

FEBRUARY 15 1979

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Docket 50-250/251
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CHebron

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ACRS (16)
OPA (CMiles)

Docket Nos. 50-250
and 50-251

Dr. Robert E. Uhrig,
Vice President
Florida Power and Light Company
Advanced Systems and Technology
P. O. Box 529100
Miami, Florida 33152

Gentlemen:

The Commission has issued the enclosed Amendment No. 43 to Facility Operating License No. DPR-31 and Amendment No. 35 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Units Nos. 3 and 4. The amendments consist of changes to the Technical Specifications in response to your applications dated October 21, 1976 and July 5, July 11, July 26, and November 1, 1977.

The amendments consist of miscellaneous changes to the Technical Specifications that (1) specify the qualification requirements for Health Physic Supervisors; (2) delete the existing requirements for an annual operating report; (3) reflect additional batteries and battery chargers; (4) update the use of respiratory protective equipment; (5) extend the time period for approval of temporary changes to procedures and (6) correct administrative errors in previous amendments.

During our review of your proposals for these amendments, we found that certain changes were necessary to meet our requirements. Your staff agreed to these changes and they have been incorporated in these amendments.

Copies of our Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

Original Signed By

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

7903160289

S OELD
GROBERG
2/15/79

THIS ACTION COMPLETES

TACS No. 8651, 8649, 8790, 8791

Enclosures:

- 1. Amendment No. 43 to DPR-31
- 2. Amendment No. 35 to DPR-41

| | | | | | |
|---------|---|-----------------|--------------|-----------|-------------|
| OFFICE | 1. Safety Evaluation 4. Notice of Issuance | DOR:ORB#1/PM | DOR:ORB#1/LA | DOR:EEM/C | DPR:ORB#1/C |
| SURNAME | cc: See next page | MGrotenhuis:pab | CParrish:cp | GKnighton | ASchwencer |
| DATE | | 1/179 | 2/5/79 | 1/179 | 2/15/79 |

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| Local PDR | A. Rosenthal |
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| VStello | G. Knighton |
| DEisenhut | |
| CParrish | |
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| OELD | |
| I&E (5) | |
| B. Jones (8) | |
| B. Scharft (15) | |
| D. Brinkman | |
| C. Hebron | |
| B. Harless | |
| ACRS (16) | |
| OPA (Clare Miles) | |
| R. Ballard | |

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Sincerely,

A. Schwencer, Chief
Operating Reactors Branch #1
Division of Operating Reactors

Enclosures:

1 Amendment No. to DPR-31

2 Amendment No. to DPR-41

3 Safety Evaluation

4 Notice of Issuance

5 See next page

| | | | |
|----------------|--------------|-----------|-------------|
| DOR:ORB#1/PM | DOR:ORB#1/LA | DOR:EEM/C | DOR:ORB#1/C |
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| 12/ /78 | 12/ /78 | 12/ 78 | 12/ /78 |

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 M. Grotenhuis
 OELD
 I&E (5)
 B. Jones (8)
 B. Scharf (10) (15)
 J. M. McGough
 C. Hebron
 B. Harless
 ACRS (16)

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 and 50-251

Florida Power and Light Company
 Advanced Systems and Technology
 ATTN: Dr. Robert E. Uhrig
 Vice President
 P. O. Box 529100
 Miami, Florida 33152

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The amendments consist of miscellaneous changes to the Technical Specifications that (1) specify the qualification requirements for Health Physic Supervisors; (2) delete the existing requirement for an annual operating report; (3) reflect additional batteries and battery chargers; (4) update the use of respiratory protective equipment; (5) reflect minor changes in the offsite organization; and (6) extend the time period for approval of temporary changes to procedures.

During our review of your proposals for these amendments, we found that certain changes were necessary to meet our requirements. Your staff agreed to these changes and they have been incorporated in these amendments.

Copies of our Safety Evaluation and the Notice of Issuance are also enclosed.

Sincerely,

A. Schwencer, Chief
 Operating Reactors Branch #1
 Division of Operating Reactors

OELD

Enclosures:

- 1. Amendment No. to DPR-31
- 2. Amendment No. to DPR 41 / 1978

| | | | | | | |
|---------|-----------------------|-----------------|--------------|---------|-----------|-------------|
| OFFICE | 3. Safety Evaluation | DOR:ORB#1/PM | DOR:ORB#1:LA | DORRS | DOR:EEB | DOR:ORB#1/C |
| SURNAME | 4. Notice of Issuance | MGrotenhuis;pab | CParrish;pab | Wnurses | Gknighton | ASchwencer |
| DATE | CC: See next page | 11/1/78 | 11/ /78 | 11/ /78 | 11/ /78 | 11/ /78 |

Docket Nos. 50-250
and 50-251

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ATTN: Dr. Robert E. Uhrig
Vice President
P. O. Box 529100
Miami, Florida 33152

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The amendments consist of miscellaneous changes to the Technical Specifications that (1) define the use and surveillance of certain motor operated valves; (2) specify the qualification requirements for Health Physic Supervisors; (3) delete the existing requirement for an annual operating report; (4) reflect additional batteries and battery chargers; (5) update the use of respiratory protective equipment; (6) reflect minor changes in the offsite organization; and (7) extend the time period for approval of temporary changes to procedures.

During our review of your proposals for these amendments, we found that certain changes were necessary to meet our requirements. Your staff agreed to these changes and they have been incorporated in these amendments.

Copies of our Safety Evaluation and the related Federal Register Notice are also enclosed.

Sincerely,

A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Enclosures and cc:
See next page

| | | | | | | |
|-----------|-------------|------------|----------|-------------|------------|----------|
| OFFICE > | DOR:ORB1 | DOR:ORB1 | OELD | EEB | DOR:ORB1 | DOR:ORB1 |
| SURNAME > | MGrotenhuis | ASchwencer | GOLDBERG | G. KNIGHTON | K. Parnish | V. Hines |
| DATE > | 08/ /78 | 08/ /78 | 08/ /78 | 9/14/78 | | |

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 B. Jones (4)
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The amendments consist of miscellaneous changes to the Technical Specifications that (1) specify the qualification requirements for Health Physic Supervisors; (2) delete the existing requirement for an annual operating report; (3) reflect additional batteries and battery chargers; (4) update the use of respiratory protective equipment; (5) reflect minor changes in the offsite organization; and (6) extend the time period for approval of temporary changes to procedures.

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A. Schwencer, Chief
 Operating Reactors Branch #1
 Division of Operating Reactors

Enclosures:

- 1. Amendment No. to DPR-31
- 2. Amendment No. to DPR-41

OELD

| | | | | | | | |
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| OFFICE | 3 | Safety Evaluation | DOR:ORB#1/PM | DOR:ORB#1:LA | DORBS | DOR:EEB | DOR:ORB#1/C |
| BURNAME | 4 | Notice of Issuance | CParrish:pab | WVilfers | GKnighton | ASchwencer | |
| CC: | | See next page | MGrotenhuis:pab | | | | |
| DATE | | | 11/1/78 | 11/1/78 | 11/1/78 | 11/1/78 | 11/1/78 |

Florida Power & Light Company

- 2 -

February 15, 1979

cc: Mr. Robert Lowenstein, Esquire
Lowenstein, Newman, Reis & Axelrad
1025 Connecticut Avenue, NW
Suite 1214
Washington, D.C. 20036

Environmental & Urban Affairs Library
Florida International University
Miami, Florida 33199

Mr. Norman A. Coll, Esquire
Steel, Hector and Davis
1400 Southeast First National
Bank Building
Miami, Florida 33131

Florida Power & Light Company
ATTN: Mr. Henry Yaeger
Plant Manager
Turkey Point Plant
P. O. Box 013100
Miami, Florida 33101

Honorable Dewey Knight
County Manager of Metropolitan
Dade County
Miami, Florida 33130

Bureau of Intergovernmental
Relations
660 Apalachee Parkway
Tallahassee, Florida 32304

Director, Technical Assessment Division
Office of Radiation Programs (AW-459)
US EPA
Crystal Mall #2
Arlington, Virginia 20460

U.S. Environmental Protection Agency
Region IV Office
ATTN: EIS COORDINATOR
345 Courtland Street, NW
Atlanta, Georgia 30308



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 43
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Florida Power and Light Company (the licensee) dated October 21, 1976, and July 5, 11 and 26, and November 1, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of the Facility Operating License No. DPR-31 is hereby amended to read as follows:

7903160292

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 43, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 15, 1979



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY
DOCKET NO. 50-251
TURKEY POINT NUCLEAR GENERATING UNIT NO. 4
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 35
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The applications for amendment by Florida Power and Light Company (the licensee) dated October 21, 1976, and July 5, 11 and 26, and November 1, 1977, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B of the Facility Operating License No. DPR-41 is hereby amended to read as follows:

(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 35, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 15, 1979

ATTACHMENT TO LICENSE AMENDMENT NOS. 43 AND 35

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NOS. DPR-31 AND DPR-41

DOCKET NOS. 50-250 AND 50-251

Replace the following pages of the Appendix "A" and "B" Technical Specifications with the enclosed pages. The revised page is identified by Amendment number and contains vertical lines indicating the area of change.

| <u>Appendix A</u> <u>Remove</u> | <u>Replace</u> |
|------------------------------------|----------------|
| iii | iii |
| iv | iv |
| B3.2-6 | B3.2-6 |
| 3.7-1 | 3.7-1 |
| B3.7-1 | B3.7-1 |
| 6-5 | 6-5 |
| 6-15 | 6-15 |
| 6-16 | 6-16 |
| 6-17 | 6-17 |
| 6-18 | 6-18 |
| 6-22 | 6-22 |
| 6-29 | 6-29 |
| 6-30 | - |
| 6-31 | - |
| 6-32 | - |
| 6-33 | * |
| 6-34 (Table 6.12-1) | - |
| 6-35 (Table 6.12-1) | - |
| 6-36 (Table 6.12-1) | - |
| 6-37 (Table 6.12-1) | - |
| <u>Appendix B</u> | |
| 1.1 | 1.1 |
| 4.3 | 4.3 |

*Retain 6.13 from p. 6-33, renumber to 6.12 and relocate to p. 6-29.

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Amendment No. 43 Unit No. 3
Amendment No. 35 Unit No. 4

Flux Difference ($\Delta\phi$) and a reference value which corresponds to the full design power equilibrium value of Axial Offset (Axial Offset = $\Delta\phi$ /fractional power). The reference value of flux difference varies with power level and burnup but expressed as axial offset it varies only with burnup.

The technical specifications on power distribution control assure that the F_q upper bound envelope as defined by Figure 3.2-3 is not exceeded and xenon distributions are not developed which at a later time, would cause greater local power peaking even though the flux difference is then within the limits specified by the procedure.

The target (or reference) value of flux difference is determined as follows. At any time that equilibrium xenon conditions have been established, the indicated flux difference is noted with part length⁺ rods withdrawn from the core and with the full length rod control rod bank more than 190 steps withdrawn (i.e., normal rated power operating position appropriate for the time in life. Control rods are usually withdrawn farther as burnup proceeds). This value, divided by the fraction of design power at which the core was operating is the design power value of the target flux difference. Values for all other core power levels are obtained by multiplying the design power value by the fractional power. Since the indicated equilibrium value was noted, no allowances for excore detector error are necessary and indicated deviation of $\pm 5\%$ ΔI are permitted from the indicated reference value. During periods where extensive load following is required, it may be impractical to establish the required core conditions for measuring the target flux difference every rated power month. For this reason, methods are permitted by Item 6c of Section 3.2 for updating the target flux differences. Figure B3.2-1 shows a typical construction of the target flux difference band at BOL and Figure B3.2-2 shows the typical variation of the full power value with burnup.

Strict control of the flux difference (and rod position) is not as necessary during part power operation. This is because xenon distribution control at part power is not as significant as the control at full power and allowance has been made in predicting the heat flux peaking factors for less strict control at part power. Strict control of the flux difference is not possible during certain physics tests or during the required, periodic excore calibra-

⁺Any reference to part-length rods no longer applies after the part-length rods are removed from the reactor.

3.7 ELECTRICAL SYSTEMS

Applicability: Applies to the availability of electrical power for the operation of auxiliaries.

Objective: To define those conditions of electrical power availability necessary (1) to provide for safe reactor operation, and (2) to provide for the continuing availability of engineered safety features.

- Specification:
1. Either reactor shall not be started from a cold shutdown without:
 - a. The associated 239 KV-4160 volt start-up transformer in service.
 - b. 4160-volt busses A and B of the associated unit, and either bus A or B of the second unit, energized.
 - c. THREE out of FOUR 480-volt load centers and 480-volt motor control centers A, B or C, and D of the associated unit energized.
 - d. TWO diesel generators operable with on site supply of 40,000 gallons of fuel available.
 - e. Four batteries and associated DC systems are operable with FOUR out of SIX battery chargers operable.

 2. During power operation or restarting from hot shutdown the following components may be inoperable:
 - a. ONE start-up transformer may be out of service provided both diesel generators are operable. The AEC shall be notified within 24 hours and be advised of plans to restore the transformer to service.

B3.7 BASES FOR LIMITING CONDITION FOR OPERATION, ELECTRICAL SYSTEMS

The electrical system equipment is arranged so that no single contingency can inactivate enough safety features to jeopardize unit safety. The 480-volt equipment is supplied from 4 load centers and the 4160-volt equipment is supplied from 2 busses for each nuclear unit.

Multiple outside sources supply power to the nuclear units. The auxiliary equipment is arranged electrically so that multiple items receive their power from the two different sources.

-One outside source of power is required to give sufficient power to run normal operating equipment. One transmission line can supply all the auxiliary power. One 239-4.16 kv start up transformer can supply the auxiliary loads for its associated nuclear unit and emergency loads (MHA) for the other nuclear unit.

The bus arrangements specified for operation⁽¹⁾ ensure that power is available to an adequate number of safeguards auxiliaries. With additional switching, more equipment could be out of service without infringing on safety.

Each diesel generator has sufficient capacity to start and run the required engineered safeguards for a MHA in one unit and safe shutdown of the second unit. The minimum diesel fuel oil inventory at all times is maintained to assure the operation of either diesel carrying 168 hour rated load for seven days.

With 4 battery chargers in service, the batteries will always be at full charge in anticipation of a loss-of-ac power incident. This ensures that adequate dc power will be available for emergency use.⁽²⁾

A unit can be safely shutdown without the use of offsite power since all vital loads (safety systems, instruments etc.) can be supplied from an emergency diesel generator.

Reference:

(1) FSAR, Section 8.4

(2) FSAR, Section 8.2

B3.7-1

Amendment No. 43 Unit No. 3
Amendment No. 35 Unit No. 4

- d. An individual qualified in radiation protection procedures shall be on site when fuel is in the reactor.
- e. ALL CORE ALTERATIONS shall be directly supervised by either a licensed Senior Reactor Operator or Senior Reactor Operator Limited to Fuel Handling who has no other concurrent responsibilities during this operation.

6.3 FACILITY STAFF QUALIFICATIONS

- 6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions.

6.3.2 HEALTH PHYSICS SUPERVISOR QUALIFICATIONS

- 6.3.2.1 The Health Physics Supervisor at the time of appointment to the position, shall, except as indicated below, meet the following:

1. He shall have a bachelor's degree or equivalent in a science or engineering subject, including some formal training in radiation protection.
2. He shall have five years of professional experience in applied radiation protection; where a master's degree in a related field is equivalent to one year experience and a doctor's degree in a related field is equivalent to two years of experience.
3. Of his five years of experience, three years shall be in applied radiation protection work in a nuclear facility dealing with radiological problems similar to those encountered at Turkey Point Plant.

- 6.3.2.2 When the Health Physics Supervisor does not meet the above requirements, compensatory action shall be taken which the plant Nuclear Safety Committee determines and the NRC Office of Nuclear Reactor Regulation concurs that the action meets the intent of Specification 6.3.2.1.

6.4 TRAINING

- 6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Training Supervisor and shall meet or exceed the requirements and recommendations of Section 5.5, ANSI N18.1-1971 and Appendix A to 10 CFR Part 55.

6.5 REVIEW AND AUDIT

- 6.5.1 Plant Nuclear Safety Committee PNSC.

6.5.1.1 FUNCTION

The PNSC shall function to advise the Plant Superintendent - Nuclear on all matters related to nuclear safety.

Amendment No.43 Unit No. 3
Amendment No.35 Unit No. 4

6.8.2 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed by the PNSC and approved by the Nuclear Plant Superintendent prior to implementation and periodically as provided by procedure.

6.8.3 Temporary changes to procedures of 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered.
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator License on the unit affected.
- c. The change is documented, reviewed by the PNSC and approved by the Plant Superintendent - Nuclear within fourteen days of implementation.

REPORTING REQUIREMENTS

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1. ROUTINE REPORTS

- a. Startup Report A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

- b. A tabulation of occupational exposure data shall be submitted annually.

A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, ^{1/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

1. This tabulation supplements the requirements of §20.407 of 10 CFR Part

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- c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Management Information and Program Control, U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, with a copy to the appropriate regional offices, to arrive no later than the fifteenth of each month following the calendar month covered by the report.

6.9.2

REPORTABLE OCCURRENCES

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

6.9.3 SPECIAL REPORTS

Special reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification where appropriate.

Twenty copies of the following reports should be sent to the Director, Nuclear Reactor Regulation.

- a. In-service inspection, reference 4.2
- b. Tendon surveillance, reference 4.4
- c. Fire protection systems, reference 3.14.

6.9.4 UNIQUE REPORTING REQUIREMENTS

a. Radioactive Effluent Releases

A report of the quantities of radioactive effluents released from the plant, with data summarized on a monthly basis following the format of U. S. NRC Regulatory Guide 21.

The report shall be submitted within 60 days after January 1 and 60 days after July 1 specifying quantities of radioactive effluents released during the previous 6 months of operation.

1. Gaseous Releases

- (a) Total radioactivity (in curies) releases of noble and activation gases.
- (b) Maximum noble gas release rate during any one-hour period.
- (c) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.
- (d) Percent of technical specification limit.

2. Iodine Releases

- (a) Total (I-131, I-133, I-135) radioactivity (in curies) released.
- (b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

Amendment No. 43 Unit 3
Amendment No. 35 Unit 4

- i. Records of Quality Assurance activities as required by Corporate Quality Assurance Manual.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PNSC and the CNRB.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

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6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20:

- a. Each High Radiation Area in which the intensity of radiation is greater than 100 mRem/hr but less than 1000 mRem/hr shall be barricaded and conspicuously posted as a High Radiation Area and entrance thereto shall be controlled by issuance of a Radiation Work Permit and any individual or group of individuals permitted to enter such areas shall be provided with a radiation monitoring device which continuously indicates the radiation dose rate in the area.
- b. Each High Radiation Area in which the intensity of radiation is greater than 1000 mRem/hr shall be subject to the provisions of 6.13.1(a) above, and in addition locked doors shall be provided to prevent unauthorized entry into such areas and the keys shall be maintained under administrative control.

1.0 DEFINITIONS

The definitions for terms used in these environmental technical specifications are listed below.

- 1.1 "National power emergency" shall mean any event causing authorized Federal officials to require or request that Florida Power & Light supply electricity to points within or without the State of Florida.
- 1.2 "A regional emergency" shall mean any of the following occurrences within the State of Florida: (1) a catastrophic natural disaster including hurricanes, floods, and tidal waves; or (2) other emergencies declared by State, county, municipal, or Federal authorities during which an uninterrupted supply of electric power is vital to public health and safety.
- 1.3 "Reactor emergency" shall mean an unanticipated equipment malfunction necessitating prompt remedial action to avoid endangering the public health or safety.
- 1.4 "Cooling system" and "condenser cooling water system" shall include any and all waterways, lakes, ponds, canals, dikes, levees, dams, barriers, or other structures, devices, or appurtenant facilities which shall be constructed and employed to reduce the temperature of water discharged from Florida Power & Light's generating facilities at Turkey Point.
- 1.5 "Licensed facilities" shall mean Turkey Point Units No. 3 and 4.
- 1.6 Frequency Definitions:
 - Daily - once each 24 hours \pm 12 hours (not less than 360 times per annum).
 - Weekly - once each 7 days \pm 3 days (not less than 48 times per annum).
 - Monthly - once each 30 days \pm 10 days (not less than 12 times per annum).
 - Quarterly - once each 90 days \pm 30 days (not less than 4 times per annum).
 - Semi-Annually - once each 182 days \pm 60 days (not less than 2 times per annum).
 - Refueling - at refueling intervals, but not less than once every 24 months, whichever occurs first.

Bases

Since the cooling canal system is operating in a closed mode, with attendant stresses to marine organisms (heat, pressure changes, turbulence, etc.) and no means of external biological recruitment, monitoring the system will determine its biological stability. The successional stage of this system can be compared to the control areas.

4.1.1.2 Groundwater Program

Objective

The purpose of this program is to evaluate the extent of salt water intrusion between the cooling canal system and the groundwater west of the canal system.

Specification

This program shall involve monitoring of wells and surface points for temperature, water level and conductivity (salinity). The South Florida Water Management District (SFWMD) and the U.S.G.S. shall determine the adequacy of the schedule and the continued need for this monitoring program.

Reporting Requirements

Summaries of the reports prepared above shall be submitted as part of the Annual Environmental Monitoring Report (Section 5.4.1).

Bases

The long-term effects of operating a salt water cooling system on the adjacent groundwater is useful. Monitoring the extent of salt water intrusion will provide data on this interaction.

4.2 Terrestrial Environment

4.2.1 Revegetation of the Cooling Canal Banks

Objectives

The purpose of this study is to assess the floristic species that colonize the mud spoil banks and their growth rates, created by constructing the cooling canals.

Specification

- 4.2.1.1 This program shall analyze soils of the berms for pH, chloride content and selected nutrients. Soil samples shall be taken at points (1) just above the canal water level, (2) half-way-between the water and the top of the soil bank, and (3) from



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENTS 43 AND 35 TO LICENSE NOS. DPR-31 AND DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING UNIT NOS. 3 AND 4

DOCKET NOS. 50-250 AND 50-251

By applications dated October 21, 1976, and July 5, 11 and 26, and November 1, 1977 Florida Power and Light Company (FPL) requested amendments to Operating Licenses DPR-31 and DPR-41 for Turkey Point Unit Nos. 3 and 4. The applications are in support of requests that (1) specify the qualification requirements for Health Physics Supervisors, (2) delete the existing requirements for an Annual Operating Report, (3) reflect additional batteries and battery chargers, (4) update the use of respiratory protective equipment, (5) extend the time period for approval of temporary changes to procedures. In addition, we have added a footnote that was inadvertently dropped from page B3.2-6, we have added p. 6-22 which was inadvertently omitted from Amendments 42 and 34, and we have corrected a typographical error on p. 1-1 and 4-3 in Amendments 41 and 33. During our review of the FPL proposals for these amendments, we found that certain changes were necessary to meet our requirements. The FPL staff agreed to these changes and they have been incorporated in these amendments.

Discussion and Evaluation

(1) Qualification requirements for Health Physics Supervisors.

On March 2, 1977 we sent a letter to FPL regarding the requirements for an individual qualified in radiation protection. This letter included the staff position regarding the criteria for "Individuals Qualified in Radiation Protection Procedures". We also noted that the Turkey Point Facility Technical Specifications do not require that the incumbent performing the function of Health Physics Supervisor meet the minimum qualification requirements of Regulatory Guide 1.8, September 1975.

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By letter dated July 11, 1977 FPL responded to our request to revise the Turkey Point, Unit Nos. 3 and 4 Technical Specifications to require the presence on site of an individual qualified in radiation procedures anytime fuel is in either reactor and to establish the minimum qualifications of the Health Physics Supervisor to the requirements of Regulatory Guide 1.8, September 1975. The response proposed a new Section 6.3.2 to the Technical Specifications to specify Health Physics Supervisor qualification requirements as requested by the staff. The changes are additional administrative controls and, with a change to include NRC concurrence on any exceptions, which the FPL staff agreed to, conform to our current guidance, thus are acceptable.

(2) Deletion of the requirement for an Annual Operating Report. -

After two years of experience with the reporting requirements for nuclear power reactors, we reviewed the scope of information licensees have been required to submit in the Licensee Event Report (LER), Annual Operating Report, Monthly Operating Report and the Startup Report. Based on our review of LER's we developed a modified format for the LER to make this document more useful for evaluation purposes. By letters sent in July and August 1977, we informed licensees of the new LER format and requested that they use it.

From our review of all licensee reports we determined that much of the information found in the Annual Operating Reports either is addressed in the LER's or Monthly Operating Reports, both of which are submitted in a more timely manner, or could be included in these reports with only a slight augmentation of the information already supplied. Therefore we concluded that the Annual Operating Report could be deleted as a Technical Specification requirement if certain additional information were to be provided in the Monthly Operating Reports. As a result, in September 1977, we sent letters to licensees informing them that a revised and improved format for Monthly Operating Reports was available and requested that they use it. Licensees were informed that if they agreed to use the revised format they should request deletion of the requirement for an Annual Operating Report except that occupation exposure data must still be submitted. On November 1, 1977 FPL submitted an amendment request which proposed to delete the requirement for an Annual Operating Report in accordance with our request.

The FPL's proposed amendment would delete all but one of the four specified items in the Annual Operating Report, the report which tabulates occupation exposure on an annual basis is needed and therefore, the requirement to submit this information has been retained. We have determined that the failed fuel examination information does not need to be supplied routinely by licensees because this historical data can be obtained in a compiled form from fuel vendors when needed. The information concerning forced reductions in power and outages will be supplied in the revised Monthly Operating Report rather than annually. The licensee has committed to use the revised Monthly Operating Report format beginning with its report for January 1978 as requested. We requested, and the licensee agreed to, use of words consistent with the Standard Technical Specifications in Specification 6.9.1.c. We conclude that all needed information will be provided. Therefore, the proposed deletion of the Annual Operating Report except for occupational exposure data which will continue to be reported annually is acceptable.

(3) Additional batteries and battery chargers.

On July 26, 1977, FPL submitted an amendment request which would provide Technical Specifications for existing batteries and battery chargers, not covered in the existing Technical Specifications, for operation of engineered safety features. The number of batteries and battery chargers delineated in the Technical Specifications would be increased to four batteries per unit and six battery chargers per unit, of which four must be operable. This amendment request would update Specification 3.7 to fully reflect the "as-built" 125 volt DC system.

During a site visit, an NRC inspector noted that the DC power supply as indicated in the Technical Specification (Section 3.7) was not consistent with the DC power supply in the FSAR (Section 8.2) Revision Nos. 32 and 33 dated January 18, 1973 and March 30, 1973 respectively. The Safety Evaluation for Turkey Point Unit Nos. 3 and 4 dated March 15, 1972 (Section 7.4) concluded that the onsite power system (including the DC system) conforms to the General Design Criteria and is acceptable. Our evaluation indicates that the review of the DC power in March 1972 included batteries which were, by oversight, not included in the Technical Specifications. We conclude that the amendment request corrects an inadvertent omission and is acceptable.

(4) Respiratory Protection Equipment

On July 29, 1977 FPL was notified that on November 27, 1976 the Commission had published in the Federal Register an amended Section 20.103 of 10 CFR Part 20 which became effective on December 29, 1976. The letter advised FPL that, pursuant to 10 CFR 20.103(c) and (f), if they desired credit for use of respiratory protective equipment at the Turkey Point Facilities after December 28, 1977, such use must be as stipulated in Regulatory Guide 8.15 rather than in the current Technical Specifications. Based on the revocation provision (Section 6.12.3) of the Technical Specifications for Turkey Point Unit Nos. 3 and 4 and in the absence of prior written objection, per our letter of July 29, 1977, deletion of Section 6.12 of the Technical Specifications for Turkey Point Unit Nos. 3 and 4 is in order. This change is consistent with our amended regulation and we therefore find it acceptable.

(5) Time limit for approval of temporary changes to procedures.

On October 21, 1976, FPL submitted a request for amendment which would increase the time limit for the Plant Nuclear Safety Committee to review and for the Plant Superintendent - Nuclear to approve temporary changes to procedures from 7 days to 14 days. The proposed amendment is based on the Standard Technical Specifications for Westinghouse Pressurized Reactors which includes those of the Turkey Point type. This change will have no deleterious effect on the safe operation of the facilities. We, therefore, find the change acceptable.

(6) Administratives Changes

In addition, we have added a footnote that was inadvertently dropped from page B3.2-6, we have added p.6-22 which was inadvertently omitted from Amendments 42 and 34, and we have corrected a tyrographical error on p.1-1 and 4-3 in Amendments 41 and 43.

Environmental Consideration

We have determined that the amendments do not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendments involve an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of these amendments.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the amendments do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the amendments do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: February 15, 1979

UNITED STATES NUCLEAR REGULATORY COMMISSIONDOCKET NOS. 50-250 AND 50-251FLORIDA POWER AND LIGHT COMPANYNOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 43 and 35 to Facility Operating Licenses Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units Nos. 3 and 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments consist of miscellaneous changes that (1) specify the qualification requirements for Health Physic Supervisors, (2) delete the existing requirement for an Annual Operating Report, (3) reflect additional batteries and battery chargers, (4) update the use of respiratory protective equipment, (5) extends the time period for approval of temporary changes to procedures, and (6) correct administrative errors in previous amendments.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act),

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and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of these amendments.

For further details with respect to this action, see (1) the applications for amendments dated October 21, 1976 and July 5, July 11 and 26, and November 1, 1977. (2) Amendment Nos. 43 and 35 to Licenses Nos. DPR-31 and DPR-41 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D.C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199. A copy of

items (2) and (3) may be obtained upon request addressed to the
U. S. Nuclear Regulatory Commission, Washington, D. C. 20555,
Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 15th day of February, 1979.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in cursive script, appearing to read "A. Schwencer".

A. Schwencer, Chief
Operating Reactors Branch No. 1
Division of Operating Reactors