

Dockets Nos. 50-250  
and 50-251

DEC 1 1 1975

Florida Power and Light Company  
ATTN: Dr. Robert E. Uhrig  
Vice President  
P. O. Box 013100  
Miami, Florida 33101

Gentlemen:

The Commission has issued the enclosed Amendment No. 14 to Facility Operating License No. DPR-31 and Amendment No. 13 to Facility Operating License No. DPR-41 for Turkey Point Nuclear Generating Units 3 and 4. These amendments include Change No. 26 to the joint Technical Specifications and are in response to your request dated September 9, 1975, and Supplements dated October 29, November 21, and November 26, 1975.

These amendments modify operating limits in the Technical Specifications to allow operation of Turkey Point Nuclear Generating Unit 3, following refueling, at 2100 psia until the extended fuel residence time is reached. The operating limits for Unit 4 set forth in its Technical Specifications remain unchanged although the Unit 4 Technical Specifications will be modified to reflect the revisions to the Unit 3 Technical Specifications.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

Enclosures:

1. Amendment No. 14
2. Amendment No. 13
3. Safety Evaluation
4. Federal Register Notice

cc: See next page

DISTRIBUTION:

Docket	SKari	State Official
NRC PDR	<del>XXXXXXXX</del>	Local Official
Local PDR	BScharf (15)	EPA
ORB Rdg	TJCarter	
OELD	CHaupt	
DI&E (3)	PColling	
NDube	SVarga	
BJones (w/4 encls)	CHebron	
JMcGough	AESteen	
JSaltzman	DEisenhut	
CParrish	ACRS (16)	
DElliott	TBAbernathy	
GLear	JRBuchanan	
KRGoller	App. Att. of Record	

OFFICE >	ORB#3	ORB#3	OELD	ORB#3	AD:RL/ORS
SURNAME >	CParrish	DElliott:acr		GLear	KRGoller
DATE >	12/	/75 12/	/75 12/	/75 12/	/75 12/

Florida Power & Light Company - -

cc:

Mr. Jack R. Newman, Esquire  
Lowenstein, Newman, Reis & Axelrad  
1025 Connecticut Avenue, N. W.  
Suite 1214  
Washington, D. C. 20036

Environmental & Urban Affairs Library  
Florida International University  
Miami, Florida 33199

Mr. Ed Maroney  
Bureau of Intergovernmental Relations  
725 South Bronough Street  
Tallahassee, Florida 32304

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 14 and No. 13 to Facility Operating Licenses Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units 3 and 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendment modifies the operating limits in the Technical Specifications to allow operation of Turkey Point Nuclear Generating Unit 3, following refueling, at 2100 psia until the extended fuel residence time is reached. The operating limits for Unit 4 set forth in the Technical Specifications remain unchanged although the Unit 4 Technical Specifications will be modified to reflect the revisions to the Unit 3 Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of Proposed Issuance of Amendment to Facility

OFFICE >

--	--	--	--	--	--

Operating License in connection with this action was published in the FEDERAL REGISTER on October 31, 1975 (40 FR 50745). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

For further details with respect to this action, see (1) the application for amendment dated September 9, 1975 and supplements dated October 29, November 21 and November 26, 1975, (2) Amendments Nos. 14 and 13 to Licenses Nos. DPR-31 and DPR-41, with Change No. 26 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 11<sup>th</sup> day of December, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

151

George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

OFFICE >	ORB#3	ORB#3 <i>DE</i>	ORB#3	OELD <i>R. Ross</i>	RL:AD/ORS
SURNAME >	CParrish <i>CP</i>	DElliott:acr	GLear <i>GL</i>	<i>GL</i>	KRGoller <i>KRG</i>
DATE >	12/ 4 /75	12/ 4 /75	12/ 9 /75	12/ 9 /75	12/ 10 /75

For further details with respect to this action, see (1) the application for License Amendments dated December 4, 1975, (2) Amendments Nos. 10 and 8 to Licenses Nos. DPR-44 and DPR-56, and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Martin Memorial Library, 159 E. Market Street, York, Pennsylvania 17401.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 10th day of December, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION

Donald M. Elliott, Acting Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing

OFFICE	ORB#3	<del>ORB#3</del>	OELD	ORB#3 DE		
SURNAME	CPA:msh:mf	<del>CPA:msh:mf</del>	KARIMANIK	Glear		
DATE	12/10/75	12/10/75	12/10/75	12/10/75		

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-277 AND 50-278

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 10 and 8 to Facility Operating Licenses Nos. DPR-44 and DPR-56, respectively, issued to Philadelphia Electric Company, Public Service Electric and Gas Company, Delmarva Power and Light Company, and Atlantic City Electric Company, for operation of the Peach Bottom Atomic Power Station, Units 2 and 3, located in York County, Pennsylvania. The amendments are effective as of the date of issuance.

The amendments extend the date for implementation of License Amendments Nos. 12 and 10, issued on November 10, 1975 and which were to be effective on December 10, 1975, to January 1, 1976.

The application for License Amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 13  
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 9, 1975 and supplements dated October 9, November 21 and November 26, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter 1;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility License No. DPR-41 is hereby amended to read as follows:

"(B) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 26"

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*Karl R. Goller*

Karl R. Goller, Assistant Director  
for Operating Reactors  
Division of Reactor Licensing

Attachment:  
Change No. 26  
Technical Specifications

Date of Issuance: December 11, 1975

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14  
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 9, 1975, and supplements dated October 9, November 21, and November 26, 1975, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility License No. DPR-31 is hereby amended to read as follows:

"(B) Technical Specifications

The Technical Specifications contained in Appendices A and B; as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 26"

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Karl R. Goller, Assistant Director  
for Operating Reactors  
Division of Reactor Licensing

Attachment:  
Change No. 26  
Technical Specifications

Date of Issuance: December 11, 1975

ATTACHMENT TO LICENSE AMENDMENT NOS. 14 & 13  
CHANGE NO. 26 TO THE TECHNICAL SPECIFICATIONS  
FACILITY OPERATING LICENSE NOS. DPR-31 AND DPR-41  
DOCKETS NOS. 50-250 AND 50-251

Replace page i, page v, page 1-6, Figure 2.1-1, page 2.3-2, page 2.3-3, page 3.2-1, Figure 3.2-1, Figure 3.2-1(a) and page B3.2-1 with the attached revised pages. Add page 3.1-7, Figure 3.2-1(b), Figure 3.2-1(c) and page B.3.2-1a.

TABLE OF CONTENTS

<u>Section</u>	<u>Title</u>	<u>Page</u>
<b>TECHNICAL SPECIFICATIONS</b>		
<b>1</b>	<b>DEFINITIONS</b>	<b>1-1</b>
1.1	Safety Limits	1-1
1.2	Limiting Safety System Settings	1-1
1.3	Limiting Conditions for Operation	1-1
1.4	Operable	1-1
1.5	Containment Integrity	1-2
1.6	Protective Instrumentation Logic	1-2
1.7	Instrumentation Surveillance	1-3
1.8	Shutdown	1-3
1.9	Power Operation	1-4
1.10	Refueling Operation	1-4
1.11	Rated Power	1-4
1.12	Thermal Power	1-4
1.13	Design Power	1-4
1.14	(Deleted)	1-5
1.15	Power Tilt	1-5
1.16	Interim Limits	1-6
1.17	Low Power Physics Tests	1-6
<b>2</b>	<b>SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS</b>	<b>2.1-1</b>
2.1	Safety Limit, Reactor Core	2.1-1
2.2	Safety Limit, Reactor Coolant System Pressure	2.2-1
2.3	Limiting Safety System Settings, Protective Instrumentation	2.3-1
<b>3</b>	<b>LIMITING CONDITIONS FOR OPERATION</b>	<b>3.1-1</b>
3.1	Reactor Coolant System	3.1-1
	Operational Components	3.1-1
	Heat Up and Cool Down	3.1-2
	Leakage	3.1-4
	Maximum Reactor Coolant Activity	3.1-5
	Reactor Coolant Chemistry	3.1-6
	DNB Parameters	3.1-7
3.2	Control Rod and Power Distribution Limits	3.2-1
	Control Rod Insertion Limits	3.2-1
	Misaligned Control Rod	3.2-2
	Rod Drop Time	3.2-2
	Inoperable Control Rods	3.2-2
	Control Rod Position Indication	3.2-3
	Power Distribution Limits	3.2-3
	In-Core Instrumentation	3.2-7
	Axial Offset Alarms	3.2-8
3.3	Containment	3.3-1
3.4	Engineered Safety Features	3.4-1
	Safety Injection and RHR Systems	3.4-1
	Emergency Containment Cooling Systems	3.4-3
	Emergency Containment Filtering System	3.4-4
	Component Cooling System	3.4-4
	Intake Cooling Water System	3.4-5
3.5	Instrumentation	3.5-1
3.6	Chemical and Volume Control System	3.6-1

## LIST OF FIGURES

<u>Figure</u>	<u>Title</u>
2.1-1	Reactor Core Thermal and Hydraulic Safety Limits, Three Loop Operation
2.1-2	Reactor Core Thermal and Hydraulic Safety Limits, Two Loop Operation
3.1-1	Reactor Coolant System Pressure Limits
3.1-2	Radiation Induced Increase in Transition Temperature for A302-B Steel
3.2-1	Control Group Insertion Limits for Unit 4, Three Loop Operation
3.2-1a	Control Group Insertion Limits for Unit 4, Two Loop Operation
3.2-1b	Control Group Insertion Limits for Unit 3, Three Loop Operation
3.2-1c	Control Group Insertion Limits for Unit 3, Two Loop Operation
3.2-2	Required Shutdown Margin
3.2-3	Hot Channel Factor Normalized Operating Envelope
3.2-4	Maximum Allowable Local KW/FT
4.12-1	Sampling Locations
6.1-1	Offsite Organization Chart
6.1-2	Plant Organization Chart
B3.2-1	Target Band on Indicated Flux Difference as a Function of Operating Power Level.
B3.2-2	Permissible Operating Band on Indicated Flux Difference as a Function of Burnup.

## 1.16 INTERIM LIMITS

### 1.16.1 Fuel Residence Time Limit

The fuel residence time for Unit 3 shall be limited to 23,500 effective full power hours (EFPH) under reduced pressure operating conditions. The fuel residence time for Unit 4 shall be limited to 30,000 EFPH. b6

### 1.16.2 Reactor Coolant Pumps Operation

The reactor shall not be operated with less than three reactor coolant pumps in operation.

## 1.17 LOW POWER PHYSICS TESTS

Low power physics tests are tests below a nominal 5% of rated power which measure fundamental characteristics of the reactor core and related instrumentation.

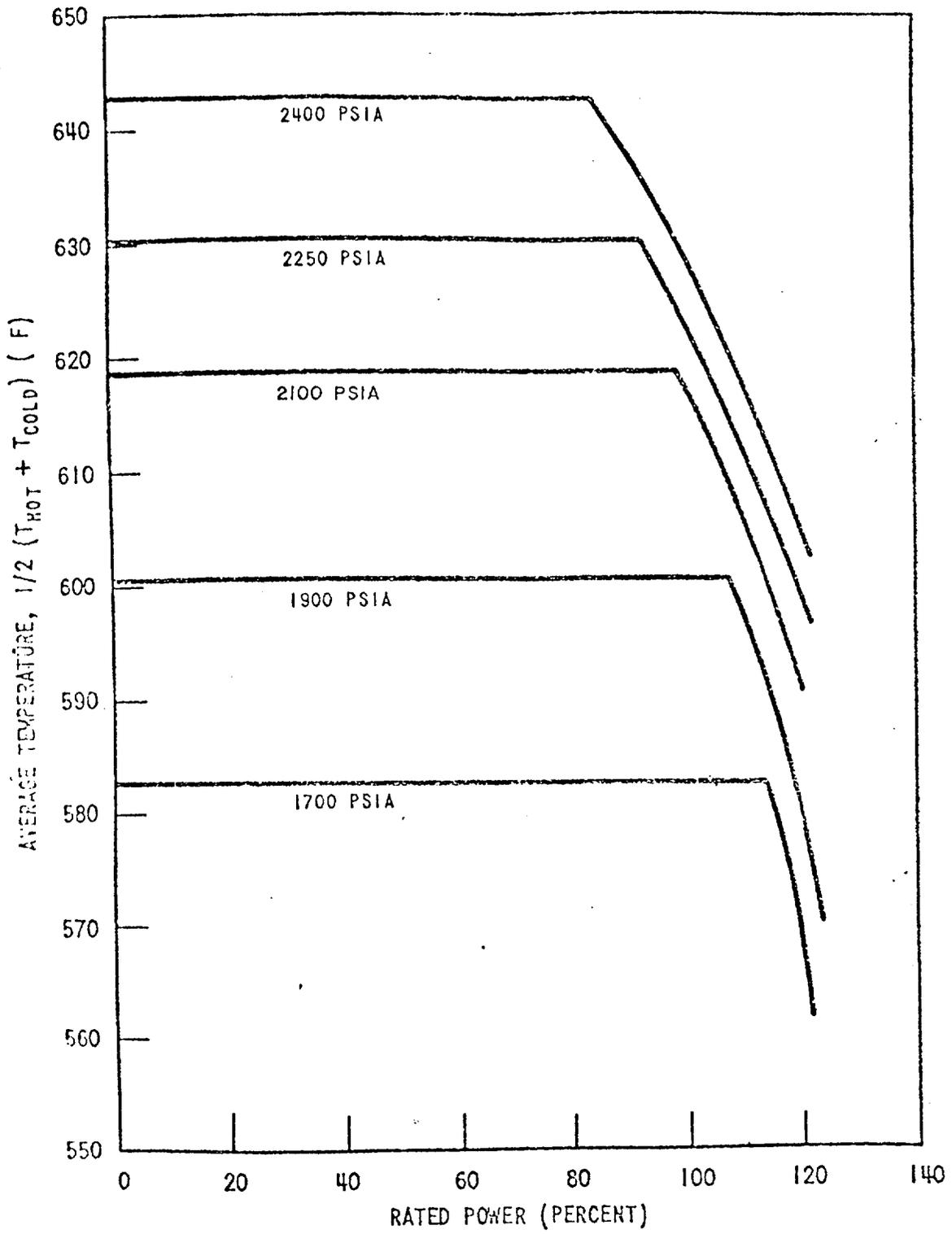


Figure 2.1-1. Reactor Core Thermal and Hydraulic Safety Limits, Three Loop Operation

Unit No. 3

Reactor Coolant Temperature

Overtemperature  $\Delta T$

$\leq \Delta T_o$

$$\left[ K_1 - 0.0174(T-566.6) + 0.000976(P-2085) - f(\Delta q) \right]$$

26

$\Delta T_o$  = Indicated  $\Delta T$  at rated power, F

T = Average temperature, F

P = Pressurizer pressure, psig

$f(\Delta q)$  = a function of the indicated difference between top and bottom detectors of the power-range nuclear ion chambers; with gains to be selected based on measured instrument response during startup tests such that:

For  $(q_t - q_b)$  within +10 percent and -14 percent where  $q_t$  and  $q_b$  are the percent power in the top and bottom halves of the core respectively, and  $q_t + q_b$  is total core power in percent of rated power,  $f(\Delta q) = 0$ .

For each percent that the magnitude of  $(q_t - q_b)$  exceeds +10 percent, the Delta-T trip set point shall be automatically reduced by 3.5 percent of its value at interim power.

For each percent that the magnitude of  $(q_t - q_b)$  exceeds -14 percent, the Delta-T trip set point shall be automatically reduced by 2 percent of its value at interim power.

$K_1$  (Three Loop Operation) = 1.120 ;  
(Two Loop Operation) = 0.88

$$\text{Over-power } \Delta T \leq \Delta T_0 \left[ 1.09 - K_1 \frac{dT}{dt} - K_2 (T - T') - f(\Delta q) \right]$$

- $\Delta T_0$  = Indicated  $\Delta T$  at rated power, F
- $T$  = Average temperature, F
- $T'$  = Indicated average temperature at nominal conditions and rated power, F
- $K_1$  = 0 for decreasing average temperature, 0.2 sec./F for increasing average temperature
- $K_2$  = 0.00134 for  $T$  equal to or more than  $T'$ ; 0 for  $T$  less than  $T'$
- $\frac{dT}{dt}$  = Rate of change of temperature, F/sec
- $f(\Delta q)$  = As defined above

Pressurizer

Low Pressurizer pressure - equal to or greater than 1915 psig.

High Pressurizer pressure - equal to or less than 2385 psig.

High Pressurizer water level - equal to or less than 92% of full scale.

Reactor Coolant Flow

Low reactor coolant flow - equal to or greater than 90% of normal indicated flow

Low reactor coolant pump motor frequency - equal to or greater than 56.1 Hz

Under voltage on reactor coolant pump motor bus - equal to or greater than 60% of normal voltage

Steam Generators

Low-low steam generator water level - equal to or greater than 5% of narrow range instrument scale

6. DNB PARAMETERS

The following DNB related parameter limits shall be maintained during power operation:

- a. Reactor Coolant System Tavg  $\leq$  570.6 °F
- b. Pressurizer Pressure  $\geq$  2070 psia\*

26

With any of the above parameters exceeding its limit, restore the parameter to within its limit within 2 hours or reduce thermal power to less than 5% of rated thermal power using normal shutdown procedures.

Compliance with this specification is demonstrated by verifying that each of the parameters is within its limits at least once each 12 hours.

\*Limit not applicable during either a THERMAL POWER ramp increase in excess of (5%) RATED THERMAL POWER per minute or a THERMAL POWER step increase in excess of (10%) RATED THERMAL POWER.

### 3.2 CONTROL ROD AND POWER DISTRIBUTION LIMITS

Applicability: Applies to the operation of the control rods and power distribution limits.

Objective: To ensure (1) core subcriticality after a reactor trip, (2) a limit on potential reactivity insertions from a hypothetical control rod ejection, and (3) an acceptable core power distribution during power operation.

Specification: 1. CONTROL ROD INSERTION LIMITS

- a. Whenever the reactor is critical, except for physics tests and control rod exercises, the shutdown control rods shall be fully withdrawn.
- b. For Unit 4, whenever the reactor is critical, except for physics tests and control rod exercises, the control group rods shall be no further inserted than the limits shown by the solid lines on Figure 3.2-1 for three loop operation and on Figure 3.2-1(a) for two loop operation. Control rod insertion limits for Unit 3, are shown on Figures 3.2-1(b) and 3.2-1(c). | 26
- c. For Unit 4, after 70% of the second and subsequent cycles as defined by burnup, the limits shall be adjusted as a linear function of burnup toward the end-of-core life as shown by the dotted lines on Figure 3.2-1. | 26
- d. The Unit 4, end-of-core life limit shown on Figure 3.2-1 and the Unit 3, control rod insertion limits shown on Figure 3.2-1(b) may be revised on the basis of physics calculations and physics data obtained during startup and subsequent operation. | 26
- e. Part length rods shall not be permitted in the core except for low power physics tests and for axial offset calibration tests performed below 75% of rated power.

CONTROL GROUP INSERTION LIMITS  
FOR THREE LOOP OPERATION

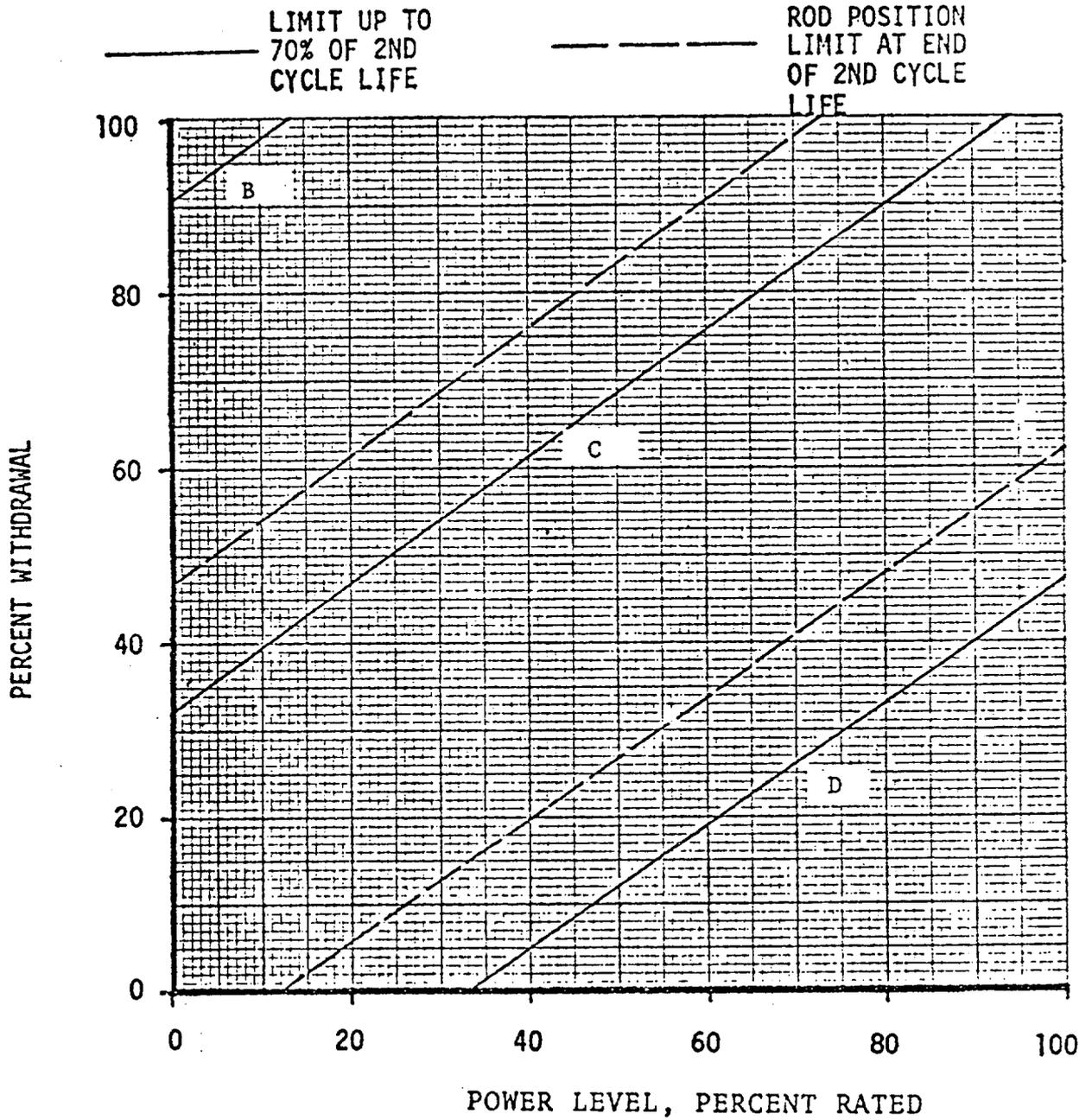


FIGURE 3.2-1

UNIT 4

CONTROL GROUP INSERTION LIMITS  
FOR TWO LOOP OPERATION

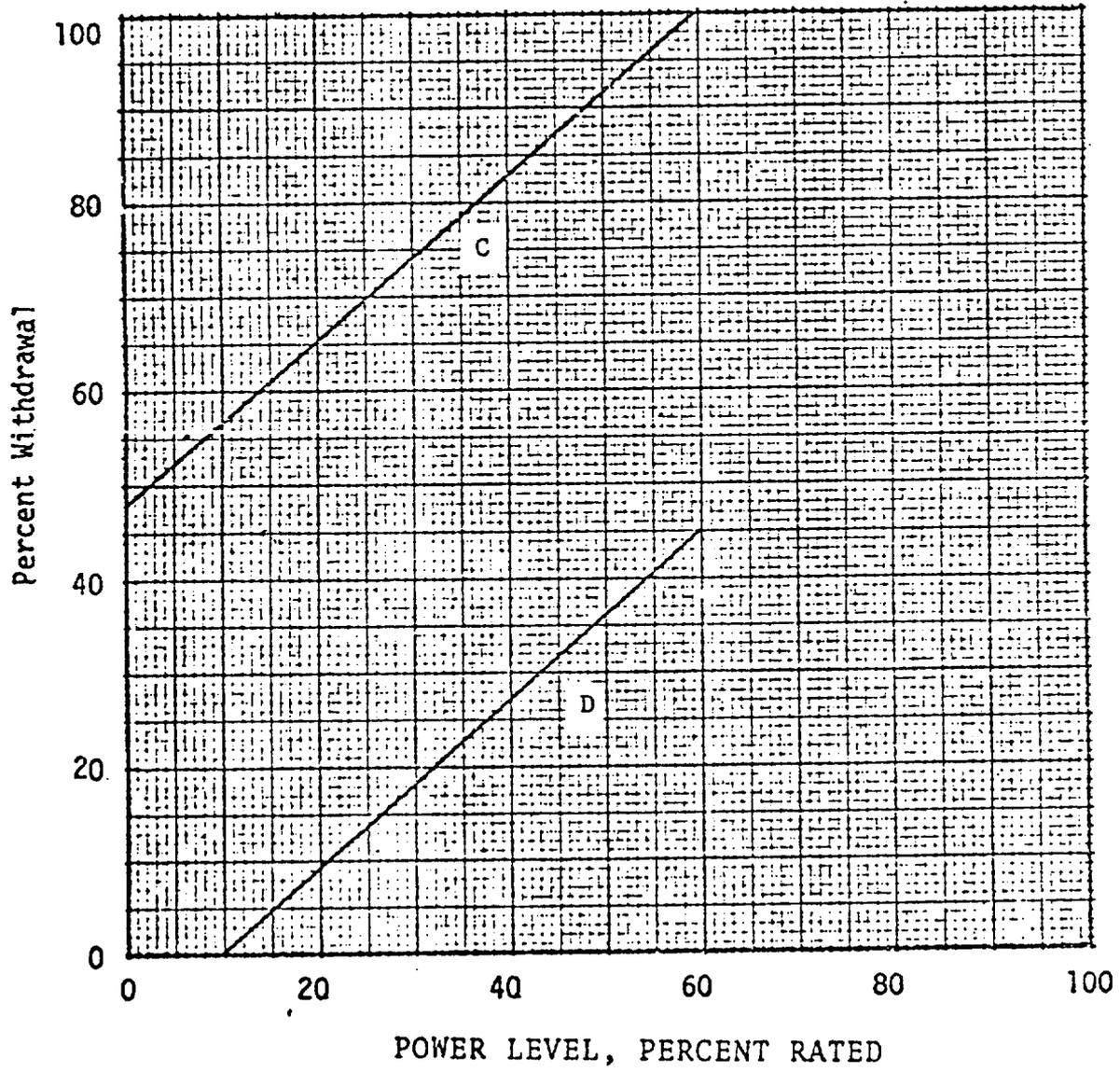


FIGURE 3.2-1(a)

UNIT 3

CONTROL GROUP INSERTION LIMITS  
FOR THREE LOOP OPERATION

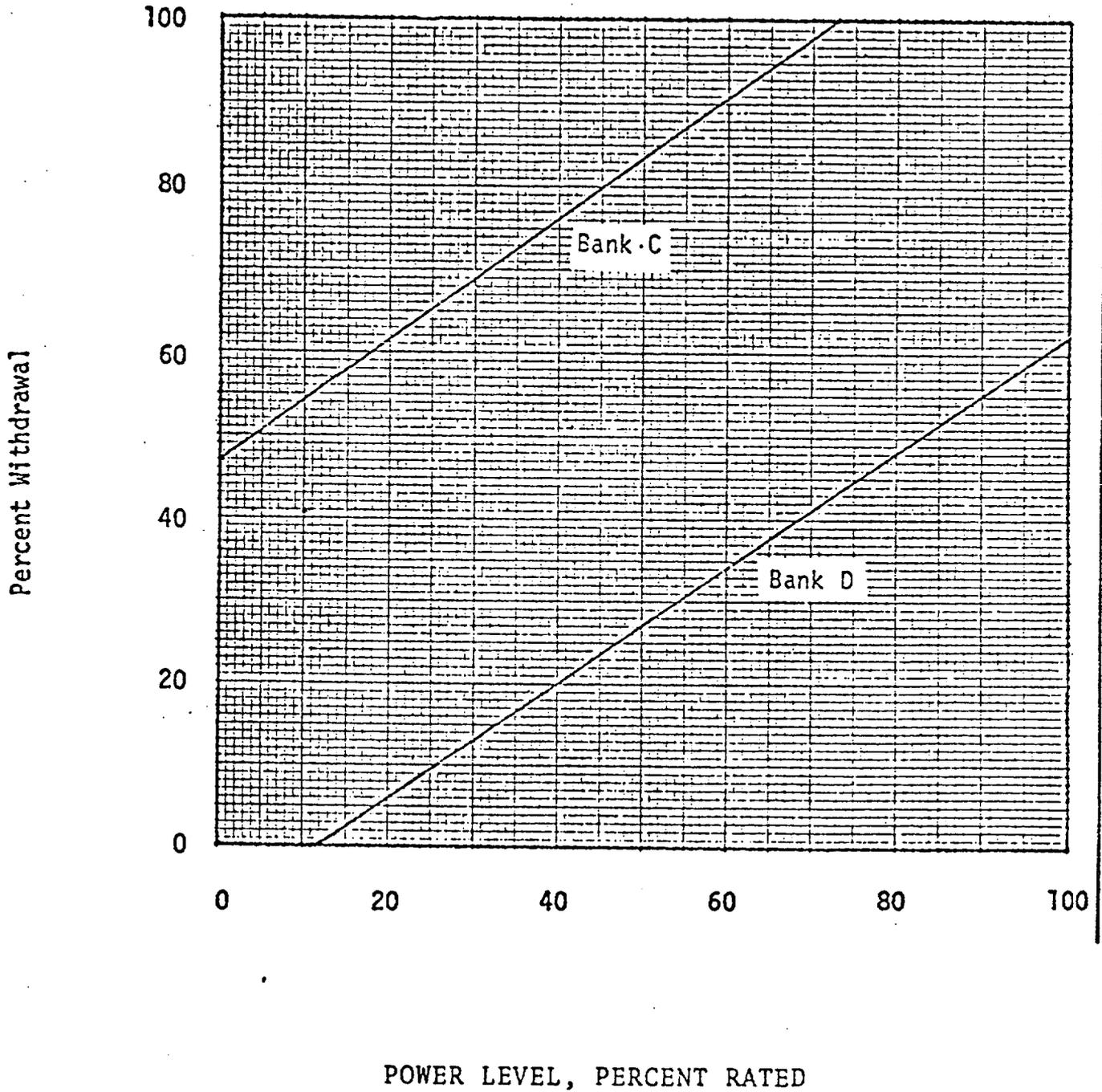


FIGURE 3.2-1(b).

UNIT 3

CONTROL GROUP INSERTION LIMITS  
FOR TWO LOOP OPERATION

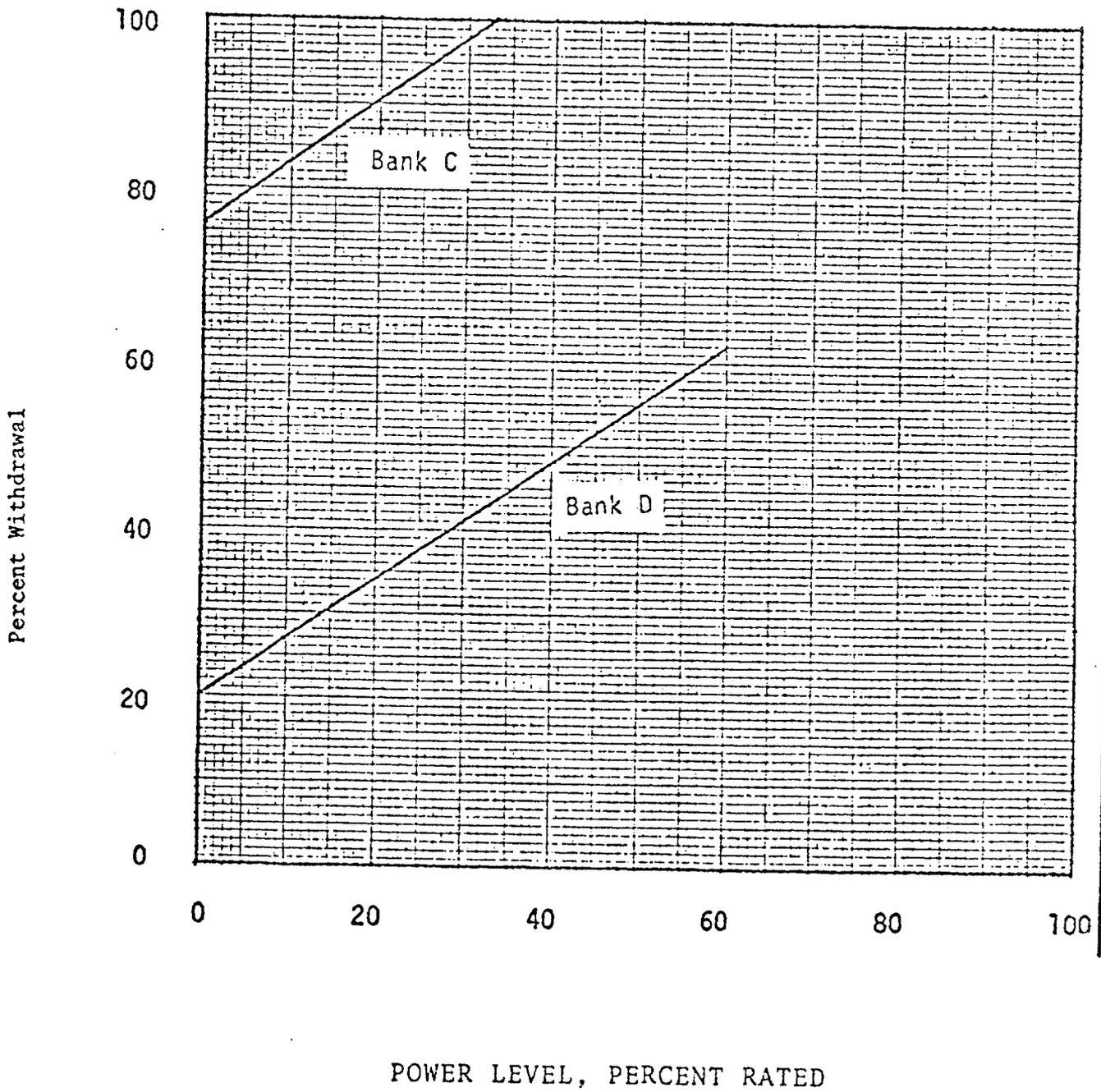


FIGURE 3.2-1(c)

Reactivity changes accompanying changes in reactor power are compensated by control rod motion. Reactivity changes associated with xenon, samarium, fuel depletion, and large changes in reactor coolant temperature (operating temperature to cold shutdown) are compensated by changes in the soluble boron concentration. During power operation, the shutdown groups are fully withdrawn and control of reactor power is by the control groups. A reactor trip occurring during power operation will put the reactor into the hot shutdown condition.

The control rod insertion limits provide for achieving hot shutdown by reactor trip at any time, assuming the highest worth control rod remains fully withdrawn, with sufficient margins to meet the assumptions used in the accident analysis.<sup>(1)</sup> In addition, they provide a limit on the maximum inserted rod worth in the unlikely event of a hypothetical rod ejection, and provide for acceptable nuclear peaking factors. The solid line shown on Figure 3.2-1 meets the shutdown requirement for the first 70% of second and subsequent cycles for Unit 4, except for two loop operation. The Unit 4 end-of-core life limit may be more restrictive, as shown by the conservative estimate represented by the dotted line. Figures 3.2-1(b) and 3.2-1(c) meet the shutdown requirements of Unit 3. The Unit 4 end-of-core-life limit and the Unit 3, rod insertion limits may be determined on the basis of startup and operating data to provide a more realistic limit which will allow for more flexibility in operation and still assure compliance with the shutdown requirement. Figure 3.2-1(a) shows the shutdown requirements for Unit 4 two loop operation. The maximum shutdown margin requirement occurs at end-of-core life and is based on the value used in analysis of the hypothetical steam break accident. Early in core life, less shutdown margin is required, and Figure 3.2-2 shows the shutdown margin equivalent to 1.77% reactivity at end-of-core-life with respect to an uncontrolled cooldown. All other accident analyses are based on 1% reactivity shutdown margin.

The overlap between successive control banks is allowed because the control rod worth is lower near the top and bottom of the core than in the center.

Positioning of the part-length rods is governed by the requirement to maintain the axial power shape within specified limits or to accept an automatic cutback of the overpower  $\Delta T$  and overtemperature  $\Delta T$  set points (see Specification 2.3). Thus, there is no need for imposing a limit on the physical positioning of the part-length rods.

UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 14 TO LICENSE NO. DPR-31, AND

AMENDMENT NO. 13 TO LICENSE NO. DPR-41

(CHANGE NO. 26 TO TECHNICAL SPECIFICATIONS)

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING UNITS 3 AND 4

DOCKETS NOS. 50-250 AND 50-251

Introduction

By letter dated September 9, 1975, Florida Power and Light Company (FPL) proposed changes to the Technical Specifications of Facility Operating Licenses DPR-31 and DPR-41 for Turkey Point Nuclear Generating Units 3 and 4. Supplemental information relating to the requested changes was supplied by FPL in their letters of October 29, November 21, and November 26, 1975. The proposed changes would modify operating limits in the Technical Specifications to allow operation of Unit 3, following refueling, at 2100 psia through core cycle 3. FPL requested: (1) the allowable fuel residence time (the minimum predicted time to clad flattening) be increased from 23,000 effective full power hours (EFPH) to 23,500 EFPH, (2) the overtemperature  $\Delta T$  and pressurizer low pressure limiting safety system settings (LSSS) be changed to allow operation at 2100 psia, and (3) the control rod insertion limits, a limiting condition for operation (LCO), be modified.

Because Units 3 and 4 share joint Technical Specifications, FPL proposed modifying the Technical Specifications for Unit 4 to reflect the proposed revision to the Unit 3 Technical Specifications. However, the operating limits for Unit 4 are unchanged by the Unit 3 reload for core cycle 3.

During the review of the proposed operating limits for Unit 3 the staff became aware of recent information that indicated that the bowing of fuel rods within the reactor may be greater than previously expected. FPL discussed the effect of this increased bowing of fuel rods in their submittals of November 21, and November 26, 1975.

Discussion

A. Reactor Core Description

The Unit 3 core loading for fuel cycle 3 will include 48 new prepressurized fuel assemblies. These assemblies, which are known as Region 5a and Region 5b fuel assemblies, have a slightly higher enrichment than do the Region 2 fuel assemblies they replace (2.60 and 2.90% U-235 vs. 2.56% U-235). However, the enrichment of the region 5a and 5b

fuel assemblies are within the enrichment range of other assemblies presently installed in the Unit 3 reactor (1.86% U-235 to 3.11% U-235). The increased enrichment compensates for the fission product reactivity poisoning produced within the reactor during previous operation. The Region 5 fuel assemblies have been fabricated by Westinghouse Electric Corporation, the fabricator of the fuel assemblies now loaded in the Turkey Point Units. They are mechanically identical to the presently installed fuel assemblies with the exception of one Region 5a fuel assembly which contains Zr O<sub>2</sub>-Y<sub>2</sub>O<sub>3</sub> marker pellets in two fuel rods. These marker pellets will permit the gathering of fuel stability data. The staff does not consider the inclusion of these inert marker pellets in two fuel rods as a significant departure from previously approved fuel element designs.

B. Minimum Time to Clad Flattening

Turkey Point Unit 3 has been operating at a reduced primary pressure of 1900 psia since early in core cycle 1. Reduced primary pressure was initiated during cycle 1 in order to lengthen the predicted time to clad flattening by reducing the pressure differential across the fuel cladding and thus reducing the clad creep rate. The presently specified Unit 3 fuel residence limit of 23,000 EFPH was calculated prior to the initiation of core cycle 2 operation and is the analytically determined minimum time to clad flattening for Region 3 fuel assemblies assuming: (1) a reactor system pressure of 1900 psia for core cycle 2 and (3) a reactor system pressure of 2250 from the beginning of core cycle 3 to the end of fuel life. Since FPL plans to increase the cycle 3 reactor system pressure to 2100 psia, instead of the previously assumed 2250 psia, FPL has recalculated the predicted minimum time to clad collapse. This recalculated value is 23,500 EFPH.

C. Departure from Nucleate Boiling (DNB) Protection

The core protection system operates by defining a region of permissible operation in terms of power, pressure, temperature, coolant flow and axial power distribution. This allowable operating region with regard to coolant temperature difference across the reactor core is determined by the equations which define the overtemperature  $\Delta T$  and overpower  $\Delta T$  reactor trips. The overtemperature  $\Delta T$  reactor trip protects the core against nucleate boiling, excessive hot channel exit quality, and hot channel boiling for any combination of power, pressure, temperature, and axial core power distribution. Similarly, the overpower  $\Delta T$  reactor trip provides protection against exceeding fuel rod design limits for accidents involving overpower excursions.

FPL has proposed modifications to the overtemperature  $\Delta T$  and pressurizer low pressure trip setpoints which are consistent with reactor operation at 2100 psia. No modifications to the overpower  $\Delta T$  reactor trip were proposed as the overpower  $\Delta T$  reactor trip is not a function of system pressure. The operating reactor coolant temperature will remain

unchanged when the reactor system pressure is increased to 2100 psia.

D. Control Rod Insertion Limits

Control of the operating reactor is provided by neutron absorbing control rods and soluble boric acid in the reactor coolant. The more boric acid contained in the reactor coolant the less the control rods need to be inserted to provide reactor control. The proposed control rod insertion limits are the result of analyses performed for the Unit 3 cycle 3 core configuration to insure: (1) an adequate shutdown margin is maintained throughout cycle life, (2) hot channel factors are maintained below design limits, (3) acceptable consequences of a rod ejection accident, and (4) acceptable consequences of rod misalignment. The maintenance of adequate shutdown margin at the end of core life is the consideration which defined the proposed control rod insertion limits.

E. Effect of Fuel Rod Bowing

FPL has employed the rod bow penalties determined by the Westinghouse Electric Company into their postulated accident and transient calculations. The rod bow penalties are based upon an analysis of all bowing experience to date. These penalties were determined from the inspection of 24 different regions of fuel assemblies (about 25,000 fuel rods) including more than 70 assemblies with burnups greater than 27,000 MWD/MTU.

Evaluation

A. Reactor Core Description

FPL's analysis of the loading pattern and their comparison of core physics parameters for core cycle 3 with those of core cycle 2 indicates the nuclear parameters for core cycle 3 fall within the range of values assumed in the Turkey Point Final Safety Analysis Report (FSAR). We agree that this comparison shows there are no significant differences between the Unit 3 cycle 2 and cycle 3 core configurations. Therefore, the consequences of previously analyzed accidents and transients are not increased by the Unit 3 reload for core cycle 3 and since these consequences were previously determined to be acceptable for Turkey Point, the conclusions of previous safety evaluations are unchanged by the core reload.

B. Minimum Time to Clad Flattening

FPL has proposed that the reactor system pressure for Unit 3 be increased to 2100 psia in order to prevent the internal fuel rod pressure from exceeding the reactor system pressure during core cycle 3. By maintaining the reactor pressure at 2100 psia, instead of increasing it to the design pressure of 2250 psia, FPL will

maximize the predicted minimum time to clad flattening and still maintain the maximum internal fuel rod pressure less than the reactor system operating pressure. FPL has recalculated the minimum time to clad flattening and has determined this time to be 23,500 EFPH for Unit 3 cycle 3 fuel assemblies, assuming reactor operation at 2100 psia. The results and conclusions of previous safety evaluations and previously approved operating limits, now in effect, remain unchanged as long as clad flattening is predicted not to occur. We have reviewed FPL's request and have approved the requested Unit 3 fuel residence time. Our approval is based on FPL's use of an approved clad flattening model and our independent determination that the model was used to determine the minimum time to clad flattening for the most critical assemblies in the cycle 3 fuel loading.

C. Departure from Nucleate Boiling (DNB) Protection

Following refueling for core cycle 3, FPL plans to operate Unit 3 at a reactor system pressure of 2100 psia. FPL, therefore, proposed overtemperature  $\Delta T$  and pressurizer low pressure reactor trip settings to be consistent with 2100 psia operation. Since the FPL does not plan to increase the temperature of the reactor coolant and since such a temperature increase would be possible using the proposed trip settings, the staff requested FPL to propose limiting conditions for operation (LCO) on reactor coolant average temperature and pressurizer pressure. FPL proposed these LCO's for inclusion in the Technical Specifications in their submittal of November 21, 1975. The inclusion of these LCO's in the Technical Specifications will add further assurance that actual reactor operating conditions are consistent with the operating conditions assumed in the analysis of Unit 3 postulated accidents and transients.

In their submittal of November 21, 1975, FPL also proposed for inclusion in the Technical Specifications a revised Reactor Core Thermal and Hydraulic Safety Limit curve for three loop operation at 2100 psia. A similar curve for two loop operation at 2100 psia was not included in the Technical Specifications, at this time, because operation of the reactor with less than three reactor coolant pumps in operation is prohibited by the Technical Specifications.

We have reviewed the proposed trip settings and operating limits and have concluded that they provide the necessary departure from nucleate boiling (DNB) protection in the event of any previously postulated accident or transient. Therefore, we can further conclude that the new limits provide adequate protection against fuel clad damage.

D. Control Rod Insertion Limits

FPL has analyzed the control rod insertion limits for three loop and two loop operation and has proposed Unit 3 control rod insertion limits

which are more conservative than those previously specified. The limiting constraint in defining the proposed control rod insertion limits is the maintenance of adequate shutdown margin at the end of core life. Use of the more conservative proposed control rod insertion limits will increase the minimum available shutdown margin, maintain an acceptable core power distribution, decrease the consequences of a control rod ejection accident and decrease the consequences of control rod misalignment. Moreover, since we find that the use of the proposed control rod insertion limits will not effect previously performed applicable safety analyses, we conclude that the proposed insertion limits are acceptable.

E. Effect of Fuel Rod Bowing

FPL has performed an analysis to evaluate the effect of fuel rod bow on the calculated emergency core cooling system (ECCS) performance during a loss of coolant accident (LOCA). The analysis was performed for Region 3 fuel, the fuel with the highest stored energy in core cycle 3, and uses the power spike distribution for fuel assembly end-of-life conditions. Our review of FPL's evaluation supports the conclusion that the peak clad temperature following a LOCA will be less than 2200°F for core cycle 3 conditions. Therefore, the calculated ECCS cooling performance for Turkey Point Unit 3 conforms to the peak clad temperature criteria of 10 CFR 50.46 (b).

In addition to the peak power penalty associated with the rod bowing phenomenon there has also been determined to be a DNB penalty resulting from the displaced coolant flow. This penalty is 5.6% maximum, but considering the limited linear power density this penalty is reduced to 3.1% for Turkey Point Unit 3 core cycle 3. This penalty is accounted for by conservatism in the present DNB model which are in excess of 3.3%.

Summary

Our evaluation supports the conclusion that: (1) clad flattening is predicted not to occur during a projected fuel residence time of 23,500 EFPH, (2) the reactor trip settings and LCO's incorporated in the Technical Specifications are appropriate for use during Unit 3 cycle 3 operation at 2100 psia, (3) the proposed control rod insertion limits are conservative when compared to those now in effect, and (4) the effect of fuel rod bowing does not change the results of previously postulated accidents or transients and does not change the conclusions of earlier safety evaluations. We have determined that there are no significant differences between core cycle 2 and core cycle 3 and that the conclusions of our earlier safety evaluations remain unchanged for both 1900 psia and 2100 psia operation. Moreover, we have concluded that the proposed changes have been appropriately incorporated into the proposed Technical Specifications and are acceptable.

Conclusion

We have concluded, based on the considerations discussed above, that:  
(1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and  
(2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATED: December 11, 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments No. 14 and No. 13 to Facility Operating Licenses Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units 3 and 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendment modifies the operating limits in the Technical Specifications to allow operation of Turkey Point Nuclear Generating Unit 3, following refueling, at 2100 psia until the extended fuel residence time is reached. The operating limits for Unit 4 set forth in the Technical Specifications remain unchanged although the Unit 4 Technical Specifications will be modified to reflect the revisions to the Unit 3 Technical Specifications.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Notice of Proposed Issuance of Amendment to Facility

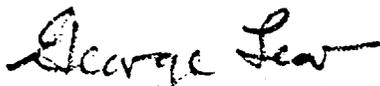
Operating License in connection with this action was published in the FEDERAL REGISTER on October 31, 1975 (40 FR 50745). No request for a hearing or petition for leave to intervene was filed following notice of the proposed action.

For further details with respect to this action, see (1) the application for amendment dated September 9, 1975 and supplements dated October 29, November 21 and November 26, 1975, (2) Amendments Nos. 14 and 13 to Licenses Nos. DPR-31 and DPR-41, with Change No. 26 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland, this 11th day of December, 1975.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Reactor Licensing



LICENSE AUTHORITY FILE COPY  
UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

DO NOT REMOVE

April 12, 1976

*posted*

*Am-15  
to  
DPR-31*

Dockets Nos. 50-250  
and 50-251

Florida Power and Light Company  
ATTN: Dr. Robert E. Uhrig  
Vice President  
P. O. Box 013100  
Miami, Florida 33101

Gentlemen:

In response to your request dated September 20, 1974, the Commission has issued the enclosed Amendments Nos. 15 and 14 to Facility Operating Licenses Nos. DPR-31 and DPR-41 for the Turkey Point Nuclear Generating Units 3 and 4.

The amendments consists of changes in the Technical Specifications that modify the surveillance requirements for the emergency containment cooling fans.

Please note that we have discontinued the use of separate identifying numbers for changes to technical specifications. Sequential amendment numbers will be continued as in the past.

Copies of the related Safety Evaluation and the Federal Register Notice also are enclosed.

Sincerely,

*George Lear*

George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Enclosures:

1. Amendment No. 15 to DPR-31
2. Amendment No. 14 to DPR-41
3. Safety Evaluation
4. Federal Register Notice

cc w/enclosures:  
See next page

cc:

Mr. Jack R. Newman, Esquire  
Lowenstein, Newman, Reis & Axelrad  
1025 Connecticut Avenue, N. W.  
Suite 1214  
Washington, D. C. 20036

Environmental & Urban Affairs Library  
Florida International University  
Miami, Florida 33199

Mr. Ed Maroney  
Bureau of Intergovernmental Relations  
725 South Bronough Street  
Tallahassee, Florida 32304



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15  
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

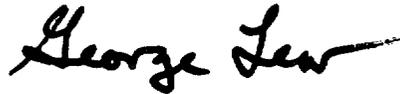
Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility Operating License No. DPR-31 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licenses shall operate the facility in accordance with the Technical Specifications, as revised".

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: April 12, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 15

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-31

DOCKET NO. 50-250

Replace page 4.6-2 with the attached revised pages.

## 2. COMPONENT TESTS

### Pumps and Fans

The containment spray pumps and the Emergency Containment Cooling fans shall be started at intervals not greater than one (1) month.

Acceptable levels of performance shall be that the pumps reach their rated shut off heads, the fan motors reach their nominal operating current for the containment atmosphere during the test, and both operate for at least fifteen minutes.

### Valves

The systems motor operated isolation valves will be tested for operation during system tests.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 14  
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
  - E. An environmental statement or negative declaration need not be prepared in connection with the issuance of this amendment.

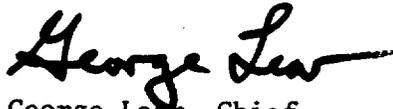
Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B. of Facility Operating License No. DPR-41 is hereby amended to read as follows:

"B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licenses shall operate the facility in accordance with the Technical Specifications, as revised".

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors

Attachment:  
Changes to the  
Technical Specifications

Date of Issuance: April 12, 1976

ATTACHMENT TO LICENSE AMENDMENT NO. 14

TO THE TECHNICAL SPECIFICATIONS

FACILITY OPERATING LICENSE NO. DPR-41

DOCKET NO. 50-251

Replace page 4.6-2 with the attached revised pages.

## 2. COMPONENT TESTS

### Pumps and Fans

The containment spray pumps and the Emergency Containment Cooling fans shall be started at intervals not greater than one (1) month.

Acceptable levels of performance shall be that the pumps reach their rated shut off heads, the fan motors reach their nominal operating current for the containment atmosphere during the test, and both operate for at least fifteen minutes.

### Valves

The systems motor operated isolation valves will be tested for operation during system tests.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 15 TO LICENSE NO. DPR-31, AND

AMENDMENT NO. 14 TO LICENSE NO. DPR-41

FLORIDA POWER AND LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING UNITS 3 AND 4

DOCKETS NOS. 50-250 AND 50-251

Introduction

By letter dated September 20, 1974, Florida Power and Light Company (FPL) proposed changes to the Technical Specifications of Facility Operating Licenses DPR-31 and DPR-41 for Turkey Point Nuclear Generating Units 3 and 4. FPL requested that the surveillance test conditions for the emergency containment cooling fans be modified to state that the tests are acceptable provided the electric current to the emergency containment cooling fans reaches its nominal operating value. The present Technical Specifications state that the current to the fan motor must reach its rated value for the test to be judged acceptable.

Discussion

The emergency containment cooling system is designed to remove sufficient heat from the reactor containment following a loss of coolant accident (LOCA) to maintain the containment pressure below its design value.

The emergency containment cooling system consists of an electric motor, fan, bare tube cooling coil, instrumentation and controls. During the pressure transient following a LOCA, two of the three fan cooling units are assumed to be operating.

The Turkey Point Technical Specifications require that the operability of the emergency containment cooling system fans be tested monthly by operating for at least 15 minutes. These monthly operational tests assure that the emergency containment cooling system fans will be available to provide their design functions when required.

FPL has proposed that the monthly operational tests for the emergency containment cooling system fans be judged acceptable provided the current to the fan motors reaches its nominal operating value. The present Technical Specifications state that the current must reach the motor's rated value

for the test to be judged acceptable. FPL requests this Technical Specification change because the rated current for the fan motor is in excess of the operating current by a significant margin. This margin is an intentional design consideration and its purpose is to assure that the fan motors operate, under nominal conditions, with a current draw less than the motor's rated current.

#### Evaluation

The proposed Technical Specification change has been evaluated by the NRC staff. We conclude that the present acceptability requirement that the emergency containment fan motors be loaded to their rated current is unobtainable and; therefore, meaningless. The fans are rated for ambient conditions following the Maximum Hypothetical Accident; and these conditions cannot be duplicated for the test. The test conditions provide much lower loads due to the lower temperatures and a lower moisture content in the atmosphere preventing the fan motors from reaching their rated current. The modified wording bases the performance of the fans on the containment atmosphere at the time of the test. Our evaluation of the proposed change included consideration of the fan motor rated current, the nominal operating current during the monthly test and the nominal current during accident conditions.

We conclude that the monthly operational test for the emergency containment fan motors, as required by the proposed Technical Specification change; (1) is consistent with recent NRC requirements, and (2) provides adequate assurance that the emergency containment cooling system fans will function when required. We further conclude that the proposed change does not increase the probability or consequences of accidents previously considered and does not significantly decrease a safety margin. Therefore, the proposed Technical Specification change is acceptable.

#### Environmental Consideration

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and pursuant to 10 CFR §51.5(d)(4) that an environmental statement, negative declaration, or environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) because the changes do not involve a significant increase in the probability or consequences of accidents previously considered and do not involve a significant decrease in a safety margin, the changes do not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: April 12, 1976

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY  
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 15 and 14 to Facility Operating Licenses Nos. DPR-31 and DPR-41, respectively, issued to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units 3 and 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

The amendments consists of changes in the Technical Specifications that modify the surveillance requirements for the emergency containment cooling fans.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment is not required since the amendment does not involve a significant hazards consideration.

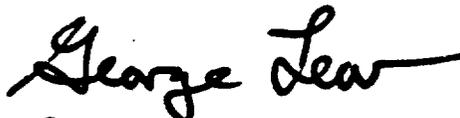
The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) an environmental statement, negative declaration or environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) the application for amendment dated September 20, 1974, (2) Amendments Nos. 15 and 14 to Licenses Nos. DPR-31 and DPR-41 and (3) the Commission's related Safety Evaluation. All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street N.W., Washington, D. C. and at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida 33199.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D.C. 20555, Attention: Director, Division of Operating Reactors.

Dated at Bethesda, Maryland, this 12 day of April 1976.

FOR THE NUCLEAR REGULATORY COMMISSION



George Lear, Chief  
Operating Reactors Branch #3  
Division of Operating Reactors