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MAY 12 1981



Dr. Robert E. Uhrig, Vice President
 Advanced Systems and Technology
 Florida Power and Light Company
 Post Office Box 529100
 Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. ⁶⁶65 to Facility Operating License No. DPR-31 and Amendment No. ⁵⁸57 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated September 12, 1980 and March 10, 1981.

These amendments revise the Technical Specifications to include a clarification of the term "operable" as it applies to the single failure criterion for safety systems in power reactors and includes an update of the organization chart.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original signed by:
 S. A. Varga

Steven A. Varga, Chief
 Operating Reactors Branch #1
 Division of Licensing

Enclosures:

1. Amendment No. ⁶⁶65 to DPR-31
2. Amendment No. ⁵⁸57 to DPR-41
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
 See next page

8105180 043

OFFICE	ORB#1:DL	ORB#1:DL	E-ORB#1:DL	AD-OR#1	OELD	ORB-1
SURNAME	<i>DEisenhut</i>	MGrotenhuis	SVarga	Novak	<i>S. Gorobzine</i>	Miner <i>SM</i>
DATE	5/5/81	5/5/81:cb	5/6/81	5/11/81	5/6/81	5/6/81

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Dear Dr. Uhrig:

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See next page

ORB#1:DL
MGrotenhuis
5/5/81:cb

C-ORB#1:DL
SVarga
5/4/81

AP-30:DL
Novak
5/4/81

OELD
5/4/81

ORB#1:DL
Finer
5/4/81

8105180-043



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

May 12, 1981

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 65 to Facility Operating License No. DPR-31 and Amendment No. 58 to Facility Operating License No. DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your applications transmitted by letters dated September 12, 1980 and March 10, 1981.

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Operating Reactors Branch #1
Division of Licensing

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1. Amendment No. 65 to DPR-31
2. Amendment No. 58 to DPR-41
3. Safety Evaluation
4. Notice of Issuance

cc w/enclosures:
See next page

8105180043

Robert E. Uhrig
Florida Power and Light Company

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Lowenstein, Newman, Reis and Axelrad
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Miami, Florida 33199

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Florida Power and Light Company
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Honorable Dewey Knight
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Resident Inspector
Turkey Point Nuclear Generating Station
U. S. Nuclear Regulatory Commission
Post Office Box 1207
Homestead, Florida 33030

Director, Criteria and Standards Division
Office of Radiation Programs (ANR-460)
U. S. Environmental Protection Agency
Washington, D. C. 20460

U. S. Environmental Protection Agency
Region IV Office
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Atlanta, Georgia 30308

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Administrator
Department of Environmental
Regulation
Power Plant Siting Section
State of Florida
2600 Blair Stone Road
Tallahassee, Florida 32301



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT PLANT UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6~~5~~
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Florida Power and Light Company (the licensee) dated September 12, 1980 and March 10, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

8105180048

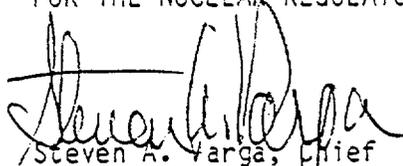
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 65, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 12, 1981



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50- 251

TURKEY POINT PLANT UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 58
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Florida Power and Light Company (the licensee) dated September 12, 1980 and March 10, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the applications, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

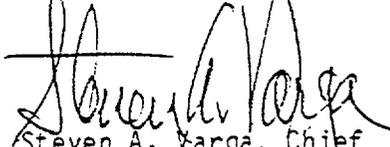
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 57, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Yarga, Chief
Operating Reactors Branch #1
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: May 12, 1981

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

<u>Remove Pages</u>	<u>Insert Pages</u>
1-1	1-1
--	3.0-1
--	B3.0-1
--	B3.0-2
3.1-1	3.1-1
3.3-1	3.3-1
3.4-2	3.4-2
3.4-3	3.4-3
3.4-4	3.4-4
3.4-5	3.4-5
3.4-6	3.4-6
3.5-1	3.5-1
3.6-2	3.6-2
3.7-1	3.7-1
3.7-2	3.7-2
3.13-1	3.13-1
3.15-1	3.15-1
6-1	6-1
6-2	6-2
6-3	6-3
6-4	6-4

Remove Pages

6-7

6-8

6-9

6-10

6-15

Insert Pages

6-7

6-8

6-9

6-10

6-15

1.0 DEFINITIONS

The definitions used for these specifications follow.

1.1 SAFETY LIMITS

Safety limits are the necessary quantitative restrictions placed upon those process variables that must be controlled in order to reasonably protect the integrity of certain of the physical barriers which guard against the uncontrolled release of radioactivity. If any safety limit is exceeded, the associated reactor shall be shut down until the AEC authorizes resumption of operation.

1.2 LIMITING SAFETY SYSTEM SETTINGS

Limiting safety system settings are set points for automatic protective devices responsive to the variables on which safety limits have been placed. These set points are so chosen that automatic protective actions will correct the most severe, anticipated abnormal situation so that a safety limit is not exceeded.

1.3 LIMITING CONDITIONS FOR OPERATION

Limiting conditions for operation are those restrictions on reactor operation, resulting from equipment performance capability, that must be enforced to ensure safe operation of the facility.

1.4 OPERABLE - OPERABILITY

A system, subsystem, train, component or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified function(s). Implicit in this definition shall be the assumption that all necessary attendant instrumentation, controls, normal and emergency electrical power sources, cooling or seal water, lubrication or other auxiliary equipment that are required for the system, subsystem, train, component or device to perform its function(s) are also capable of performing their related support function(s).

3.0 LIMITING CONDITIONS FOR OPERATION

Applicability

- 3.0.1 If one of the below listed limiting conditions for operation can not be satisfied because fewer components are operable than are required, the unit shall be placed in hot shutdown within eight hours and cold shutdown within the following 30 hours unless appropriate corrective action is taken before the time expires. This specification applies only to specifications 3.3.1, 3.4.1.b, 3.4.2.b, 3.4.3.b, 3.4.4.b, 3.4.5.b, 3.5, 3.6.d, 3.7.2 and 3.13.3.
- 3.0.2 For purposes of determining if a component is operable for LCO considerations, the component need not be considered inoperable due to inoperability of its normal or emergency power supply if all of its redundant components are operable with their normal or emergency power supplies operable.

B3.0 BASES FOR LIMITING CONDITIONS FOR OPERATION, APPLICABILITY

- 3.0.1 This specification delineates the ACTION to be taken for circumstances not directly provided for in the ACTION statements and whose occurrence would violate the intent of the specification. For example, Specification 3.4.1 requires each Reactor Coolant System accumulator to be OPERABLE and provides explicit ACTION requirements if one accumulator is inoperable. Under the terms of Specification 3.0.1, if more than one accumulator is inoperable, the unit is required to be in at least HOT SHUTDOWN within 7 hours. As a further example, Specification 3.4.2.a requires two Containment Spray Systems to be OPERABLE and provides explicit ACTION requirements if one spray system is inoperable: Under the terms of Specification 3.0.1, if both of the required Containment Spray Systems are inoperable, the unit is required to be in at least HOT SHUTDOWN within 7 hours and in at least COLD SHUTDOWN in the next 30 hours. It is assumed that the unit is brought to the required condition within the required times by promptly* initiating and carrying out the appropriate ACTION statement.
- 3.0.2 This specification delineates what additional conditions must be satisfied to permit operation to continue, consistent with the ACTION statements for power sources, when a normal or emergency power source is not OPERABLE. It specifically prohibits operation when one division is inoperable because its normal or emergency power source is inoperable and a system, subsystem, train, component or device in another division is inoperable for another reason.

The provisions of this specification permit the ACTION statements associated with individual systems, subsystems, trains, components, or devices to be consistent with the ACTION statements of the associated electrical power source. It allows operation to be governed by the time limits of the ACTION statement associated with the Limiting Condition for Operation for the normal or emergency

* Promptly means one hour to act and six hours to hot shutdown.

power source, not the individual ACTION statements for each system, subsystem, train, component or device that is determined to be inoperable solely because of the inoperability of its normal or emergency power source.

For example, Specification 3.7.1 requires in part that two emergency diesel generators be OPERABLE. The ACTION statement provides for an out-of-service time when one emergency diesel generator is not OPERABLE. If the definition of OPERABLE were applied without consideration of Specification 3.0.2, all systems, subsystems, trains, components and devices supplied by the inoperable emergency power source would also be inoperable. This would dictate invoking the applicable ACTION statements for each of the applicable Limiting Conditions for Operation. However, the provisions of Specification 3.0.2 permit the time limits for continued operation to be consistent with the ACTION statement for the inoperable emergency diesel generator instead, provided the other specified conditions are satisfied. In this case, this would mean that the corresponding normal power source must be OPERABLE, and all redundant systems, subsystems, trains, components, and devices must be OPERABLE, or otherwise satisfy Specification 3.0.2 (i.e., be capable of performing their design function and have at least one normal or one emergency power source OPERABLE). If they are not satisfied, shutdown is required in accordance with this specification.

In cold shutdown or refueling condition Specification 3.0.2 is not applicable, and thus the individual ACTION statements for each applicable Limiting Condition for Operation in these conditions must be adhered to.

3.1 REACTOR COOLANT SYSTEM

Applicability: Applies to the operating status of the Reactor Coolant System.

Objective: To specify those limiting conditions for operation of the Reactor Coolant System which must be met to assure safe reactor operation.

Specification: 1. OPERATIONAL COMPONENTS

a. Reactor Coolant Pumps

1. A minimum of ONE pump shall be in operation when the reactor is in power operation, except during low power physics tests.
2. A minimum of ONE pump, or ONE Residual Heat Removal Pump, shall be in operation during reactor coolant boron concentration reduction.
3. Reactor power shall not exceed 10% of rated power unless at least TWO reactor coolant pumps are in operation.
4. Reactor power shall not exceed 45% of rated power with only two pumps in operation unless the overtemperature ΔT trip setpoint, K_1 , for two loop operation, has been set at 0.88.
5. A reactor coolant pump shall not be started when cold leg temperature is $\leq 275^\circ\text{F}$ unless steam generator secondary water temperature is less than 50°F above the RCS temperature (including instrument error).

b. Steam Generators

1. A minimum of TWO steam generators shall be operable when the average coolant temperature is above 350F.

c. Pressurizer Safety Valves

1. ONE valve shall be operable whenever the head is on the reactor vessel except during hydrostatic test.
2. THREE valves shall be operable when the reactor coolant average temperature is above 350F or the reactor is critical.

3.3 CONTAINMENT

Applicability: Applies to the integrity of the containment.

Objective: To define the operating status of the containment.

Specification: CONTAINMENT INTEGRITY

1. The containment integrity (as defined in 1.5) shall not be violated unless the reactor is in the cold shutdown condition. Specification 3.0.1 applies to 3.3.1.
2. The containment integrity shall not be violated when the reactor vessel head is removed unless the reactor is in the refueling shutdown condition.

INTERNAL PRESSURE

If the internal pressure exceeds 3 psig or the internal vacuum exceeds 2 psig, the condition shall be corrected within 8 hours or the reactor shall be brought to hot shutdown.

5. TWO residual heat removal pumps shall be operable.
 6. TWO residual heat exchangers shall be operable.
 7. All valves, interlocks and piping associated with the above components and required for post accident operation, shall be operable, except valves that are positioned and locked. Valves 864-A, B; 862-A, B; 865-A, B, C; 866-A, B shall have power removed from their motor operators by locking open the circuit breakers at the Motor Control Centers. The air supply to valve 758 shall be shut off to the valve operator.
- b. During power operation, the requirements of 3.4.1a may be modified to allow one of the following components to be inoperable (including associated valves and piping) at any one time except for the cases stated in 3.4.1.b.2. If the system is not restored to meet the requirements of 3.4.1a within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.4.1a are not satisfied within an additional 48 hours the reactor shall be placed in the cold shutdown condition. Specification 3.0.1 applies to 3.4.1.b.
1. ONE accumulator may be out of service for a period of up to 4 hours.
 2. ONE of FOUR safety injection pumps may be out of service for 30 days. A second safety injection pump may be out of service, provided the pump is restored to operable status within 24 hours. TWO of the FOUR safety injection pumps shall be tested to demonstrate operability before initiating maintenance of the inoperable pumps.
 3. ONE channel of heat tracing on the flow path may be out of service for 24 hours.

5. ONE residual heat exchanger may be out of service for a period of 24 hours.
6. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating maintenance, all valves that provide the duplicate function shall be tested to demonstrate operability.

2. EMERGENCY CONTAINMENT COOLING SYSTEMS

- a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met:
 1. THREE emergency containment cooling units are operable.
 2. TWO containment spray pumps are operable.
 3. All valves and piping associated with the above components, and required for post accident operation, are operable.
- b. During power operation, the requirements of 3.4.2a may be modified to allow one of the following components to be inoperable (including associated valves and piping) at any one time. If the system is not restored to meet the requirements of 3.4.2a within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.4.2a are not satisfied within an additional 48 hours the reactor shall be placed in the cold shutdown condition. Specification 3.0.1 applies to 3.4.2.b.

1. ONE emergency containment cooling unit may be out of service for a period of 24 hours. Prior to initiating maintenance the other TWO units shall be tested to demonstrate operability.
2. ONE containment spray pump may be out of service provided it is restored to operable status within 24 hours. The remaining containment spray pump shall be tested to demonstrate operability before initiating maintenance on the inoperable pump.
3. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves that provide the duplicate function shall be tested to demonstrate operability.

3. EMERGENCY CONTAINMENT FILTERING SYSTEM

- a. The reactor shall not be made critical, except for low power physics tests unless:
 1. THREE emergency containment filtering units are operable.
 2. All valves, interlocks and piping associated with the above components and required for post-accident operation, are operable.
- b. During power operation:
 1. ONE unit may be inoperable for a period of 24 hours if the other TWO are operable.
 2. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating maintenance, all valves that provide the duplicate function shall be tested to demonstrate operability.
 3. Specification 3.0.1 applies to 3.4.3.b.

4. COMPONENT COOLING SYSTEM

- a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met:

1. THREE component cooling pumps are operable.
 2. THREE component cooling heat exchangers are operable.
 3. All valves, interlocks and piping associated with the above components are operable.
- b. During power operation, the requirements of 3.4.4a may be modified as stated below. If the system is not restored to meet the conditions of 3.4.4a within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.4.4a are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition. Specification 3.0.1 applies to 3.4.4.b.
1. ONE pump may be out of service for 7 days.
 2. ONE additional pump and ONE heat exchanger may be out of service for a period of 24 hours.

5. INTAKE COOLING WATER SYSTEM

- a. The reactor shall not be made critical unless the following conditions are met:
1. THREE intake cooling water pumps and TWO headers are operable.
 2. All valves, interlocks and piping associated with the operation of these pumps, and required for post accident operation, are operable.

b. During power operation, the requirements of 3.4.5.a., above, may be modified to allow any one of the following components to be inoperable provided the remaining systems are in continuous operation. If the system is not restored to meet the requirements of 3.4.5.a. within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.4.5.a. are not satisfied within an additional 48 hours, the reactor shall be placed in the cold shutdown condition. Specification 3.0.1 applies to 3.4.5.b. †

1. One of the two headers may be out of service for a period of 24 hours.
2. One intake cooling water pump may be out of service for a period of 24 hours.

3.5 INSTRUMENTATION

Applicability: Applies to reactor and safety features instrumentation systems.

Objective: To delineate the conditions of the instrumentation and safety circuits necessary to ensure reactor safety.

Specification: Tables 3.5-1 through 3.5-4 state the minimum instrumentation operating conditions.

Specification 3.0.1 applies to Tables 3.5-1 through 3.5-3.

1. TWO associated charging pumps shall be operable.
 2. THREE boric acid transfer pumps shall be operable.
 3. The boric acid tanks together shall contain a minimum of 6160 gallons of a 20,000 to 22,500 ppm boron solution at a temperature of at least 145F.
 4. System piping, interlocks and valves shall be operable to the extent of establishing one flow path from the boric acid tanks, and one flow path from the refueling water storage tank, to each Reactor Coolant System.
 5. TWO channels of heat tracing shall be operable for the flow path from the boric acid tanks.
 6. The primary water storage tank contains not less than 30,000 gallons of water.
- d. During power operation, the requirements of 3.6.b and c may be modified to allow one of the following components to be inoperable. If the system is not restored to meet the requirements of 3.6b and c within the time period specified, the reactor(s) shall be placed in the hot shutdown condition. If the requirements of 3.6.b and c are not satisfied within an additional 48 hours, the reactor(s) shall be placed in the cold shutdown condition. Specification 3.0.1 applies to 3.6.d.
1. One of the two operable charging pumps may be removed from service provided that it is restored to operable status within 24 hours.
 2. One boric acid transfer pump may be out of service provided that it is restored to operable status within 24 hours.
 3. One channel of heat tracing may be out of service for 24 hours.

3.7 ELECTRICAL SYSTEMS

Applicability: Applies to the availability of electrical power for the operation of auxiliaries.

Objective: To define those conditions of electrical power availability necessary (1) to provide for safe reactor operation, and (2) to provide for the continuing availability of engineered safety features.

- Specification:
1. Either reactor shall not be started from a cold shutdown without:
 - a. The associated 239 KV-4160 volt start-up transformer in service.
 - b. 4160-volt busses A and B of the associated unit, and either bus A or B of the second unit, energized.
 - c. THREE out of FOUR 480-volt load centers and 480-volt motor control centers A, B or C, and D of the associated unit energized.
 - d. TWO diesel generators operable with on site supply of 40,000 gallons of fuel available.
 - e. Four batteries and associated DC systems are operable with FOUR out of SIX battery chargers operable.
 2. During power operation or restarting from hot shutdown the following components may be inoperable:
 - a. ONE start-up transformer may be out of service provided both diesel generators are operable. The NRC shall be notified within 24 hours and be advised of plans to restore the transformer to service.

- b. Power operation may continue if ONE diesel generator is out of service provided (1) the remaining diesel generator is tested daily and its associated engineered safety features are operable, and (2) either start-up transformer is operable. If the diesel outage is to be seven (7) days or more the NRC shall be notified. |
- c. ONE battery may be out of service for a period of twenty four hours.
- d. Specification 3.0.1 applies to 3.7.2. |

3.13

SHOCK SUPPRESSORS (SNUBBERS)

Applicability: Applies to the operational status of safety-related pipe restraints (snubbers).

Objective: To define the limiting conditions for operation applied to the operability of safety-related snubbers.

- Specification:
1. During all modes of operation except Cold Shutdown and Refueling Shutdown, all (safety-related) snubbers listed in Table 3.13-1 shall be operable except as noted in 3.13.2 through 3.13.4 below.
 2. From the time that a snubber is determined to be inoperable, continued reactor operation is permissible only during the succeeding 72 hours unless the snubber is sooner made operable or replaced with an operable snubber.
 3. If the requirements of 3.13.1 and 3.13.2 cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a cold shutdown condition within 36 hours. Specification 3.0.1 applies to 3.13.3. |
 4. If a hydraulic snubber is determined to be inoperable while the reactor is in the cold shutdown mode or the refueling mode, the snubber shall be made operable or replaced with an operable snubber prior to reactor startup.
 5. Snubbers may be added to Table 3.13-1 without prior license amendment provided that a revision to Table 3.13-1 is included with a license amendment request within 60 days.

3.15

OVERPRESSURE MITIGATING SYSTEM

Applicability: Establishes operating limitations to assure that the limits of 10 CFR 50, Appendix G, are not exceeded.

Objectives: To minimize the possibility of an overpressure transient which could exceed the limits of 10 CFR 50, Appendix G.

- Specification:
1. At RCS temperature less than or equal to 380°F, valves MOV-*-843 A, MOV-*-843 B, MOV-*-866 A, and MOV-*-866 B shall be closed.
 2. If any of the valves in 3.15.1 are found to be open while RCS temperature is less than or equal to 380°F, perform at least one of the following within the next 8 hours:
 - a. block the corresponding flow path to the reactor vessel,
 - b. close the valve, or
 - c. depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or
 - d. verify at least one pressurizer power operated relief valve is maintained open.
 3. At RCS temperature less than or equal to 275°F, two pressurizer power operated relief valves shall be operable at the low setpoint range.
 - a. If one power operated relief valve is inoperable with RCS temperature less than or equal to 275°F, perform at least one of the following within 7 days:
 - (1) restore operability of the power operated relief valve, or
 - (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or
 - (3) verify at least one pressurizer power operated relief valve is maintained open.
 - b. If both power operated relief valves are inoperable with RCS temperature less than or equal to 275°F, perform at least one of the following within the next 24 hours:
 - (1) restore operability of at least one power operated relief valve, or
 - (2) depressurize and vent the RCS through an opening with an area of at least 2.20 square inches, or
 - (3) verify at least one pressurizer power operated relief valve is maintained open.

6.0 ADMINISTRATIVE CONTROLS

6.1 RESPONSIBILITY

6.1.1 The Plant Manager - Nuclear shall be responsible for overall licensed facility operation and shall delegate in writing the succession to this responsibility during his absence.

6.2 ORGANIZATION

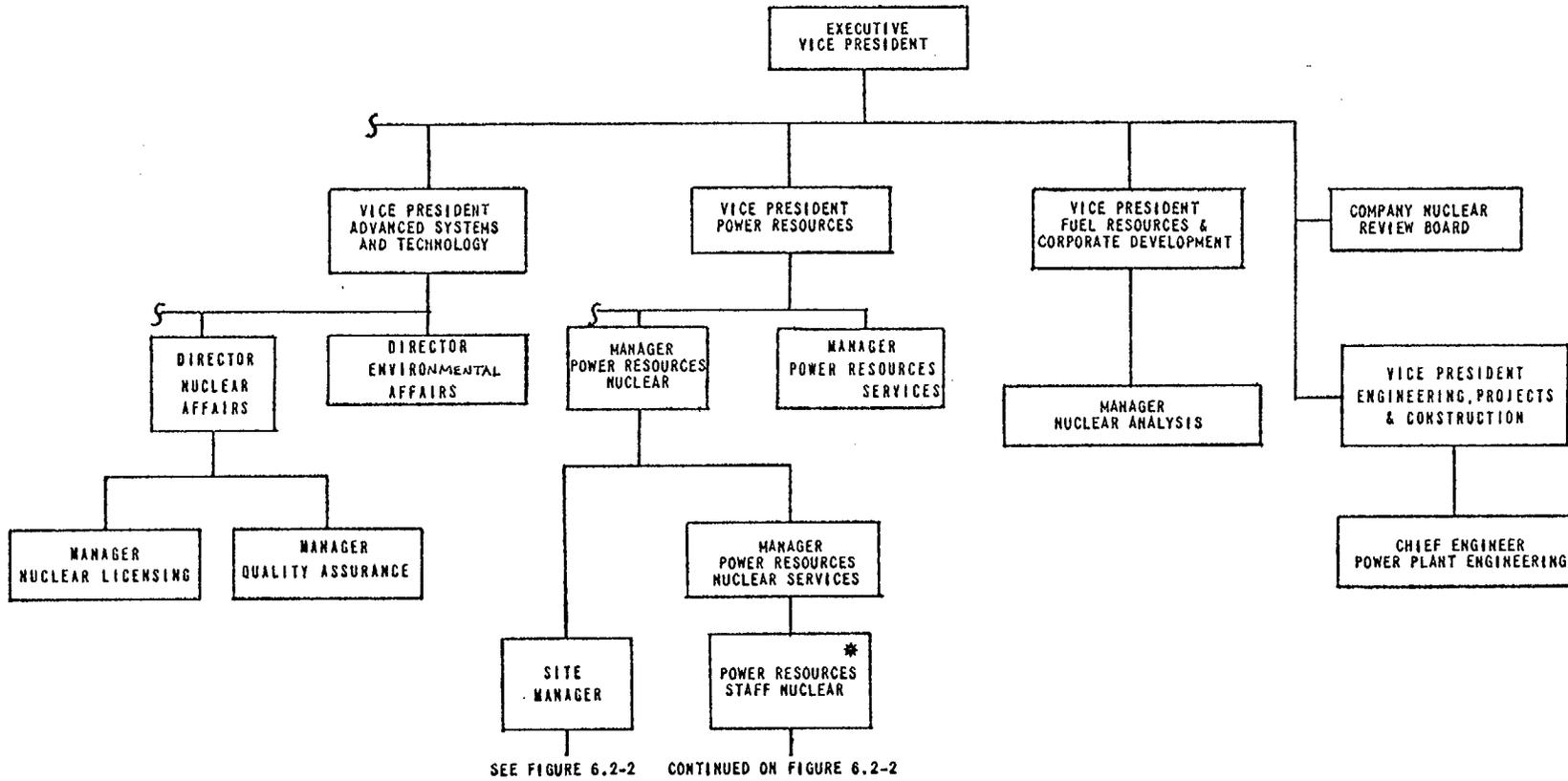
6.2.1 OFFSITE

The offsite organization for facility management and technical support shall be shown on Figure 6.2-1.

6.2.2 FACILITY STAFF

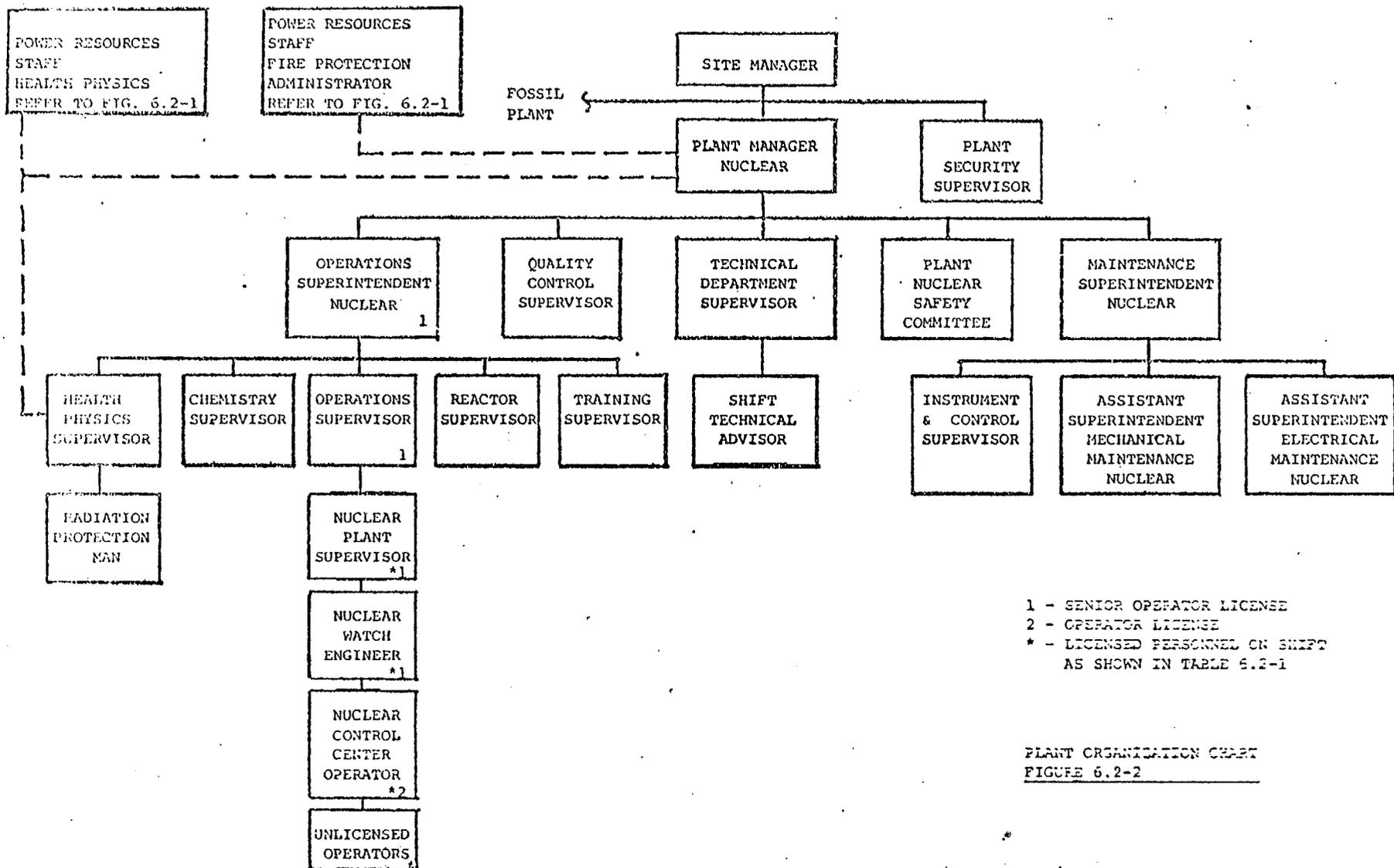
The Facility organization shall be as shown on Figure 6.2-2 and:

- a. Each on duty shift shall be composed of at least the minimum shift crew composition shown in Table 6.2-1.
- b. At least one licensed Operator shall be in the control room when fuel is in the reactor.
- c. At least two licensed Operators shall be present in the control room during reactor start-up, scheduled reactor shutdown and during recovery from reactor trips.



* INCLUDES HEALTH PHYSICS, RADIOCHEMISTRY, PLANT SUPPORT, FIRE PROTECTION ADMINISTRATOR, EMERGENCY PLANNING, ETC.

FIGURE 6.2-1 OFFSITE ORGANIZATION FOR FACILITY MANAGEMENT AND TECHNICAL SUPPORT



- 1 - SENIOR OPERATOR LICENSE
- 2 - OPERATOR LICENSE
- * - LICENSED PERSONNEL ON SHIFT AS SHOWN IN TABLE 6.2-1

PLANT ORGANIZATION CHART
FIGURE 6.2-2

6.4.2 A training program for the Fire Emergency response members shall be maintained under the direction of the Fire Protection Administrator and should meet or exceed the requirements of Section 27 of the NFPA Code-1975, except for fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 Plant Nuclear Safety Committee (PNSC)

6.5.1.1 FUNCTION

The PNSC shall function to advise the Plant Manager-Nuclear on all matters related to nuclear safety.

6.5.1.2 COMPOSITION

The Plant Nuclear Safety Committee shall be composed of the:

1. Chairman: Plant Manager - Nuclear
2. Vice Chairman: Operations Superintendent - Nuclear
3. Technical Department Supervisor
4. Maintenance Superintendent - Nuclear
5. Instrument and Control Supervisor
6. Health Physics Supervisor
7. Reactor Supervisor

6.5.1.3 ALTERNATES

Alternate members shall be appointed in writing by the PNSC Chairman to serve on a temporary basis; however, no more than two alternates shall participate in PNSC activities at any one time.

6.5.1.4 MEETING FREQUENCY

The PNSC shall meet at least once per calendar month and as convened by the PNSC Chairman.

6.5.1.5 QUORUM

A quorum of the PNSC shall consist of the Chairman or Vice Chairman and four (4) members including alternates.

6.5.1.6 RESPONSIBILITIES

The Plant Nuclear Safety Committee shall be responsible for:

- a. Review of 1) all procedures and changes thereto required by Section 6.8 and 2) any other proposed procedures or changes thereto as determined by the Plant Manager - Nuclear to affect nuclear safety.
- b. Review of all proposed tests and experiments that affect nuclear safety.
- c. Review of all proposed changes to the Technical Specifications in Appendix A of the license.
- d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety.
- e. Investigation of all violations of the Technical Specifications and preparation and forwarding a report covering evaluation and recommendations to prevent recurrence to the Manager of Power Resources - Nuclear, to the Vice President of Power Resources and to the Chairman of the Company Nuclear Review Board.

- f. Review of facility operations to detect potential safety hazards.
- g. Performance of special reviews and investigations and reports thereon as requested by the Chairman of the Company Nuclear Review Board.
- h. Review of the Plant Security Plan and implementing procedures and submitting recommended changes to the Chairman of the Company Nuclear Review Board.
- i. Review of the Emergency Plan and implementing procedures and submitting recommended changes to the Chariman of the Company Nuclear Review Board.

6.5.1.7 AUTHORITY

The Plant Nuclear Safety Committee shall:

- a. Recommend to the Plant Manager - Nuclear written approval or disapproval (in minutes of PNSC meeting) of items considered under 6.5.1.6(a) through (d) above.
- b. Render determinations in writing (in minutes of PNSC meetings) with regard to whether or not each item considered under 6.5.1.6(a) through (e) above constitutes an unreviewed safety question.
- c. Provide immediate written notification to the Vice President - Power Resources and the Company Nuclear Review Board of disagreement

between the PNSC and the Plant Manager - Nuclear; however, the Plant Manager - Nuclear shall have responsibility for resolution of such disagreements pursuant to 6.1.1 above.

6.5.1.8 RECORDS

The Plant Nuclear Safety Committee shall maintain written minutes of each meeting and copies shall be provided to the Vice President - Power Resources and Chairman of the Company Review Board.

6.5.2 COMPANY NUCLEAR REVIEW BOARD (CNRB)

6.5.2.1 FUNCTION

The Company Nuclear Review Board shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations.
- b. Nuclear engineering.
- c. Chemistry and radiochemistry.
- d. Metallurgy.
- e. Instrumentation and control
- f. Radiological safety.
- g. Mechanical and electrical engineering.
- h. Quality assurance practices.

6.5.2.2 COMPOSITION

The CNRB shall be composed of the:

1. Chairman: Vice President - Advanced Systems and Technology
2. Member: Chief Engineer - Power Plant Engineering
3. Member: Vice President - Power Resources
4. Member: Manager of Power Resources - Nuclear
5. Member: Director of Nuclear Affairs
6. Member: Power Plant Engineering Manager
7. Member: Power Plant Engineering supervisor

6.5.2.3 ALTERNATES

Alternate members shall be appointed in writing by the (CNRB) Chairman to serve on a temporary basis; however, no more than two alternates shall participate in (CNRB) activities at any one time.

6.5.2.4 CONSULTANTS

Consultants shall be utilized as determined by the CNRB to provide expert advice to the CNRB.

6.5.2.5 MEETING FREQUENCY

The CNRB shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

6.5.2.6 QUORUM

A quorum of the CNRB shall consist of the Chairman or designated acting Chairman and four (4) members including alternates. No more

- 6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PNSC and approved by the Plant Manager - Nuclear prior to implementation and periodically as provided by procedure.
- 6.8.3. Temporary changes to procedures of 6.8.1 above may be made provided:
- a. The intent of the original procedure is not altered.
 - b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operators License on the unit affected.
 - c. The change is documented, reviewed by the PNSC and approved by the Plant Manager - Nuclear within fourteen days of implementation



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 65 TO FACILITY OPERATING LICENSE NO. DPR-31
AND AMENDMENT NO. 57 TO FACILITY OPERATING LICENSE NO. DPR-41
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT PLANT UNIT NOS. 3 AND 4
DOCKET NOS. 50-250 AND 50-251

I. Introduction

By letters dated September 12, 1980 and March 10, 1981 Florida Power and Light Company (the licensee) submitted applications to amend Facility Operating License Nos. DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4. The applications requested amendments to the Technical Specifications, Appendix A to the licenses, regarding: (1) a clarification of the term operability as it applies to the single failure criterion for safety systems in power reactors and (2) an update of the organization chart, Figures 6.2-1 and 6.2-2.

Item 2 is a routine update of the organization chart. This is an administrative matter which does not decrease the safety of the plant operation and therefore is acceptable. It will not be evaluated any further.

II. Discussion

On April 10, 1980 we sent a letter to all power reactor licensees to clarify the meaning of the term "operable" and to request licensees to take specific actions to assure that it is appropriately applied at their facilities. IE Information Notice 79-35, "Control of Maintenance and Essential Equipment" also contained information on this subject.

Because of the importance of assuring safety system availability, the staff had concluded that all facility Technical Specifications should contain operability requirements similar to the NRC's Standard Technical Specifications, and that appropriate procedures should be implemented to assure that the necessary records, such as plant logs or similar documents, are reviewed to determine compliance with these specifications (1) promptly upon discovering a component, train, or subsystem to be inoperable, and (2) prior to removing a component from service.

To be consistent with our guidance the time required for hot shutdown in 3.0.1 should be increased from seven to eight hours. We have discussed this with the licensee staff. They have concurred and the change has been made.

III. Evaluation

The licensee has submitted Technical Specifications which propose to: (1) revise Specification 1.4 to incorporate an explicit definition of Operable, and (2) add Specifications 3.0.1 and 3.0.2 to ensure that the Limiting Condi-

tions for Operation do address multiple outages of redundant components, and the effects of outages of any support systems.

In addition, the Technical Specifications are revised to include references to indicate that Specification 3.0.1 applies in appropriate places.

We have reviewed the proposed revision to Technical Specification 1.4 and the addition of Technical Specifications 3.0.1 and 3.0.2 and find that they comply with the requirements of the Standard Technical Specifications as requested in our letter dated April 10, 1980. The purposes of these changes are to clarify the meaning of the term Operable and to assure proper action is taken to preserve the single failure criterion for systems that are relied upon in the FSAR. Essentially, the proposed changes do not change the intent of the existing Technical Specification; the proposed changes more precisely define the proper courses of action to ensure safe operation of the units. Based on our review we find that the proposed changes to the Technical Specifications are acceptable.

IV. Conclusions

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 12, 1981

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-250 AND 50-251FLORIDA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENT TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 65 to Facility Operating License No. DPR-31, and Amendment No. 57 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of Turkey Point Plant, Unit Nos. 3 and 4 (the facilities) located in Dade County, Florida. The amendments are effective as of the date of issuance.

These amendments revise the Technical Specifications to include a clarification of the term "operable" as it applies to the single failure criterion for safety systems in power reactors and includes an update of the organization chart.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

- 2 -

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendment dated September 12, 1980 and March 10, 1981, (2) Amendment Nos. 65 and 57 to License Nos. DPR-31 and DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 12th day of May, 1981.

FOR THE NUCLEAR REGULATORY COMMISSION



Steven A. Varga, Chief
Operating Reactors Branch #1
Division of Licensing