

Docket Nos. 50-250
and 50-251

Florida Power and Light Company
ATTN: Dr. Robert E. Uhrig
Vice President
P. O. Box 013100
Miami, Florida 33101

Gentlemen:

The Commission has issued the enclosed Amendment No. 13 to Facility Operating License No. DPR-31 and Amendment No. 12 to Facility Operating License No. DPR-41 for the Turkey Point Nuclear Generating Units 3 and 4. These amendments include Change No. 25 to the Technical Specifications, Appendix A, and is in response to your request dated December 20, 1974.

These amendments incorporate into the Turkey Point Nuclear Generating, Units 3 & 4 Technical Specifications changes to the reporting requirements. Changes to your proposal were necessary to meet our requirements. These have been discussed with your staff. The Technical Specifications are based on Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4.

We request that you use the formats presented in the Appendices to Regulatory Guide 1.16, Revision 4, for reporting operating information and that you report events of the type described under the section "Events of Potential Public Interest". Instructions for using these reporting formats are contained in Regulatory Guide 1.16 (a copy is enclosed for your use), and AEC report OOE-SS-001 titled "Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File" of which you were previously provided a copy. This report is modified by updated instructions dated August 21, 1975 which are enclosed. Copy requirements are summarized in Regulatory Guide 10.1, "Compilation of Reporting Requirements for Persons Subject to NRC Regulations", a copy of which is also enclosed. This guide will assist you in identifying reports that are required by the Commission's regulations set forth in Title 10 Code of Federal Regulations but are not contained in your Technical Specifications. Reports that are required by the regulations have not been repeated in your Technical Specifications.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

Const. JPB

OFFICE ➤	RL:ORB#3	RL:ORB#3 <i>DE</i>	RL:ORB#3 <i>DE</i>		
SURNAME ➤	CParrish <i>cp</i>	DElliott:acr	GElear	George Lear, Chief	
DATE ➤	11/24/75	11/25/75	11/25/75	Operating Reactors Branch #3	
				Division of Reactor Licensing	

Enclosures:

1. Amendments Nos. 13 and 12
2. Regulatory Guide 1.16
3. Updated Instructions
4. Regulatory Guide 10.1
5. Safety Evaluation
6. Federal Register Notice

cc w/encls:

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Lowenstein, Newman, Reis & Axelrad
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Suite 1214
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Environmental & Urban Affairs Library
Florida International University
Miami, Florida 33199

Mr. Ed Maroney
Bureau of Intergovernmental Relations
725 South Bronough Street
Tallahassee, Florida 32304

OFFICE➤						
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company (the licensee) dated December 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulation of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility License No. DPR-31 is hereby amended to read as follows:

"(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 25"

3. This license amendment is effective 30 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for Donald M. Ellert
George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 25
Technical Specifications

Date of Issuance: November 26, 1975

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT NO. 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 12
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power & Light Company (the licensee) dated December 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

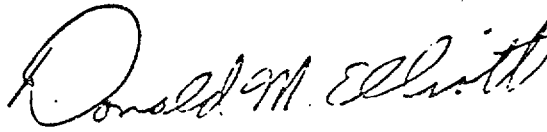
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility License No. DPR-41 is hereby amended to read as follows:

"(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 25"

3. This license amendment is effective 60 days from the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


for George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 25
Technical Specifications

Date of Issuance: November 26, 1975

ATTACHMENT TO LICENSE AMENDMENTS NOS. 13& 12
CHANGE NO. 25 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSE NO. DPR-31 & DPR-41
DOCKETS NOS. 50-250 AND 50-251

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and pages 6-16 through 6-27 with the attached revised
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1.14 (Deleted)

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1.15 POWER TILT

The power tilt is the ratio of the maximum to average of the upper out-of-core normalized detector currents or the lower out-of-core normalized detector currents whichever is greater. If one out-of-core detector is out of service, the remaining three detectors are to be used to compute the average.

than a minority of the quorum shall have line responsibility for operation of the facility.

6.5.2.7 REVIEW

The CNRB shall review:

- a. The safety evaluations for 1) changes to procedures, equipment or systems and, 2) tests or experiments completed under the provision of Section 50.59, 10 CFR, to verify that such actions did not constitute an unreviewed safety question.
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR.
- d. Proposed changes in Technical Specifications or Licenses.
- e. Violations of applicable statutes, codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance.
- f. Significant operating abnormalities or deviations from normal and expected performance of plant equipment that affect nuclear safety
- g. All events which are required by regulations or Technical Specifications to be reported to the NRC in writing within 24 hours.

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6.6	<u>REPORTABLE OCCURRENCE ACTION</u>	25
6.6.1	The following actions shall be taken in the event of a REPORTABLE OCCURRENCE:	25
	a. The REPORTABLE OCCURRENCE shall be reported to the Commission pursuant to the requirements of Section 6.9.	25
	b. A Reportable Occurrence Report shall be prepared. The report shall be reviewed by the Plant Nuclear Safety Committee.	25
	c. The Reportable Occurrence Report shall be submitted to the CNRB, the Vice President of Power Resources, and the Commission within the time allotted in Section 6.9.	25
6.7	<u>SAFETY LIMIT VIOLATION</u>	
6.7.1	The following actions shall be taken in the event a Safety Limit is violated:	
	a. The provisions of 10 CFR 50.36(c)(1)(i) shall be complied with immediately.	
	b. The Safety Limit violation shall be reported immediately to the Commission, the Vice President of Power Resources and to the CNRB.	
	c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PNSC. This report shall describe 1) applicable circumstances preceding the violation, 2) effects of the violation upon facility components, systems or structures, and 3) corrective action taken to prevent recurrence.	
	d. The Safety Limit Violation Report shall be submitted to the CNRB, the Vice President of Power Resources and the Commission within ten (10) days of the violation.	
6.8	<u>PROCEDURES</u>	
6.8.1	Written procedures and administrative policies shall be established, implemented and maintained that meet or exceed the requirements and recommendations of Section 5.1 and 5.3 of ANSI N18.7-1972 and Appendix "A" of USNRC Regulatory Guide 1.33 except as provided in 6.8.2 and 6.8.3 below.	

In addition to the applicable reporting requirements of Title 10, Code of Federal Regulations, the following identified reports shall be submitted to the Director of the appropriate Regional Office of Inspection and Enforcement unless otherwise noted.

6.9.1. ROUTINE REPORTS

- a. Startup Report. A summary report of plant startup and power escalation testing shall be submitted following (1) receipt of an operating license, (2) amendment to the license involving a planned increase in power level, (3) installation of fuel that has a different design or has been manufactured by a different fuel supplier, and (4) modifications that may have significantly altered the nuclear, thermal, or hydraulic performance of the plant. The report shall address each of the tests identified in the FSAR and shall in general include a description of the measured values of the operating conditions or characteristics obtained during the test program and a comparison of these values with design predictions and specifications. Any corrective actions that were required to obtain satisfactory operation shall also be described. Any additional specific details required in license conditions based on other commitments shall be included in this report.

Startup reports shall be submitted within (1) 90 days following completion of the startup test program, (2) 90 days following resumption or commencement of commercial power operation, or (3) 9 months following initial criticality, whichever is earliest. If the Startup Report does not cover all three events (i.e., initial criticality, completion of startup test program, and resumption or commencement of commercial power operation), supplementary reports shall be submitted at least every three months until all three events have been completed.

- b. Annual Operating Report^{1/} Routine operating reports covering the operation of the unit during the previous calendar year should be submitted prior to March 1 of each year. The initial report shall be submitted prior to March 1 of the year following initial criticality.

The annual operating reports made by licensees shall provide a comprehensive summary of the operating experience gained during the year, even though some repetition of previously reported information may be involved. References in the annual operating report to previously submitted reports shall be clear.

^{1/} A single submittal may be made for a multiple unit station. The submittal should combine those sections that are common to all units at the station.

Each annual operating report shall include:

- (1) A narrative summary of operating experience during the report period relating to safe operation of the facility, including safety-related maintenance not covered in item 1.b.(2)(e) below.
- (2) For each outage or forced reduction in power^{2/} of over 20% of design power level where the reduction extends for greater than four hours:
 - (a) the proximate cause and the system and major component involved (if the outage or forced reduction in power involved equipment malfunction);
 - (b) a brief discussion of (or reference to reports of) any reportable occurrences pertaining to the outage or power reduction;
 - (c) corrective action taken to reduce the probability of recurrence, if appropriate;
 - (d) operating time lost as a result of the outage or power reduction (for scheduled or forced outages,^{3/} use the generator off-line hours; for forced reduction in power, use the approximate duration of operation at reduced power);
 - (e) a description of major safety-related corrective maintenance performed during the outage or power reduction, including the system and component involved and identification of the critical path activity dictating the length of the outage or power reduction; and
 - (f) a report of any single release of radioactivity or radiation exposure specifically associated with the outage which accounts for more than 10% of the allowable annual values.

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^{2/} The term "forced reduction in power" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the load on the unit be reduced for corrective action immediately or up to and including the very next weekend. Note that routine preventive maintenance, surveillance and calibration activities requiring power reductions are not covered by this action.

^{3/} The term "forced outage" is normally defined in the electric power industry as the occurrence of a component failure or other condition which requires that the unit be removed from service for corrective action immediately or up to and including the very next weekend.

(3) A tabulation on an annual basis of the number of station, utility and other personnel (including contractors) receiving exposures greater than 100 mrem/yr and their associated man rem exposure according to work and job functions, ^{4/} e.g., reactor operations and surveillance, inservice inspection, routine maintenance, special maintenance (describe maintenance), waste processing and refueling. The dose assignment to various duty functions may be estimates based on pocket dosimeter, TLD, or film badge measurements. Small exposures totalling less than 20% of the individual total dose need not be accounted for. In the aggregate, at least 80% of the total whole body dose received from external sources shall be assigned to specific major work functions.

(4) Indications of failed fuel resulting from irradiated fuel examinations, including eddy current tests, ultrasonic tests, or visual examinations completed during the report period.

c. Monthly Operating Report. Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the appropriate Regional Office, to arrive no later than the tenth of each month following the calendar month covered by the report.

6.9.2 REPORTABLE OCCURRENCES

Reportable occurrences, including corrective actions and measures to prevent reoccurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

4/ This tabulation supplements the requirements of §20.407 of 10 CFR Part 20

- a. Prompt Notification With Written Followup. The types of events listed below shall be reported as expeditiously as possible, but within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the appropriate Regional Office, or his designate no later than the first working day following the event, with a written followup report within two weeks. The written followup report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Failure of the reactor protection system or other systems subject to limiting safety system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety system setting in the technical specifications or failure to complete the required protective function.

Note: Instrument drift discovered as a result of testing need not be reported under this item but may be reportable under items 2.a(5), 2.a(6), or 2.b(1) below.

- (2) Operation of the unit or affected systems when any parameter or operation subject to a limiting condition is less conservative than the least conservative aspect of the limiting condition for operation established in the technical specifications.

Note: If specified action is taken when a system is found to be operating between the most conservative and the least conservative aspects of a limiting condition for operation listed in the technical specifications, the limiting condition for operation is not considered to have been violated and need not be reported under this item, but it may be reportable under item 2.b(2) below.

- (3) Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.

Note: Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

- (4) Reactivity anomalies, involving disagreement with the predicted value of reactivity balance under steady state conditions during power operation, greater than or equal to $1\% \Delta k/k$; a calculated reactivity balance indicating a shut-down margin less conservative than specified in the technical specifications; short-term reactivity increases that correspond to a reactor period of less than 5 seconds or, if sub-critical, an unplanned reactivity insertion of more than $0.5\% \Delta k/k$ or occurrence of any unplanned criticality.
- (5) Failure or malfunction of one or more components which prevents or could prevent, by itself, the fulfillment of the functional requirements of system(s) used to cope with accidents analyzed in the SAR.
- (6) Personnel error or procedural inadequacy which prevents or could prevent, by itself, the fulfillment of the functional requirements of systems required to cope with accidents analyzed in the SAR.

Note: For items 2.a(5) and 2.a(6) reduced redundancy that does not result in a loss of system function need not be reported under this section but may be reportable under items 2.b(2) and 2.b(3) below.

- (7) Conditions arising from natural or man-made events that, as a direct result of the event require plant shutdown, operation of safety systems, or other protective measures required by technical specifications.
- (8) Errors discovered in the transient or accident analyses or in the methods used for such analyses as described in the safety analysis report or in the bases for the technical specifications that have or could have permitted reactor operation in a manner less conservative than assumed in the analyses.
- (9) Performance of structures, systems, or components that requires remedial action or corrective measures to prevent operation in a manner less conservative than assumed in the accident analyses in the safety analysis report or technical specifications bases; or discovery during plant life of conditions not specifically considered in the safety analysis report or technical specifications that require remedial action or corrective measures to prevent the existence or development of an unsafe condition.

Note: This item is intended to provide for reporting of potentially generic problems.

- b. Thirty Day Written Reports. The reportable occurrences discussed below shall be the subject of written reports to the Director of the appropriate Regional Office within thirty days of occurrence of the event. The written report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

- (1) Reactor protection system or engineered safety feature instrument settings which are found to be less conservative than those established by the technical specifications but which do not prevent the fulfillment of the functional requirements of affected systems.
- (2) Conditions leading to operation in a degraded mode permitted by a limiting condition for operation or plant shutdown required by a limiting condition for operation.

Note: Routine surveillance testing, instrument calibration, or preventative maintenance which require system configurations as described in items 2.b(1) and 2.b(2) need not be reported except where test results themselves reveal a degraded mode as described above.

- (3) Observed inadequacies in the implementation of administrative or procedural controls which threaten to cause reduction of degree of redundancy provided in reactor protection systems or engineered safety feature systems.
- (4) Abnormal degradation of systems other than those specified in item 2.a(3) above designed to contain radioactive material resulting from the fission process.

Note: Sealed sources or calibration sources are not included under this item. Leakage of valve packing or gaskets within the limits for identified leakage set forth in technical specifications need not be reported under this item.

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Special reports shall be submitted covering the activities identified below pursuant to the requirements of the applicable reference specification where appropriate.

Twenty copies of the following reports should be sent to the Director, Nuclear Reactor Regulation.

- a. In-service inspection, reference 4.2.
- b. Tendon surveillance, reference 4.4.

UNIQUE REPORTING REQUIREMENTSa. Radioactive Effluent Releases

A report of the quantities of radioactive effluents released from the plant, with data summarized on a monthly basis following the format of U. S. NRC Regulatory Guide 21.

The report shall be submitted within 60 days after January 1 and 60 days after July 1 specifying quantities of radioactive effluents released during the previous 6 months of operation.

1. Gaseous Releases

- (a) Total radioactivity (in curies) releases of noble and activation gases.
- (b) Maximum noble gas release rate during any one-hour period.
- (c) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.
- (d) Percent of technical specification limit.

2. Iodine Releases

- (a) Total (I-131, I-133, I-135) radioactivity (in curies) released.
- (b) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.

(c) Percent of technical specification limit.

3. Particulate Releases

- (a) Gross radioactivity (β , γ) released (in curies) excluding background radioactivity.
- (b) Gross alpha radioactivity released (in curies) excluding background radioactivity.
- (c) Total radioactivity released (in curies) of nuclides with half-lives greater than eight days.
- (d) Percent of technical specification limit.

4. Liquid Releases

- (a) Gross radioactivity (β , γ) released (in curies) excluding tritium and average concentration released to the unrestricted area.
- (b) Total tritium and alpha radioactivity (in curies) released and average concentration released to the unrestricted area.
- (c) Total dissolved gas radioactivity (in curies) and average concentration released to the unrestricted area.
- (d) Total volume (in liters) of liquid waste released.
- (e) Total volume (in liters) of dilution water used prior to release from the restricted area.
- (f) The maximum concentration of gross radioactivity (β , γ) released to the unrestricted area (averaged over the period of release).
- (g) Total radioactivity (in curies) released, by nuclide, based on representative isotopic analyses performed.
- (h) Percent of technical specification limit for total activity released.

5. Solid Waste

- (a) The total amount of solid waste packaged (in cubic feet)

- (b) The total estimated radioactivity (in curies) involved.
- (c) The dates of shipment and disposition (if shipped off-site).

b. Environmental Radiological Monitoring

A report on the environmental monitoring program shall be submitted within 60 days after January 1 and 60 days after July 1 covering the previous 6 months of operation. This report may be combined with the report of the quantities of radioactive effluents.

1. For each medium sampled e.g., air, baybottom, surface water, soil, fish including:
 - (a) Number of sampling locations
 - (b) Total number of samples
 - (c) Number of locations at which levels are found to be significantly above local backgrounds
 - (d) Highest, lowest, and the annual average concentrations or levels of radiation for the sampling point with the highest average and description of the location of that point with respect to the site.
2. If levels of radioactive materials in environmental media indicate the likelihood of public intakes in excess of 3% of those that could result from continuous exposure to the concentration values listed in Appendix B, Table II, Part 20, estimates of the likely resultant exposure to individuals and to population groups, and assumptions upon which estimates are based shall be provided.
3. If statistically significant variations of off-site environmental concentrations with time are observed, correlation of these results with effluent releases shall be provided.

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6.10 RECORD RETENTION

6.10.1 The following records shall be retained for at least five (5) years:

- a. Records and logs of facility operation covering time interval at each power level.
- b. Records and logs of principal maintenance activities, inspections, repair and replacement of principal items of equipment related to nuclear safety.
- c. REPORTABLE OCCURRENCE Reports.
- d. Records of surveillance activities, inspections and calibrations required by these Technical Specifications.
- e. Records of reactor tests and experiments.
- f. Records of changes made to Operating Procedures.
- g. Records of radioactive shipments.
- h. Records of sealed source leak tests and results.

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 13 TO FACILITY LICENSE NO. DPR-31, AND
AMENDMENT NO. 12 TO LICENSE NO. DPR-41
CHANGE NO. 25 TO TECHNICAL SPECIFICATIONS
FLORIDA POWER AND LIGHT COMPANY
TURKEY POINT NUCLEAR GENERATING UNITS 3 AND 4
DOCKET NOS. 50-250 AND 50-251

Introduction

By letter dated December 20, 1974, Florida Power and Light Company (FPL) proposed changes to the Technical Specifications appended to Facility Operating Licenses No. DPR-31 and DPR-41 for the Turkey Point Nuclear Generating Units 3 and 4. The proposed changes involve changes to the reporting requirements.

Discussion

The proposed changes would be administrative in nature and would affect the conduct of operation. The proposed changes are intended to provide uniform license requirements. Areas covered by the proposed uniform specifications include reporting requirements and an abnormal occurrence definition change.

In Section 208 of the Energy Reorganization Act of 1974 "abnormal occurrences" is defined as an unscheduled incident or event which the Commission determines is significant from the standpoint of public health or safety. The term "abnormal occurrence" is reserved for usage by NRC. Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4, enumerates required reports consistent with Section 208. The proposed change to required reports identifies the reports required of all licensees not already identified by the regulations and those unique to this facility. The proposal would formalize present reporting and would delete any reports no longer needed for assessment of safety related activities.

Evaluation

The new guidance for reporting operating information does not identify any event as an "abnormal occurrence". The proposed reporting requirements also delete reporting of information no longer required and duplication of reported information. The standardization of required reports and desired format for the information will permit more rapid recognition of potential problems.

During our review of the proposed changes, we found that certain modifications to the proposal were necessary to have conformance with the desired regulatory position. These changes were discussed with your staff and have been incorporated into the proposal.

We have concluded that the proposal as modified improves the licensee's program for evaluating plant performance and the reporting of the operating information needed by the Commission to assess safety related activities and is acceptable. The modified reporting program is consistent with the guidance provided by Regulatory Guide 1.16, "Reporting of Operating Information - Appendix A Technical Specifications", Revision 4. The administrative controls are consistent with requirements being incorporated in Technical Specifications for new licensed facilities.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: November 26, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION
DOCKETS NOS. 50-250 AND 50-251
FLORIDA POWER AND LIGHT COMPANY
NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

Notice is hereby given that the U.S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 13 and 12 to Facility Operating Licenses Nos. DPR-31 and DPR-41 issued to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units Nos. 3 and 4, located in Dade County, Florida. The amendment is effective 60 days from the date of issuance.

The amendment modifies the reporting requirements of the Technical Specifications for the Turkey Point Nuclear Generating, Units 3 and 4.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

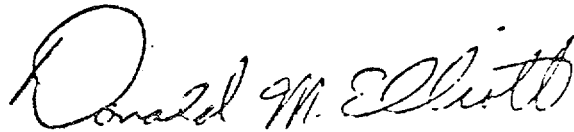
For further details with respect to this action, see (1) the application for amendment dated December 20, 1974, (2) Amendments Nos. 13 and 12 to Licenses Nos. DPR-31 and DPR-41, with Change No. 25 and (3)

the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 26 day of November, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "Donald M. Elliott". The signature is fluid and cursive, with a large initial "D" and "E".

Donald M. Elliott, Acting Chief
Operating Reactors Branch #3
Division of Reactor Licensing