JUN 24 1981

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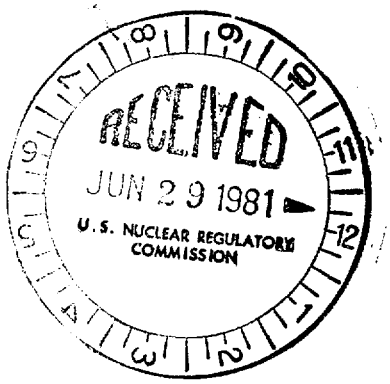

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems & Technology
Florida Power & Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 69 to Facility Operating License No. DPR-31 and Amendment No. 61 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Unit Nos. 3 and 4. The amendments consist of changes to the licenses in response to your submittal dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20, and August 4, 1978, January 26, 1979, and March 28, 1980, and incorporate the Final Order of the Atomic Safety and Licensing Board dated June 19, 1981.

These amendments approve the steam generator repair program for the Turkey Point Plant Unit Nos. 3 and 4 and provide license conditions related to the repair operation.

Copies of the Safety Evaluation (NUREG-0756) and the Final Environmental Statement (NUREG-0743) have been sent to you on December 18, 1980 and March 30, 1981, respectively. The Notice of Issuance is enclosed.

Sincerely,

Original Signed By:

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:
1. Amendment No. 69 to License DPR-31
2. Amendment No. 61 to License DPR-41
3. Notice of Issuance

cc w/enclosures:
See next page

SEE PREVIOUS CONCURRENCES

OFFICE	ORB 1	ORB 1	ORB 1	AD:OR*	OELD*	D:DL	
SURNAME	CParrish	MGrotenhuis	/rsSVarga	TNovak	Olmstead	DEisenhut	
DATE	6/ /81	6/ /81	6/ /81	6/23/81	6/24/81	6/ /81	

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and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems & Technology
Florida Power & Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. to Facility Operating License No. DPR-31 and Amendment No. to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Unit Nos. 3 and 4. The amendments consist of changes to the licenses in response to your submittal dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20, and August 4, 1978, January 26, 1979, and March 20, 1980, and incorporate the conditions ordered by the Board in its Final Order of June 19, 1981.

These amendments approve the steam generator repair program for the Turkey Point Plant Unit Nos. 3 and 4 and provide license conditions related to the repair and post-repair operation.

Copies of the Safety Evaluation (NUREG-0756) and the Final Environmental Statement (NUREG-0743) have been sent to you on December 18, 1980 and March 30, 1981, respectively. The Notice of Issuance is enclosed.

Sincerely,

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing

Enclosures:
1. Amendment No. to License DPR-31
2. Amendment No. to License DPR-41
3. Notice of Issuance

cc w/enclosures:
See next page

- see change to notice and change to conditions subject to consumer changes -

OFFICE	ORB 1	ORB 1	ORB 1	AD:OR	OELD	D:DL	
SURNAME	CParrish	MGrotenhuis/rs	SVarga	JNovak	DImstead	DEisenhut	
DATE	6/23/81	6/22/81	6/23/81	6/23/81	6/24/81	6/ /81	

NRC FORM 318 (10-80) NRCM 0240 OFFICIAL RECORD COPY USGPO: 1981—335-960

Attachment B to license conditions

Steam generator repairs will be conducted in accordance with the commitments made in the application, further described in the affidavit of Alan J. Gould dated June 12, 1981 and documented in the Atomic Safety and Licensing Board's Final Order dated June 19, 1981.

JUN 24 1981

M. GROTENHUIS

Discussed with Bill Olmstead and amended 3.I (1) to include the Board's Final Order rather than add a partially redundant 3.1.(3).

M Grotenhuis

cc: Mr. Robert Lowenstein, Esquire
Lowenstein, Newman, Reis & Axelrad
1025 Connecticut Avenue, NW
Suite 1214
Washington, D.C. 20036

Environmental & Urban Affairs Library
Florida International University
Miami, Florida 33199

Mr. Norman A. Coll, Esquire
Steel, Hector and Davis
1400 Southeast First National
Bank Building
Miami, Florida 33131

Florida Power & Light Company
ATTN: Mr. Henry Yaeger
Plant Manager
Turkey Point Plant
P. O. Box 013100
Miami, Florida 33101

Honorable Dewey Knight
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Bureau of Intergovernmental
Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
U. S. Environmental Protection Agency
Crystal Mall #2
Arlington, Virginia 20460

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, NW
Atlanta, Georgia 30308

Mr. Mark P. Oncavage
12200 S. W. 110th Avenue
Miami, Florida 33176

Neil Chonin, Esquire
1400 Ameri-First Building
One Southeast Third Avenue
Miami, Florida 33131

Henry H. Harnage, Esquire
Peninsula Federal Building, 10th Floor
200 S. E. First Street
Miami, Florida 33131

Ms. Cheryl A. Flaxman
1023 Polk Street
Hollywood, Florida 33019

Burt Saunders, Asst. County Attorney
Courthouse, 16th Floor
Miami, Florida 33131

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 69
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1977, as supplemented December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980. complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-31 is hereby amended by adding a new paragraph 3.I as follows:

   3.I Steam Generator Repair Program

   (1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 3 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.

(2) During the repair program the following temporary license conditions* will be imposed:

(a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.

(b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.

(c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a final report provided within 60 days after completion of the repair. These reports will include:

(i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.

(ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.

(iii) An estimate of radioactivity released in both liquid and gaseous effluents.

(iv) An estimate of the solid radioactive waste generated during the repair effort including volume and radioactive content.

(d) Procedures shall be prepared to assure that power can be restored by manual operator actions to the fuel pool of the unit undergoing repair within eight hours (3.2.2.2).

(e) The remedy chosen by FPL to provide the availability of the diesel fuel supply while the oil-retention dike is removed from the main diesel safety tank shall be addressed and adequately demonstrated by FPL prior to initiating the construction changes affecting the dike (3.2.2.2).

(f) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (2.7).

(g) Sixty days prior to fuel loading, FPL should submit for evaluation by the NRC a steam generator secondary water chemistry control and monitoring program (3.2.4) which will address the following:

*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.

(i) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;

(ii) Identification of the procedures used to measure the values of the critical parameters;

(iii) Identification of process sampling points;

(iv) Procedure for the recording and management of data;

(v) Procedures defining corrective actions* for off-control point chemistry conditions; and

(vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

(h) Sixty days prior to the decontamination of the channel head, FPL should meet the following conditions (3.2.5):

(i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.

(ii) Written procedures should be provided to include accountability controls of all tools, equipment, materials, and supplies that are to be used in the channel heads to prevent inadvertent entry of such items into the reactor primary coolant system. These controls should be in effect whenever the inflatable plug seals and their associated cover plates are not in place in the nozzles of the reactor coolant system piping.

(iii) Written procedures should be provided to restrict materials to be used in the channel head area to prevent the presence of materials having potential adverse effects on the reactor coolant system components (for example, chloride-bearing materials).

*Branch Technical Position MTEB 5-3 describes the acceptable means for monitoring secondary side water chemistry in PWR steam generators, including corrective actions for off-control point chemistry conditions. However, the staff is amenable to alternatives, particularly to Branch Technical Position B.3.b(9) of MTEB 5-3 (96-hour time limit to repair or plug confirmed condenser tube leaks).

(iv) Written procedures should be provided to include instructions to require that the channel head area, including the nozzles, be inspected and confirmed to be free of all loose materials, equipment, and tools prior to removing the cover plate from the inflatable plug seal.

(v) Prior to closing up the reactor coolant system and starting the RCS pumps, any loose debris, including the abrasive grits, in the channel head, RCS hot leg, and cold leg should be cleaned up.

(vi) Prior to resumption of power operation, the licensee should submit for NRC review and acceptance a report which will include an analysis of the possible effects of any foreign material which has entered the primary coolant system and has not been retrieved. The report should include all work on the decontamination and steam generator repair.

(i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6).

(j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).

(k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: JUN 2 4 1981

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 61
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1977, as supplemented December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-41 is hereby amended by adding a new paragraph 3.H as follows:

   3.H Steam Generator Repair Program

   (1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 4 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.

(2) During the repair program the following temporary license conditions* will be imposed:

(a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.

(b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.

(c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a final report provided within 60 days after completion of the repair. These reports will include:

(i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.

(ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.

(iii) An estimate of radioactivity released in both liquid and gaseous effluents.

(iv) An estimate of the solid radioactive waste generated during the repair effort including volume and radioactive content.

(d) Procedures shall be prepared to assure that power can be restored by manual operator actions to the fuel pool of the unit undergoing repair within eight hours (3.2.2.2).

(e) The remedy chosen by FPL to provide the availability of the diesel fuel supply while the oil-retention dike is removed from the main diesel safety tank shall be addressed and adequately demonstrated by FPL prior to initiating the construction changes affecting the dike (3.2.2.2).

(f) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (2.7).

(g) Sixty days prior to fuel loading, FPL should submit for evaluation by the NRC a steam generator secondary water chemistry control and monitoring program (3.2.4) which will address the following:

*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.

(i) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;

(ii) Identification of the procedures used to measure the values of the critical parameters;

(iii) Identification of process sampling points;

(iv) Procedure for the recording and management of data;

(v) Procedures defining corrective actions* for off-control point chemistry conditions; and

(vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

(h) Sixty days prior to the decontamination of the channel head, FPL should meet the following conditions (3.2.5):

(i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.

(ii) Written procedures should be provided to include accountability controls of all tools, equipment, materials, and supplies that are to be used in the channel heads to prevent inadvertent entry of such items into the reactor primary coolant system. These controls should be in effect whenever the inflatable plug seals and their associated cover plates are not in place in the nozzles of the reactor coolant system piping.

(iii) Written procedures should be provided to restrict materials to be used in the channel head area to prevent the presence of materials having potential adverse effects on the reactor coolant system components (for example, chloride-bearing materials).

*Branch Technical Position MTEB 5-3 describes the acceptable means for monitoring secondary side water chemistry in PWR steam generators, including corrective actions for off-control point chemistry conditions. However, the staff is amenable to alternatives, particularly to Branch Technical Position B.3.b(9) of MTEB 5-3 (96-hour time limit to repair or plug confirmed condenser tube leaks).

(iv) Written procedures should be provided to include instructions to require that the channel head area, including the nozzles, be inspected and confirmed to be free of all loose materials, equipment, and tools prior to removing the cover plate from the inflatable plug seal.

(v) Prior to closing up the reactor coolant system and starting the RCS pumps, any loose debris, including the abrasive grits, in the channel head, RCS hot leg, and cold leg should be cleaned up.

(vi) Prior to resumption of power operation, the licensee should submit for NRC review and acceptance a report which will include an analysis of the possible effects of any foreign material which has entered the primary coolant system and has not been retrieved. The report should include all work on the decontamination and steam generator repair.

(i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6).

(j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).

(k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut, Director
Division of Licensing
Office of Nuclear Reactor Regulation

Date of Issuance: JUN 2 4 1981

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 69 TO FACILITY OPERATING LICENSE NO. DPR-31

DOCKET NO. 50-250

Replace the following pages of Facility Operating License No. DPR-31 with the attached pages as indicated. The changed area in the license is indicated by a marginal line.

Remove Pages	Insert Pages
5	5
6	6
	7
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	9

G. Fire Protection

The licensee may proceed with and is required to provide a schedule for and to complete the modifications identified in Paragraphs 3.1.1 through 3.1.19 of the NRC's Fire Protection Safety Evaluation, dated March 21, 1979 for the facility. These modifications are to be completed prior to December 1980. If any modifications cannot be completed on schedule the licensee shall submit a report explaining the circumstances together with a revised schedule.

In addition, the licensee shall submit the additional information identified in Sections 3.1 and 3.2 of the related Safety Evaluation in accordance with the schedule contained herein. In the event these dates for submittal cannot be met, the licensee shall submit a report, explaining the circumstances, together with a revised schedule.

The licensee is required to develop and implement the administrative controls which are consistent with the licensee's letters of August 28 and November 7, 1978 within three months from the date of this amendment.

H. Safeguards Contingency Plan

The licensee shall fully implement and maintain in effect all provisions of the Commission-approved Safeguards Contingency Plan, including amendments and changes made pursuant to the authority of 10 CFR 50.54(p). The approved Contingency Plan consists of documents withheld from public disclosure pursuant to 10 CFR 2.790(d) and identified as "Turkey Point Plant Safeguards Contingency Plan" dated June 17, 1980 and submitted pursuant to 10 CFR 73.40. The Contingency Plan shall be fully implemented, in accordance with 10 CFR 73.40(b), within 30 days of this approval.

I. Steam Generator Repair Program

(1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 3 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.

(2) During the repair program the following temporary license conditions* will be imposed:

*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.

(a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.

(b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.

(c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a final report provided within 60 days after completion of the repair. These reports will include:

(i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.

(ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.

(iii) An estimate of radioactivity released in both liquid and gaseous effluents.

(iv) An estimate of the solid radioactive waste generated during the repair effort including volume and radioactive content.

(d) Procedures shall be prepared to assure that power can be restored by manual operator actions to the fuel pool of the unit undergoing repair within eight hours (3.2.2.2).

(e) The remedy chosen by FPL to provide the availability of the diesel fuel supply while the oil-retention dike is removed from the main diesel safety tank shall be addressed and adequately demonstrated by FPL prior to initiating the construction changes affecting the dike (3.2.2.2).

(f) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (2.7).

(g) Sixty days prior to fuel loading, FPL should submit for evaluation by the NRC a steam generator secondary water chemistry control and monitoring program (3.2.4) which will address the following:

(i) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;

(ii) Identification of the procedures used to measure the values of the critical parameters;

(iii) Identification of process sampling points;

(iv) Procedure for the recording and management of data;

(v) Procedures defining corrective actions* for off-control point chemistry conditions; and

(vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

(h) Sixty days prior to the decontamination of the channel head, FPL should meet the following conditions (3.2.5):

(i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.

(ii) Written procedures should be provided to include accountability controls of all tools, equipment, materials, and supplies that are to be used in the channel heads to prevent inadvertent entry of such items into the reactor primary coolant system. These controls should be in effect whenever the inflatable plug seals and their associated cover plates are not in place in the nozzles of the reactor coolant system piping.

(iii) Written procedures should be provided to restrict materials to be used in the channel head area to prevent the presence of materials having potential adverse effects on the reactor coolant system components (for example, chloride-bearing materials).

*Branch Technical Position MTEB 5-3 describes the acceptable means for monitoring secondary side water chemistry in PWR steam generators, including corrective actions for off-control point chemistry conditions. However, the staff is amenable to alternatives, particularly to Branch Technical Position B.3.b(9) of METB 5-3 (96-hour time limit to repair or plug confirmed condenser tube leaks).

(iv) Written procedures should be provided to include instructions to require that the channel head area, including the nozzles, be inspected and confirmed to be free of all loose materials, equipment, and tools prior to removing the cover plate from the inflatable plug seal.

(v) Prior to closing up the reactor coolant system and starting the RCS pumps, any loose debris, including the abrasive grits, in the channel head, RCS hot leg, and cold leg should be cleaned up.

(vi) Prior to resumption of power operation, the licensee should submit for NRC review and acceptance a report which will include an analysis of the possible effects of any foreign material which has entered the primary coolant system and has not been retrieved. The report should include all work on the decontamination and steam generator repair.

(i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6).

(j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).

(k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

4. FPL shall proceed with implementation of the recommendations set forth in paragraphs 7b and c of the "Summary and Conclusions" section of the "Final Environmental Statement Related to Operation of Turkey Point Plant, Florida Power and Light Company, Docket Nos. 50-250 and 50-251," issued July 1972 by the AEC Directorate of Licensing. No later than thirty (30) days from the date of issuance of this license, FPL shall submit to the AEC, for review and approval, its plan for the implementation of such recommendations.

5. This license is effective as of the date of issuance, and shall expire at midnight April 27, 2007.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed by D. J. Skovholt
for

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachment:
Appendix A - Technical Specifications

Date of Issuance: July 19, 1972

ATTACHMENT TO LICENSE AMENDMENT

AMENDMENT NO. 61 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NO. 50-251

Replace the following pages of Facility Operating License No. DPR-41 with the attached pates as indicated. The changed area in the license is indicated by a marginal line.

Remove Pages	Insert Pages
7	7
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H. Steam Generator Repair Program

(1) The Turkey Point Plant steam generator repair program, as described in the licensee's "Steam Generator Repair Report" dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20 and August 4, 1978, January 26, 1979 and March 28, 1980, and the affidavit of A. J. Gould dated June 12, 1981, for Unit No. 4 is approved pursuant to the Atomic Safety and Licensing Board Final Order dated June 19, 1981.

(2) During the repair program the following temporary license conditions* will be imposed:

(a) All fuel shall be removed from the reactor pressure vessel of the unit under repair and stored in the spent fuel pool.

(b) The health physics program and procedures which have been established for the steam generator repair program shall be implemented.

(c) Progress reports shall be provided at 60-day intervals from the start of the repair program and due 30 days after close of the interval with a final report provided within 60 days after completion of the repair. These reports will include:

(i) A summary of the occupation exposure expended to date using the format and detail of Table 3.3-2 of the "Steam Generator Repair Report" as supplemented.

(ii) An evaluation of the effectiveness of dose reduction techniques as specified in Section 3.3.5 of the "Steam Generator Repair Report" as supplemented in reducing occupational exposures.

(iii) An estimate of radioactivity released in both liquid and gaseous effluents.

(iv) An estimate of the solid radioactive waste generated during the repair effort including volume and radioactive content.

(d) Procedures shall be prepared to assure that power can be restored by manual operator actions to the fuel pool of the unit undergoing repair within eight hours (3.2.2.2).

(e) The remedy chosen by FPL to provide the availability of the diesel fuel supply while the oil-retention dike is removed from the main diesel safety tank shall be addressed and adequately demonstrated by FPL prior to initiating the construction changes affecting the dike (3.2.2.2).

*References in parentheses refer to the Safety Evaluation Report (NUREG-0756) December 1980.

(f) Sixty days prior to fuel loading, the program for preoperational testing and startup shall be submitted for NRC review (2.7).

(g) Sixty days prior to fuel loading, FPL should submit for evaluation by the NRC a steam generator secondary water chemistry control and monitoring program (3.2.4) which will address the following:

(i) Identification of a sampling schedule for the critical parameters and of control points for these parameters for each mode of operation: normal operation, hot startup, cold startup, hot shutdown, cold wet layup;

(ii) Identification of the procedures used to measure the values of the critical parameters;

(iii) Identification of process sampling points;

(iv) Procedure for the recording and management of data:

(v) Procedures defining corrective actions* for off-control point chemistry conditions; and

(vi) A procedure identifying (a) the authority responsible for the interpretation of the data and (b) the sequence and timing of administrative events required to initiate corrective action.

FPL should verify that the steam generator secondary water chemistry control program incorporates technical recommendations of the NSSS vendor. Any significant deviations from NSSS vendor recommendations should be noted and justified technically.

(h) Sixty days prior to the decontamination of the channel head, FPL should meet the following conditions (3.2.5):

(i) A system should be set up so that the pressure in the inflatable plug seal in the RCS pipe nozzles should be monitored. Upon loss of seal pressure, injection of the grit slurry should be stopped immediately and the seal plug replaced.

*Branch Technical Position MTEB 5-3 describes the acceptable means for monitoring secondary side water chemistry in PWR steam generators, including corrective actions for off-control point chemistry conditions. However, the staff is amenable to alternatives, particularly to Branch Technical Position B.3.b(9) of MTEB 5-3 (96-hour time limit to repair or plug confirmed condenser tube leaks).

(ii) Written procedures should be provided to include accountability controls of all tools, equipment, materials, and supplies that are to be used in the channel heads to prevent inadvertent entry of such items into the reactor primary coolant system. These controls should be in effect whenever the inflatable plug seals and their associated cover plates are not in place in the nozzles of the reactor coolant system piping.

(iii) Written procedures should be provided to restrict materials to be used in the channel head area to prevent the presence of materials having potential adverse effects on the reactor coolant system components (for example, chloride-bearing materials).

(iv) Written procedures should be provided to include instructions to require that the channel head area, including the nozzles, be inspected and confirmed to be free of all loose materials, equipment, and tools prior to removing the cover plate from the inflatable plug seal.

(v) Prior to closing up the reactor coolant system and starting the RCS pumps, any loose debris, including the abrasive grits, in the channel head, RCS hot leg, and cold leg should be cleaned up.

(vi) Prior to resumption of power operation, the licensee should submit for NRC review and acceptance a report which will include an analysis of the possible effects of any foreign material which has entered the primary coolant system and has not been retrieved. The report should include all work on the decontamination and steam generator repair.

(i) Sixty days prior to the movement of the used steam generator lower assemblies from the containment, the procedures for the move, associated QA requirements, and a description of the equipment to be used shall be provided to the NRC (3.2.6).

(j) Before storage or shipment of the used steam generator lower assemblies, the seal welds must be coated with a heavy bodied varnish such as glyptol (3.2.6).

(k) If credit for the unplugged configuration of the repaired steam generators is to be taken, a new ECCS analysis using the approved model will be required (3.3.1).

4. This license is effective as of the date of issuance, and shall expire at midnight April 27, 2007.

FOR THE ATOMIC ENERGY COMMISSION

Original Signed By

A. Giambusso, Deputy Director
for Reactor Projects
Directorate of Licensing

Attachments:
Appendix A - Technical Specifications
Appendix B - Environmental Technical Specifications

Date of Issuance: April 10, 1973

7590-01

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment Nos. 69 and 61 to Facility Operating License Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company for operation of the Turkey Point Plant Unit Nos. 3 and 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments approve the steam generator repair program for the Turkey Point Plant Unit Nos. 3 and 4 and provide license conditions related to the repair operation.

The amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Notice of Proposed Issuance of Amendments to Facility Operating Licenses in Connection with this action was published in the FEDERAL REGISTER on December 13, 1977 (42 FR 62569). On August 3, 1979, Mr. Mark P. Oncavage was granted status as an intervenor.

On May 28, 1981, the Atomic Safety and Licensing Board (ASLB) issued a Memorandum and Order granting summary disposition on all contentions and cancelling the evidentiary hearing. On June 19, 1981 the ASLB issued its Final Order which authorized the Director of Nuclear Reactor Regulation to issue appropriate license amendments to permit the proposed steam generator repair.

The Commission has issued a Final Environmental Statement on March 30, 1981, which was noticed in the Federal Register on April 3, 1981 (46 FR 20340), and has concluded that the action will not significantly affect the quality of the human environment.

For further details with respect to this action see (1) the Report dated September 20, 1977, as supplemented on December 20, March 7, April 25, June 20, and August 4, 1978, January 26, 1979, and March 28, 1980; (2) Amendment Nos. 69 and 61 to License Nos. DPR-31 and DPR-41; (3) the Commission's related Safety Evaluation (NUREG-0756) dated December 1980; and (4) the Commission's related Final Environmental Statement (NUREG-0743) dated March 1981. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199.

A copy of items (2), (3) and (4) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland this 24th day of June 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

Steven A. Varga, Chief
Operating Reactors Branch No. 1
Division of Licensing