
2. Hot Shutdown	<0.99	0	$>540^{\circ}F$
3. Cold Shutdown	<0.99	0	$<200^{\circ}F$
4. Refueling Shutdown	<0.90	0	$>160^{\circ}F$

NOTES:

1. Refer to Bases T.S. 84.18 for definitions of systems required flowpaths.
2. These are shared systems. For this reason, with either reactor being within the applicable modes of operation, the flowpath verification shall be performed for that unit at the designated frequency.

B4.18 BASES FOR SAFETY RELATED SYSTEM FLOWPATH VERIFICATION

This surveillance is designed to verify that flowpaths exist in order for the specified safety related systems to perform as required by Section 14 of the FSAR.

Verify that all readily accessible valves that are in the flowpath of the safety related systems listed below are in the proper positions to fulfill the described requirements of the systems. Also verify that power is being fed through the in-plant AC electrical distribution system from 4160-volt buses down to the 480-volt MCC's.

1. The high head safety injection system, in the event of a LOCA, provides borated water from the RWST, through the HHSI pumps, to the reactor core.
2. The low head safety injection system, in the event of a LOCA, provides borated water from the RWST through the RHR pumps, through the RHR heat exchangers, to the reactor core.
3. The containment spray system, in the event of a LOCA, ensures that containment pressure does not exceed design pressure by delivering borated water from the RWST, through the containment spray pumps, into the containment atmosphere.
4. The component cooling water system provides water from the component cooling water pumps, through the component cooling water heat exchangers, to all safety related equipment utilizing component cooling water as a heat sink, and back to the component cooling water pumps.
5. The intake cooling water system provides a supply of cooling water from the intake, through the intake cooling water pumps, through the component cooling water heat exchangers.
6. The auxiliary feedwater system, in the event of a loss of normal feedwater, provides a heat sink for the reactor coolant system. Steam from the main steam lines of Unit 3 and Unit 4 is supplied to the auxiliary feedwater pump turbines and auxiliary feedwater from the condensate storage tanks, through the auxiliary feedwater pumps, is then delivered to the main feedwater lines.
7. The emergency diesel generators are on-site sources of emergency power to safeguard equipment needed to control the maximum hypothetical accident on one unit and maintain the other unit at hot shutdown. The diesel generator fuel supply from the day tank to the skid tank and the air start system from the compressors to the starting motors are essential to the operation of the emergency diesel generators.
8. The boric acid system provides chemical neutron absorber (for chemical reactivity control) from the boric acid storage tanks through the charging pumps and/or VCT via the normal and/or emergency boration path, to the reactor core. (This does not include heat tracing)

9. The post-accident containment ventilation system, during accident conditions, facilitates controlled venting of the containment atmosphere from the containment penetration, through the filters, through the waste gas decay tanks, to the outside atmosphere.
10. The in-plant AC electrical distribution system provides sufficient normal and emergency auxiliary electrical power, from the 4160-volt buses to the 480-volt load centers to the 480-volt motor control centers, in order to assure the capability for a safe and orderly shutdown as well as continued maintenance of the units in a safe condition under all credible circumstances.
11. The fire protection water system provides water from the raw water storage tank, through the fire pumps, and through the fire water distribution system to extinguish any probable combination of simultaneous fires which might occur.
12. The post-accident sampling system provides a safe method for determining the isotopic activity in the reactor coolant system. Samples of the reactor coolant are delivered from the containment to the liquid and gas analyzing equipment where radioisotopic identification and concentration, dissolved hydrogen and oxygen concentration, boron concentration, chloride concentration, and pH measurements are made. The gas samples are then returned to the containment via R-11 and R-12 and liquid samples to the containment sump via instrument air bleed lines.
13. The post-accident hydrogen monitoring system, during accident conditions, provides samples of the containment atmosphere to the hydrogen analyzing equipment and back to the containment.

Verifying the existence of flowpaths will help prevent the possibility that one of the safety related systems is inadvertently rendered non-functional and incapable of performing the described safety function.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 97 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 91 TO FACILITY OPERATING LICENSE NO. DPR-41
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT PLANT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

I. Introduction

By letter to the NRC dated May 13, 1983, Florida Power and Light Company (the licensee) has proposed changes to the Technical Specifications for Turkey Point Units 3 and 4 which will incorporate an additional requirement for a monthly walkdown of all accessible safety-related flowpaths to verify the flowpaths exist to assure the safety-related systems can perform their function. The proposed changes require that each valve (manual, power operated or automatic) is in its correct position and verify the availability of power to those components required for operability of the designated flowpaths. These requirements will be included in Section 4.18 "Safety Related System Flowpaths." Table 4.18-1 identifies the systems, specifies the frequency and the applicable operational mode of the reactor.

II. Evaluation

We have reviewed the information submitted by the licensee requesting that Appendix A to Facility Operating License DPR-31 and DPR-41 be amended to include a requirement for a monthly walkdown of all safety-related flowpaths to verify correct valve position and have concluded the proposed Technical Specifications are acceptable.

Our conclusion is based on the following:

1. Technical Specification (TS) 4.18 including Table 4.18-1 - This proposed change will support a recent commitment made to the NRC by which all safety systems as shown on Table 4.18-1 will be verified to ensure proper line-ups and availability of flowpaths at monthly intervals. These additional TS changes are consistent with the Standard Technical Specifications for Westinghouse Pressurized Water Reactors, NUREG-0452, Revision 4, paragraph 4.5.2.6.2.

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2. B14.18 - This section describes the basis, including the needs and purpose, for verifying the availability of safety systems flowpath which will provide additional assurance that the systems will perform as designed.
3. Administrative Procedure Q103.19, dated August 18, 1983, Monthly Verification of Safety Related Systems Flowpaths, has been reviewed by NRC Region II staff and adequately implements the proposed TS 4.18.

III. Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

IV. Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 26, 1983