Docket File

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Docket Files 50-250 and 50-251
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ACRS (16)
TERA
NSIC
V. Noonan

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-31 and Amendment No. 46 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated February 13, 1980 as supplemented March 5, 1980.

These amendments incorporate the results of a revised ECCS analysis for a steam generator tube plugging level of 22%.

Copies of the Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 54 to DPR-31
2. Amendment No. 46 to DPR-41
3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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SURNAME	MGrotenhuis:jb	CSParrish	ASchwencer	WPGammill	D Moore	
DATE	03/10/80	03/10/80	03/10/80	03/10/80	03/11/80	

#17

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

March 13, 1980

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig, Vice President
Advanced Systems and Technology
Florida Power and Light Company
Post Office Box 529100
Miami, Florida 33152

Dear Dr. Uhrig:

The Commission has issued the enclosed Amendment No. 54 to Facility Operating License No. DPR-31 and Amendment No. 46 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Unit Nos. 3 and 4, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated February 13, 1980 as supplemented March 5, 1980.

These amendments incorporate the results of a revised ECCS analysis for a steam generator tube plugging level of 22%.

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Sincerely,

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:
1. Amendment No. 54 to DPR-31
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3. Safety Evaluation
4. Notice of Issuance

cc: w/enclosures
See next page

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County Manager of Metropolitan
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Miami, Florida 33130

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 54
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated February 13, 1980 as supplemented March 5, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-31 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 54 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1980

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 46
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Florida Power and Light Company (the licensee) dated February 13, 1980 as supplemented March 5, 1980 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of Facility Operating License No. DPR-41 is hereby amended to read as follows:

   (B) Technical Specifications

   The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: March 13, 1980

ATTACHMENT TO LICENSE AMENDMENTS

AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-31

AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NOS. 50-250 AND 50-251

Revise Appendix A as follows:

Remove Pages	Insert Pages
3.2-3	3.2-3
3.2-4	3.2-4
Figure 3.2-3a	Figure 3.2-3a
Figure 3.2-3b	Figure 3.2-3b

reactivity insertion upon ejection greater than 0.3% k/k at rated power. Inoperable rod worth shall be determined within 4 weeks.

b. A control rod shall be considered inoperable if
   (a) the rod cannot be moved by the CRDM, or
   (b) the rod is misaligned from its bank by more than 15 inches, or
   (c) the rod drop time is not met.

c. If a control rod cannot be moved by the drive mechanism, shutdown margin shall be increased by boron addition to compensate for the withdrawn worth of the inoperable rod.

CONTROL ROD POSITION INDICATION

If either the power range channel deviation alarm or the rod deviation monitor alarm are not operable rod positions shall be logged once per shift and after a load change greater than 10% of rated power. If both alarms are inoperable for two hours or more, the nuclear overpower trip shall be reset to 93% of rated power.

POWER DISTRIBUTION LIMITS

a. Hot channel factors:

(1) With steam generator tube plugging >22% and ≤25%, the hot channel factors (defined in the basis) must meet the following limits at all times except during low power physics tests:

$$F_q\ (Z) \leq (\ *\ /P) \times K(Z),\ \text{for } P > .5$$

$$F_q\ (Z) \leq (\ *\ ) \times K(Z),\ \text{for } P \leq .5$$

$$F^N_{\Delta H} \leq 1.55\ [1.+0.2\ (1-P)]$$

Where P is the fraction of rated power at which the core is operating; K(Z) is the function given in Figure 3.2-3b; Z is the core height location of F_q.

If F_q, as predicted by approved physics calculations, exceeds (*), the power will be limited to the rated power multiplied by the ratio of (*) divided by the predicted F_q, or augmented surveillance of hot channel factors shall be implemented.

(2) With steam generator tube plugging ≤ 22%, the hot channel factors (defined in the basis) must meet the following limits at all times except during low power physics tests:

$$F_q\ (Z) \leq (1.99/P) \times K(Z),\ \text{for } P > .5$$

$$F_q\ (Z) \leq (3.98) \times K(Z),\ \text{for } P \leq .5$$

$$F^N_{\Delta H} \leq 1.55\ [1.+0.2\ (1-P)]$$

Where P is the fraction of rated power at which the core is operating; K(Z) is the function given in Figure 3.2-3a; Z is the core height location of F_q.

* To be supplied based on results of revised ECCS analysis for 25% steam generator tube plugging. Pending NRC approval of this analysis based on a 25% plugging limit, a 22% tube plugging limit shall be in force.

If F_q, as predicted by approved physics calculations, exceeds 1.99, the power will be limited to the rated power multiplied by the ratio of 1.99 divided by the predicted F_q, or augmented surveillance of hot channel factors shall be implemented.

b. Following initial loading before the reactor is operated above 75% of rated power and at regular effective full rated power monthly intervals thereafter, power distribution maps, using the movable detector system shall be made, to confirm that the hot channel factor limits of the specification are satisfied. For the purpose of this comparison,

(1) The measurement of total peaking factor, F_q^{Meas}, shall be increased by three percent to account for manufacturing tolerances and further increased by five percent to account for measurement error.

(2) The measurement of the enthalpy rise hot channel factor, $F_{\Delta H}^N$, shall be increased by four percent to account for measurement error.

If either measured hot channel factor exceeds its limit specified under Item 6a, the reactor power shall be reduced so as not to exceed a fraction of the rated value equal to the ration of the F_q or $F_{\Delta H}^N$ limit to measured value, whichever is less, and the high neutron flux trip setpoint shall be reduced by the same ratio. If subsequent in-core mapping cannot, within a 24 hour period, demonstrate that the hot channel factors are met, the reactor shall be brought to a hot shutdown condition with return to power authorized only for the purpose of physics testing. The reactor may be returned to higher power levels when measurements indicate that hot channel factors are within limits.

c. The reference equilibrium indicated axial flux difference as a function of power level (called the target flux difference) shall be measured at least once per effective full power quarter. If the axial flux difference has not been measured in the last effective full power month, the target flux difference must be updated monthly by linear interpolation using the most recent measured value and the value predicted for the end of the cycle life.

d. Except during physics tests or during excore calibration procedures and as modified by items 6e through 6g below, the indicated axial flux difference shall be maintained within a $\pm$ 5% band about the target flux difference (this defines the target band on axial flux difference).

e. If the indicated axial flux difference at a power level greater than 90% of rated power deviates

HOT CHANNEL FACTOR
NORMALIZED OPERATING ENVELOPE

(for steam generator tube plugging 22% and F_q=1.99)

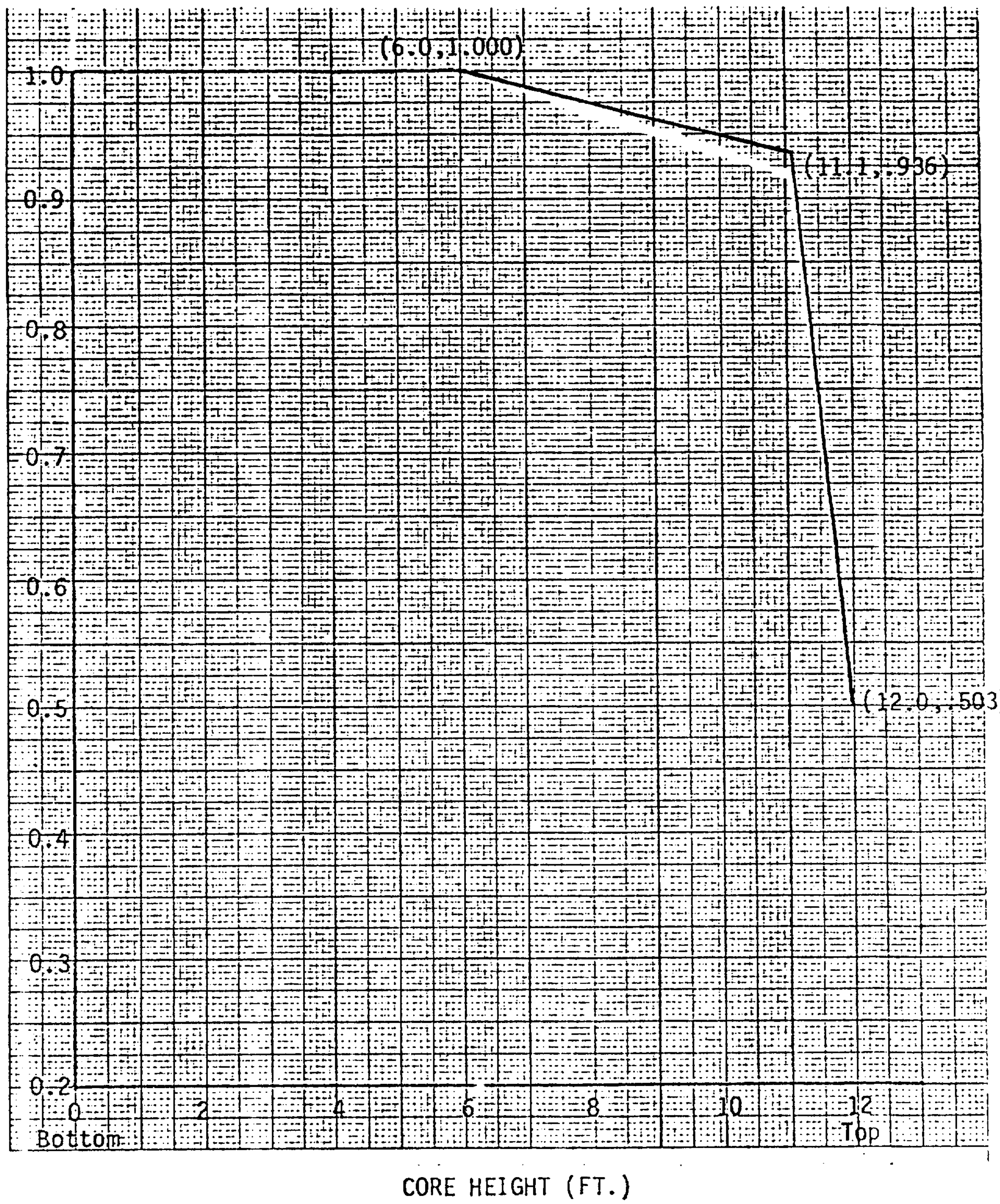
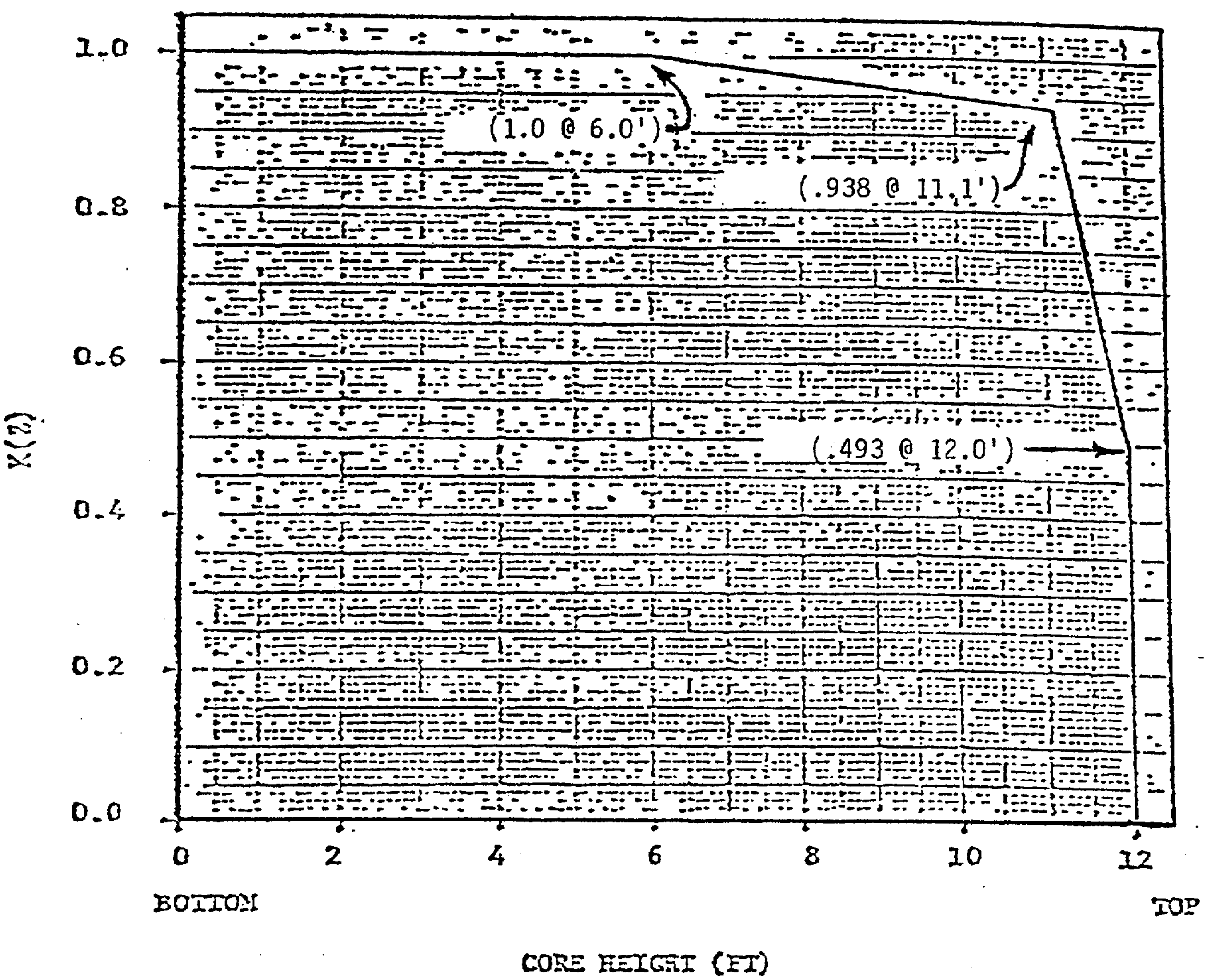


Figure 3.2-3a

Amendment No. 54, Unit 3
Amendment No. 46, Unit 4

HOT CHANNEL FACTOR-NORMALIZED
OPERATING ENVELOPE (FOR STEAM
GENERATOR TUBE PLUGGING ≤25% and F_q = *)



* To be supplied based on results of revised ECCS analysis for 25% steam generator tube plugging.

FIGURE 3.2-3b

Amendment No. 54, Unit 3
Amendment No. 46, Unit 4

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 54 TO FACILITY OPERATING LICENSE NO. DPR-31

AND AMENDMENT NO. 46 TO FACILITY OPERATING LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING, UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

Introduction

By letter dated February 13, 1980 (reference,1 L-51-80), as supplemented March 5, 1980 (reference 2) Florida Power and Light Company (the licensee) requested an amendment to Facility Operating License DPR-31 and DPR-41 for the Turkey Point Plant Unit Nos. 3 and 4. The proposed amendments would incorporate the results of a revised ECCS analysis based on a steam generator plugging limit of 22% into the Technical Specifications for these units.

Background

On November 9, 1979 the licensee was notified by Westinghouse, the NSSS vendor, that an input error had been identified in each of two loss-of-coolant accident (LOCA) analyses specifically applicable to Turkey Point Unit Nos. 3 and 4. The LOCA analyses for a 22% and a 25% steam generator plugging limit were affected. Based on Westinghouse calculations, correction of the error in the 22% tube plugging limit analysis would require a reduction in the maximum allowable F_Q from 2.10 to 1.99. At the time of the notification, less than 22% of the steam generator tubes were plugged. This is still the case. The licensee administratively reduced the F_Q limit to 1.90 on both plants pending NRC review.

The licensee made a prompt telephone notification to IE Region II on November 9, 1979 which was confirmed in writing on November 13, 1979. Licensee Event Report (LER) 250-79-33 was issued on the same subject on November 15, 1979 (reference 9).

On November 23, 1979 (reference 16) LER 250-79-35 was issued stating that Westinghouse had found that a non-conservative feature could exist

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in the 10 CFR 50 Appendix K LOCA analysis with respect to the part of the calculation related to rod burst. Based on revised Westinghouse calculations, the licensee further reduced the F_Q limit from 1.90 to 1.89 on both units by administrative control pending NRC review. Also, augmented surveillance was applied for both units according to the Technical Specifications.

The February 13, 1980 amendment request and its March 5, 1980 supplement included a revised ECCS analysis based on a steam generator tube plugging limit of 22%. The ECCS analysis based on a steam generator tube plugging limit of 25% is currently under review.

In addition, the licensee provided sensitivity study indicating that a rod burst penalty caused by introducing the new fuel performance models developed by the NRC (reference 3) is compensated by the conservatisms existing in the present ECCS models (reference 2) and therefore no reduction of F_Q due to this effect is required.

The changes to the Technical Specifications request by the licensee are the following:

(a) change of F_Q to 1.99 for plant operation with 22% or less of the steam generator tubes plugged.

(b) change of the Hot Channel Factor Normalized Operating Envelope for a steam generator tube plugging limit of 22% (Figure 3.2-3a).

(c) deletion of the Hot Channel Factor Normalized Operating Envelope for steam generator tube plugging levels between 15% and 19% (Figure 3.2-3).

Since the limiting value of F_Q is below the level at which the excore detectors could provide reliable readings and because the "18 case FAC analyses" performed for both units indicated that the maximum predicted F_Q would exceed the LOCA determined limits, the licensee is required either to operate the plant with the augmented power distribution surveillance or at the suitably reduced power levels.

Evaluation

The licensee has provided an evaluation of the performance of Emergency Core Cooling System (ECCS) for both Units 3 and 4 corresponding to the hot channel peaking factor value of F_Q=1.99 and assuming a steam generator plugging limit of 22%, a 3% reduction in thermal design flow and a removal of 65°F fuel temperature conservatism in the PAD fuel performance evaluation code. The reduction of thermal design flow was introduced to compensate for an additional hydraulic resistance caused by the plugged steam generator tubes. It is a conservative assumption. The removal of 65°F fuel temperature conservatism is a non-conservative assumption because in itself it would cause the peak cladding temperature to increase. However, other assumptions existing in the PAD code compensate for it and as a result the fuel performance evaluation by the code is conservative.

The LOCA analysis was performed using the February 1978 version of the Westinghouse Evaluation Model (reference 4) which was reviewed and approved by us. It was performed for a double ended cold leg guillotine break (DECLG) with a discharge coefficient of C_D=0.4. The licensee has shown in the previous submittal (reference 6) that this break size remains unaffected by the number of the steam generator tubes plugged (reference 7).

The previous LOCA analysis for Units 3 and 4 (reference 8) was performed using the same evaluation model and assuming the same steam generator tube plugging limit. However, the value of F_Q was 2.10 for both units. This value was subsequently administratively reduced to 1.90 after an error was discovered in the input to the SATAN computer code, used in LOCA evaluation (reference 9). It was further reduced to 1.89 to account for the changes in the fuel performance models (reference 10).

The currently submitted LOCA analysis includes the input corrections to the SATAN code, but it does not include the changes caused by the modified fuel performance models. The input parameters assumed in the analysis are listed below:

Core Power:	102% of 2200 MWt (rated power)
Peak Linear Power:	102% of 11.31 KW/ft
Peaking Factor:	1.99
Accumulator Water Volume:	875 cu ft/each

The results of the analysis indicate a peak cladding temperature of 2100°F, a maximum local Zr-water reaction of 7.365% and a total Zr-water reaction of less than 0.3%. All these values are below the limits specified in 10 CFR 50.46.

The licensee did not include a small break analysis since steam generator tubes plugged did not affect significantly the results of the original analysis.

The licensee has provided additional calculations (reference 2) to assess the potential impact of the recent concerns related to the fuel performance model changes included in draft report NUREG-0630 (reference 3). Adoption of these changes would produce an increase of the peak cladding temperature by 405°F, due to the fuel burst model change and by 450°F, due to the fuel strain model change. To compensate for these changes and keep the peak cladding temperature below the 2200°F limit, the peaking factor F_Q should by reduced by 0.054. There are, however, two compensating effects which could provide credits offsetting the above mentioned penalties in LOCA analysis. These effects are due to the changes involving the slip and break flow models which have been approved by us for UHI plants after an extensive review. It is estimated that the total benefit of use of these models would be an increase of 0.38 units in F_Q. However, at the present moment, no adequate basis exists for considering horizontal slip. Also an uncertainty exists in translating the phenomena at blowdown to an effect during reflood. It is our current best technical judgment that application of these model changes would result in an increase of F_Q by 0.15 (reference 11). This value more than offsets the penalties in F_Q and the results of the LOCA analysis submitted by the licensee (reference 1) could be considered conservative.

The licensee has performed the "18 case FAC analyses" for Unit 3, Cycle 7 and Unit 4, Cycle 6 (reference 12) because the limiting peaking factor in the LOCA analysis was below the value for which the excore detectors could give reliable measurements. The results of these analyses have indicated that for both units the predicted maximum peaking factor exceed the limiting value of F_Q. The licensee is therefore required either to limit power to the rate power multiplied by the ratio of 1.99 divided by the predicted peaking factor or to implement the augmented surveillance discussed in reference 13 and ascertain that the peaking factor would not exceed th limiting value of 1.99. This requirement could be lifted anytime during plant operation if the licensee demonstrates by the "18 case FAC analysis" that the maximum predicted F_Q is within the LOCA determined limit.

Summary

Based on the review of the submitted documents, we conclude that the results of the LOCA analysis performed with F_Q=1.99 are conservative relative to the 10 CFR 50.46 criteria. We consider the resultant changes to the Technical Specifications acceptable for operating Units 3 and 4 with up to a maximum of 22% of steam generator tubes plugged.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: March 13, 1980

References

1. Letter for R. E. Uhrig (FPL) to D. G. Eisenhut (NRC), Serial No. L-51-80, dated February 13, 1980.

2. Letter from R. E. Uhrig (FPL) to D. G. Eisenhut (NRC), Serial No. L-80-74, dated March 5, 1980.

3. NUREG-0630, Cladding Swelling and Rupture Models for LOCA Analysis, November 1979.

4. WCAP-9220, Westinghouse ECCS Evaluation Model, February 1978 Version, February 1978.

5. Letter from J. F. Stolz (NRC) to T. M. Anderson (Westinghouse), dated August 29, 1978.

6. Letter from R. E. Uhrig (FPL) to V. Stello (NRC), dated December 9, 1976.

7. Letter from R. E. Uhrig (FPL) to G. Lear (NRC), Serial No. L-77-217, dated July 11, 1977.

8. Letters from R. E. Uhrig (FPL) to V. Stello (NRC), Serial Nos. L-79-122, and L-79-124, dated May 18, 1979.

9. Letter from A. D. Schmidt (FPL) to J. P. O'Reilly (NRC-Region II), Serial No. PRN-LI-79-414, dated November 15, 1979.

10. Letter from A. D. Schmidt (FPL) to J. P. O'Reilly (NRC-Region II), Serial No. PRN-LI-79-423, dated November 23, 1979.

11. G. N. Lauben (NRC) to R. P. Denise (NRC) Memorandum, "Review Status of Considered Revisions to Vendor ECCS Evaluation Models," dated December 21, 1979.

12. Letter from R. E. Uhrig (FPL) to D. G. Eisenhut (NRC), Serial No. L-80-68, dated March 3, 1980.

13. Letter from R. E. Uhrig (FPL) to V. Stello (NRC), Serial No. L-78-127, dated April 10, 1978.

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 54 to Facility Operating License No. DPR-31, and Amendment No. 46 to Facility Operating License No. DPR-41 issued to Florida Power and Light Company (the licensee), which revised Technical Specifications for operation of Turkey Point Nuclear Generating, Unit Nos. 3 and 4 (the facilities) located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments incorporate the results of a revised ECCS analysis for a steam generator tube plugging level of 22%.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the application for amendments dated February 13, 1980 as supplemented March 5, 1980, (2) Amendment Nos. 54 and 46 to License Nos. DPR-31 and DPR-41, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental and Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 13th day of March, 1980.

FOR THE NUCLEAR REGULATORY COMMISSION

A. Schwencer

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors