September 6, 1972

Docket No. 50-250

Florida Power & Light Company
ATTN: Dr. James Coughlin
P. O. Box 3100
Miami, Florida 33101

Change No. 1
License No. DPR-31

Gentlemen:

Your letter dated August 11, 1972, proposed a change in the Technical Specifications attached as Appendix A to Facility Operating License No. DPR-31. The proposed change, a reduction in power tilt limits, is the outgrowth of the reevaluation of these limits you performed at our request. This proposal has been designated Change No. 1.

We have reviewed your reevaluation of power tilt limits and find that it adequately answers the questions we raised regarding this matter in our original inquiry dated August 1, 1972. Further, we find that the Technical Specification changes contained in your proposal reflect this reevaluation and are acceptable.

We have concluded that the proposed change does not involve significant hazard considerations not described or implicit in the Final Safety Analysis Report and that there is reasonable assurance that the health and safety of the public will not be endangered.

Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specification change proposed in your letter dated August 11, 1972, is hereby authorized. To effect this change pages 3.2-3 and 3.2-4 are revised as follows:

1. The power tilt limit contained in paragraphs 6.b and 6.c is changed from 1.10 to 1.02.
2. The power tilt limit contained in paragraph 6.d is changed from 1.25 to 1.12.

const Rev

The Technical Specifications of Facility Operating License No. DPR-31 are hereby changed as set forth in the revised pages 3.2-3 and 3.2-4 which are enclosed.

Sincerely,

Original signed by R. C. DeYoung

R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing

Enclosure:
Revised pages 3.2-3 & 3.2-4

cc:
Mr. Jack Newman
Lowenstein, Newman & Reis
1100 Connecticut Avenue NW
Washington, D. C. 20036

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it shall not be permitted if the rod has a potential reactivity insertion upon ejection greater than 0.3% $\Delta k/k$ at rated power. Inoperable rod worth shall be determined within 4 weeks.

b. A control rod shall be considered inoperable if
   (a) the rod cannot be moved by the CRDM, or
   (b) the rod is misaligned from its bank by more than 15 inches, or
   (c) the rod drop time is not met.

c. If a control rod cannot be moved by the drive mechanism, shutdown margin shall be increased by boron addition to compensate for the withdrawn worth of the inoperable rod.

5. CONTROL ROD POSITION INDICATION

If either the power range channel deviation alarm or the rod deviation monitor alarm are not operable, rod positions shall be logged once per shift and after a load change greater than 10% of rated power. If both alarms are inoperable for two hours or more, the nuclear overpower trip shall be reset to 93% of rated power.

6. POWER DISTRIBUTION LIMITS

a. After initial fuel loading and refueling operation the in-core instrumentation shall be used to confirm power distribution prior to operation at rated power.

b. If the quadrant to average power tilt exceeds 1.02 except for physics tests, then within 8 hours,
   1. The tilt shall be returned to less than 1.02, or
   2. The hot channel factors shall be determined and power shall be reduced one percent for each percent the hot channel factor exceeds the values in 2, or

3. Power shall be restricted to rated power less two percent for each one percent the tilt ratio exceeds 1.0.

c. If after a period of 24 hours, the power tilt in 6.b. above is not corrected to less than 1.02,

1. An evaluation as to the cause of the discrepancy shall be made and reported to the Atomic Energy Commission, and

2. The nuclear overpower, overpower ΔT and overtemperature ΔT trips shall be reduced one percent for each percent the operating power level has been reduced.

d. If the quadrant to average power tilt exceeds 1.12, except for physics tests, the reactor shall be put in the hot shutdown condition, and the Atomic Energy Commission shall be notified.

7. IN-CORE INSTRUMENTATION

a. A minimum of 16 thimbles, at least 2 per quadrant, and the necessary associated detectors shall be operable during the check and calibration of nuclear instrumentation ion chambers.

b. Power shall be limited to 90% of rated power for 3 loop or 50% of rated power for 2 loop operation if the requirements in 7.a. are not met.