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Docket No. 50-250
50-251

MAY 1 - 1973

Florida Power & Light Company
ATTN: Dr. James Coughlin
P. O. Box 3100
Miami, Florida 33101

Change No. 5
License Nos. DPR-31 and 41

Gentlemen:

By letter dated March 16, 1973, you proposed seven revisions to the Technical Specifications attached as Appendix A to Facility Operating Licenses DPR-31 and 41. This action is designated Change No. 5.

We have reviewed these proposed changes and approve them on the basis that, with the exception of the seventh, all the changes take the form of corrections or clarifications. The seventh change, certain revisions in the plant operating organization arising from personnel reassignments and a recent FPL reorganization, is acceptable in that the plant organization remains in conformity with our requirements regarding numbers and qualifications of key personnel.

We conclude that the changes do not involve significant hazard considerations not described or implicit in the Final Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered. Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Facility Operating Licenses DPR-31 and 41 are hereby changed as set forth in revised pages which are enclosed.

Sincerely,

/s/

R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing

Enclosure:
As Stated

cc: Jack Newman

bcc: J. R. Buchanan, ORNL
Thomas B. Abernathy, DTIE

OFFICE ▶	PWR-2	PWR-2	RO	AD/PWRs		
7701 SURNAME ▶	PSCheck:nlg	Karl Kniel	RHEngelken	RCDeYoung		
DATE ▶	4/23/73	4/26/73	5/1/73	5/1/73		

AEC-318 (Rev. 9-53) AECM 0240 GPO c43—16—81465-1 445-678

UNITED STATES
ATOMIC ENERGY COMMISSION
WASHINGTON, D.C. 20545

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Docket No. 50-250
50-251

Florida Power & Light Company
ATTN: Dr. James Coughlin
P. O. Box 3100
Miami, Florida 33101

Change No. 5
License Nos. DPR-31 and 41

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We conclude that the changes do not involve significant hazard considerations not described or implicit in the Final Safety Analysis Report and there is reasonable assurance that the health and safety of the public will not be endangered. Accordingly, pursuant to Section 50.59 of 10 CFR Part 50, the Technical Specifications of Facility Operating Licenses DPR-31 and 41 are hereby changed as set forth in revised pages which are enclosed.

Sincerely,

R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing

Enclosure:
As Stated

cc: Jack Newman

P.O. BOX 3100 MIAMI, FLORIDA 33101

FLORIDA POWER & LIGHT COMPANY

March 16, 1973

Mr. R. C. DeYoung, Assistant Director
for Pressurized Water Reactors
Directorate of Licensing
U. S. Atomic Energy Commission
Washington, D. C. 20545

Dear Mr. DeYoung:

Re: Turkey Point Units 3 and 4
Docket Nos. 50-250 and 50-251
Proposed Changes in Technical Specifications

In accordance with 10 CFR 50.59 Florida Power & Light Company herewith submits twenty-two (22) copies of proposed changes in the Technical Specifications for the subject facility as listed below:

1. Page 1-5, Specification 1.14

   Inserted "normalized" in two places. As read or raw detector currents can not be used directly for power tilt calculations.

2. Page 3.1-5

   Corrected paragraph designations to e. and f.

3. Page 3.4-1 Paragraph 1.a., Page 3.4-3 Paragraph 2.a., Page 3.4-4 Paragraphs 3.a. and 4.a.

   Changed "temperature" to "power".

4. Table 3.5-3

   Corrected "2.2" to "2.1" in no. 1.3 and 2.2.

5. Page 3.6-2 Paragraph d.

   Changed "3.6c" to "3.6b and c".

6. Table 4.1-1 (3 sheets)

   Added note to item 1. and changed "chambers" to "detectors". Added footnote marked "†" and noted applicable items.

7. Pages 6.1-1, 6.1-2, 6.1-3, 6.1-4, 6.1-5, 6.1-6, 6.1-7, 6.1-8, 6.2-1, 6.3-1, 6.4-5, 6.4-6, Figures 6.1-1 and 6.1-2.

   The above have been updated to show current plant organization and new titles of Power Resources (formerly Production Department) personnel.

1829

In 6.1.3 the Plant Manager, Plant Superintendent-Nuclear, and Assistant Plant Superintendent-Nuclear Maintenance positions are new or rewritten.

In 6.1.4 the Radiochemist has been added to the Plant Nuclear Safety Committee.

The proposed new Technical Specifications pages are all identified by today's date in the lower left hand corner.

Yours very truly,

James Coughlin
Vice President

JC:rp
Enclosures

cc: Mr. Jack R. Newman

STATE OF FLORIDA)
) SS
COUNTY OF DADE)

JAMES COUGHLIN, being first duly sworn, deposes and says:

That he is a Vice President of Florida Power & Light Company, the Applicant herein;

That he has executed the foregoing instrument; that the statements made in this said instrument are true and correct to the best of his knowledge, information and belief; and that he is authorized to execute the instrument on behalf of said Applicant.

Signed _____

Subscribed and sworn to before me this 16th day of March, 1973.

Rebecca H Powell
Notary Public in and for the
County of Dade, State of Florida

NOTARY PUBLIC, STATE of FLORIDA at LARGE
MY COMMISSION EXPIRES APRIL 2, 1976
BONDED THRU MAYNARD BONDING AGENCY

My Commission expires _____

1.13 ABNORMAL OCCURRENCE

An abnormal occurrence is defined as any of the following:

1. A safety system setting less conservative than the limiting setting established in the Technical Specifications.
2. Violation of a limiting condition for operation established in the Technical Specifications.
3. An uncontrolled or unplanned release of radioactive material from any plant system designed to act as a boundary for such material in an amount of significance with respect to limits prescribed in Technical Specifications.
4. Failure of a component of an engineered safety feature or safety system that causes or threatens to cause the feature or system to be incapable of performing its intended function. Simultaneous failure of more than one component making up a redundant system shall be considered a failure under this definition. In addition, any failure of a component of an engineered safety feature or safety system shall be considered a failure under this definition unless it can be shown that the fault was not generic in nature.
5. Abnormal degradation of one of the several boundaries designed to contain the radioactive materials resulting from the fission process.
6. Significant (greater than 1% $\Delta k/k$) uncontrolled or unanticipated changes in reactivity.
7. Observed inadequacies in the implementation of administrative or procedural controls such that the inadequacy causes or threatens to cause the existence or development of an unsafe condition in connection with the operation of the plant.
8. Conditions arising from natural or offsite manmade events that affect or threaten to affect the safe operation of the plant.

1.14 POWER TILT

The power tilt is the ratio of the maximum to average of the upper out-of-core normalized detector currents or the lower out-of-core normalized detector currents whichever is greater. If one out-of-core detector is out of service, the remaining three detectors are to be used to compute the average.

e. After shutdown, corrective action shall be taken before operation is resumed.

f. Above 2% of rated power, two leak detection systems of different principles shall be operable, one of which is sensitive to radioactivity. The latter may be out of service for 48 hours provided two other systems are operable.

4. MAXIMUM REACTOR COOLANT ACTIVITY

The total specific activity of the reactor coolant due to nuclides with half-lives of more than 30 minutes, excluding tritium, shall not exceed $135/\bar{E}$ * µCi/cc whenever the reactor is critical or the average reactor coolant temperature is greater than 500F.

If the limit above is not satisfied, the reactor shall be shutdown and cooled to 500F or less within 6 hours.

* $\bar{E}$ is the average of beta and gamma energy (Mev) per disintegration of the specific activity.

3.4 ENGINEERED SAFETY FEATURES

Applicability: Applies to the operating status of the Engineered Safety Features.

Objective: To define those limiting conditions for operation that are necessary: (1) to remove decay heat from the core in emergency or normal shutdown situations, (2) to remove heat from containment in normal operating and emergency situations, and (3) to remove airborne iodine from the containment atmosphere in the event of a Maximum Hypothetical Accident.

Specification:

1. SAFETY INJECTION AND RESIDUAL HEAT REMOVAL SYSTEMS

a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met:

1. The refueling water tank shall contain not less than 320,000 gal. of water with a boron concentration of at least 1950 ppm.

2. The boron injection tank shall contain not less than 900 gal. of a 20,000 to 22,500 ppm boron solution. The solution in the tank, and in isolated portions of the inlet and outlet piping, shall be maintained at a temperature of at least 145F. TWO channels of heat tracing shall be operable for the flow path.

3. Each accumulator shall be pressurized to at least 600 psig and contain 775-791 ft^3 of water with a boron concentration of at least 1950 ppm, and shall not be isolated.

4. FOUR safety injection pumps shall be operable.

5. ONE residual heat exchanger may be out of service for a period of 24 hours.

6. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating maintenance, all valves that provide the duplicate function shall be tested to demonstrate operability.

2. EMERGENCY CONTAINMENT COOLING SYSTEMS

a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met:

1. THREE emergency containment cooling units are operable.

2. TWO containment spray pumps are operable.

3. All valves and piping associated with the above components, and required for post accident operation, are operable.

b. During power operation, the requirements of 3.4.2a may be modified to allow one of the following components to be inoperable (including associated valves and piping) at any one time. If the system is not restored to meet the requirements of 3.4.2a within the time period specified, the reactor shall be placed in the hot shutdown condition. If the requirements of 3.4.2a are not satisfied within an additional 48 hours the reactor shall be placed in the cold shutdown condition.

1. ONE emergency containment cooling unit may be out of service for a period of 24 hours. Prior to initiating maintenance the other TWO units shall be tested to demonstrate operability.

2. ONE containment spray pump may be out of service provided it is restored to operable status within 24 hours. The remaining containment spray pump shall be tested to demonstrate operability before initiating maintenance on the inoperable pump.

3. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating repairs, all valves that provide the duplicate function shall be tested to demonstrate operability.

3. EMERGENCY CONTAINMENT FILTERING SYSTEM

a. The reactor shall not be made critical, except for low power physics tests unless:

1. THREE emergency containment filtering units are operable.

2. All valves, interlocks and piping associated with the above components and required for post-accident operation, are operable.

b. During power operation:

1. ONE unit may be inoperable for a period of 24 hours if the other TWO are operable.

2. Any valve in the system may be inoperable provided repairs are completed within 24 hours. Prior to initiating maintenance, all valves that provide the duplicate function shall be tested to demonstrate operability.

4. COMPONENT COOLING SYSTEM

a. The reactor shall not be made critical, except for low power physics tests, unless the following conditions are met:

TABLE 3.5-3

INSTRUMENT OPERATING CONDITIONS FOR ISOLATION FUNCTIONS

NO.	FUNCTIONAL UNIT	1 MIN. OPERABLE CHANNELS	2 MIN. DEGREE OF REDUNDANCY	3 OPERATOR ACTION IF CONDITIONS OF COLUMN 1 OR 2 CANNOT BE MET
1.	CONTAINMENT ISOLATION			
1.1	Manual	2	*	Cold Shutdown
1.2	Safety Injection	See Item No. 1 of Table 3.5-2		Cold Shutdown
1.3	High Containment Pressure	See Item 2.1 of Table 3.5-2		Cold Shutdown
2.	STEAM LINE ISOLATION			
2.1	High Steam Flow in 2/3 Lines and 2/3 Low T_{avg} or 2/3 Low Steam Pressure	See Item 1.5 in Table 3.5-2		Cold Shutdown
2.2	High Containment Pressure	See Item No. 2.1 of Table 3.5-2		Cold Shutdown
2.3	Manual	1/line		Hot Shutdown
3.	FEEDWATER LINE ISOLATION			
3.1	Safety Injection	See Item No. 1 of Table 3.5-2		Cold Shutdown

* Must actuate two push buttons simultaneously

1. TWO associated charging pumps shall be operable.

2. THREE boric acid transfer pumps shall be operable.

3. The boric acid tanks together shall contain a minimum of 6160 gallons of a 20,000 to 22,500 ppm boron solution at a temperature of at least 145F.

4. System piping, interlocks and valves shall be operable to the extent of establishing one flow path from the boric acid tanks, and one flow path from the refueling water storage tank, to each Reactor Coolant System.

5. TWO channels of heat tracing shall be operable for the flow path from the boric acid tanks.

6. The primary water storage tank contains not less than 30,000 gallons of water.

d. During power operation, the requirements of 3.6.b and c may be modified to allow one of the following components to be inoperable. If the system is not restored to meet the requirements of 3.6b and c within the time period specified, the reactor(s) shall be placed in the hot shutdown condition. If the requirements of 3.6.b and c are not satisfied within an additional 48 hours, the reactor(s) shall be placed in the cold shutdown condition.

1. One of the two operable charging pumps may be removed from service provided that it is restored to operable status within 24 hours.

2. One boric acid transfer pump may be out of service provided that it is restored to operable status within 24 hours.

3. One channel of heat tracing may be out of service for 24 hours.

TABLE 4.1-1

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TEST OF INSTRUMENT CHANNELS

	Channel Description	Check	Calibrate	Test	Remarks
1.	Nuclear Power Range (Check, Calibrate and Test only applicable above 10% of rated power.)	S (1) M*(4)	D (2) Q*(4)	M (3)	1) Load v.s. flux curve 2) Thermal power calculation 3) Signal to ΔT; bistable action (permissive, rod stop, trips) 4) Upper & lower detectors for symmetric offset (+5 to -5%)
2.	Nuclear Intermediate Range	S (1)	N.A.	P (2)	1) Once/shift when in **service** 2) Log level; bistable action (permissive, rod stop, trip)
3.	Nuclear Source Range	S (1)	N.A.	P (2)	1) Once/shift when in service 2) Bistable action (alarm, trip
4.	Reactor Coolant Temperature	S†	R	B/W (1)† (2)†	1) Overtemperature-ΔT 2) Overpower-ΔT
5.	Reactor Coolant Flow	S†	R	M†	
6.	Pressurizer Water Level	S†	R	M†	
7.	Pressurizer Pressure	S†	R	M†	
8.	4 kv Voltage & Frequency	N.A.	R**	R	Reactor protection circuits only
9.	Analog Rod Position	S†	R	M†	With step counters

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TABLE 4.1-1 (Continued)

	Channel Description	Check	Calibrate	Test	Remarks
10.	Rod Position Bank Counters	S†	N.A.	N.A.	With Analog Rod Position
11.	Steam Generator Level	S†	R	M†	
12.	Charging Flow	N.A.	R	N.A.	
13.	Residual Heat Removal Pump Flow	N.A.	R	N.A.	
14.	Boric Acid Tank Level	W	R	N.A.	
15.	Refueling Water Storage Tank Level	W†	R	N.A.	
16.	Volume Control Tank Level	N.A.	R	N.A.	
17A.	Containment Pressure	W	R	M	Wide Range
17B.	Containment Pressure	W	R	M	Narrow Range
18A.	Process Radiation	D	A***	M	
18B.	Area Radiation	D	A	M	
19.	Boric Acid Control	N.A.	N.A.	R	
20.	Containment Sump Level	N.A.	R	N.A.	
21.	Accumulator Level and Pressure	S†	R	N.A.	
22.	Steam Line Pressure	S†	R	M†	

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TABLE 4.1-1 (Continued)

	Channel Description	Check	Calibrate	Test	Remarks
23.	Environmental Monitors	N.A.	A(1)	M(1)	(1) Flow
24.	Logic Channels	N.A.	N.A.	M†	
25.	Emer. Portable Survey Instruments	N.A.	A	M	
26.	Seismograph	N.A.	N.A.	Q	Make trace Test battery (change semi-annually)

* Using moveable in-core detector system.
** Frequency only
*** Effluent monitors only. Calibration shall be as specified in 3.9.

S - Each Shift
D - Daily
W - Weekly
B/W - Every Two Weeks
M - Monthly
Q - Quarterly
P - Prior to each startup if not done previous week
R - Each Refueling Shutdown
A - Annually
N.A. - Not applicable
† - N.A. during cold or refueling shutdowns. The specified tests, however, will be performed within one surveillance interval prior to startups.

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6.0 ADMINISTRATIVE CONTROLS

6.1 ORGANIZATION, REVIEW AND AUDIT

6.1.1 a) The Plant Manager is responsible for the safe operation of the nuclear and fossil facilities.

b) The Plant Superintendent - Nuclear is directly responsible for the safe operation of the nuclear facility.

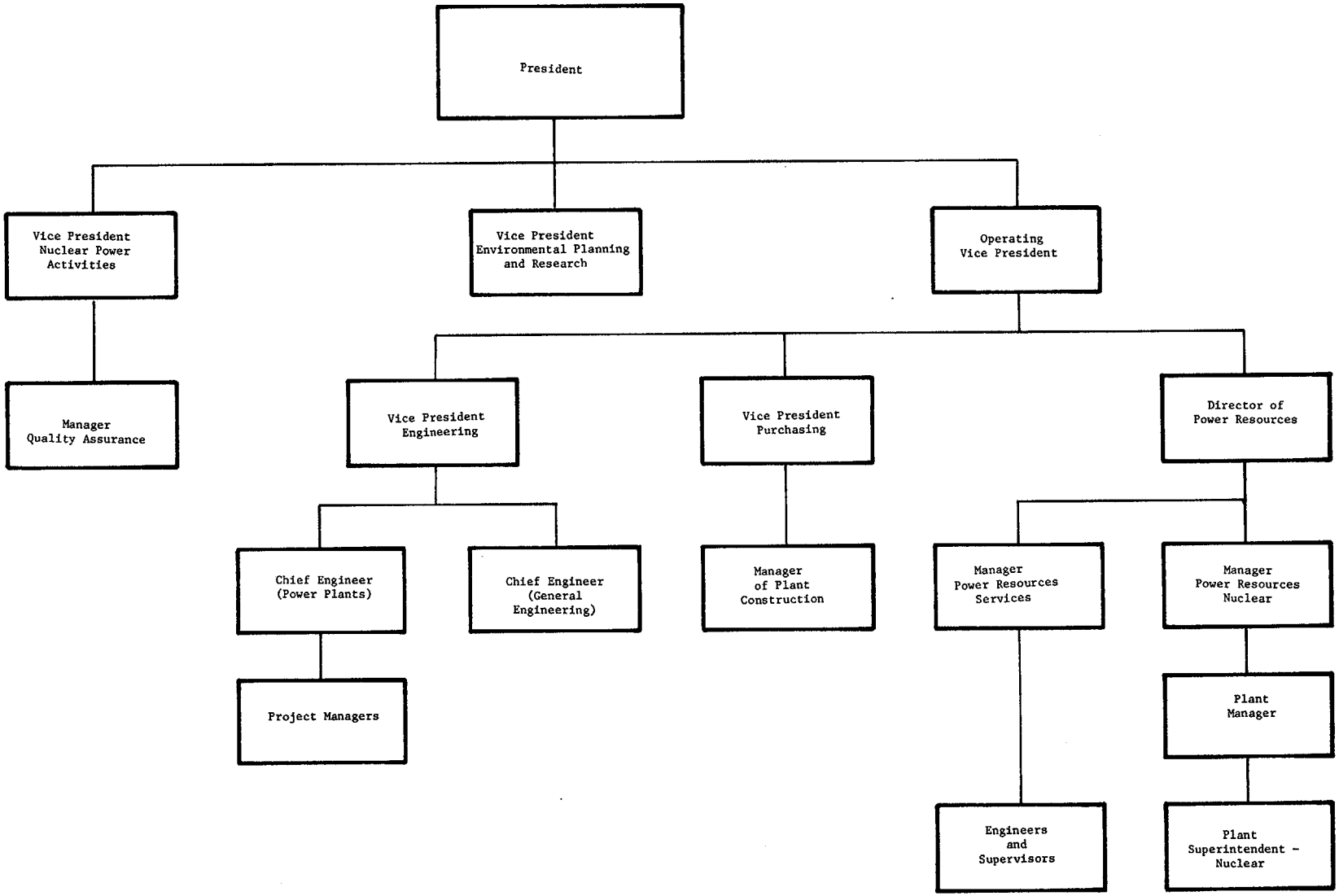
6.1.2 In all matters pertaining to operation of the nuclear units and to these Technical Specifications, the Plant Superintendent - Nuclear shall report to and be directly responsible to the Plant Manager. The Company management organization is shown in Figure 6.1-1.

6.1.3 Plant organization for conduct of operations is shown in Figure 6.1-2.

1. Minimum qualifications with regard to educational background and experience of classifications shown in Figure 6.1-2 are as follows:

   a. Plant Manager

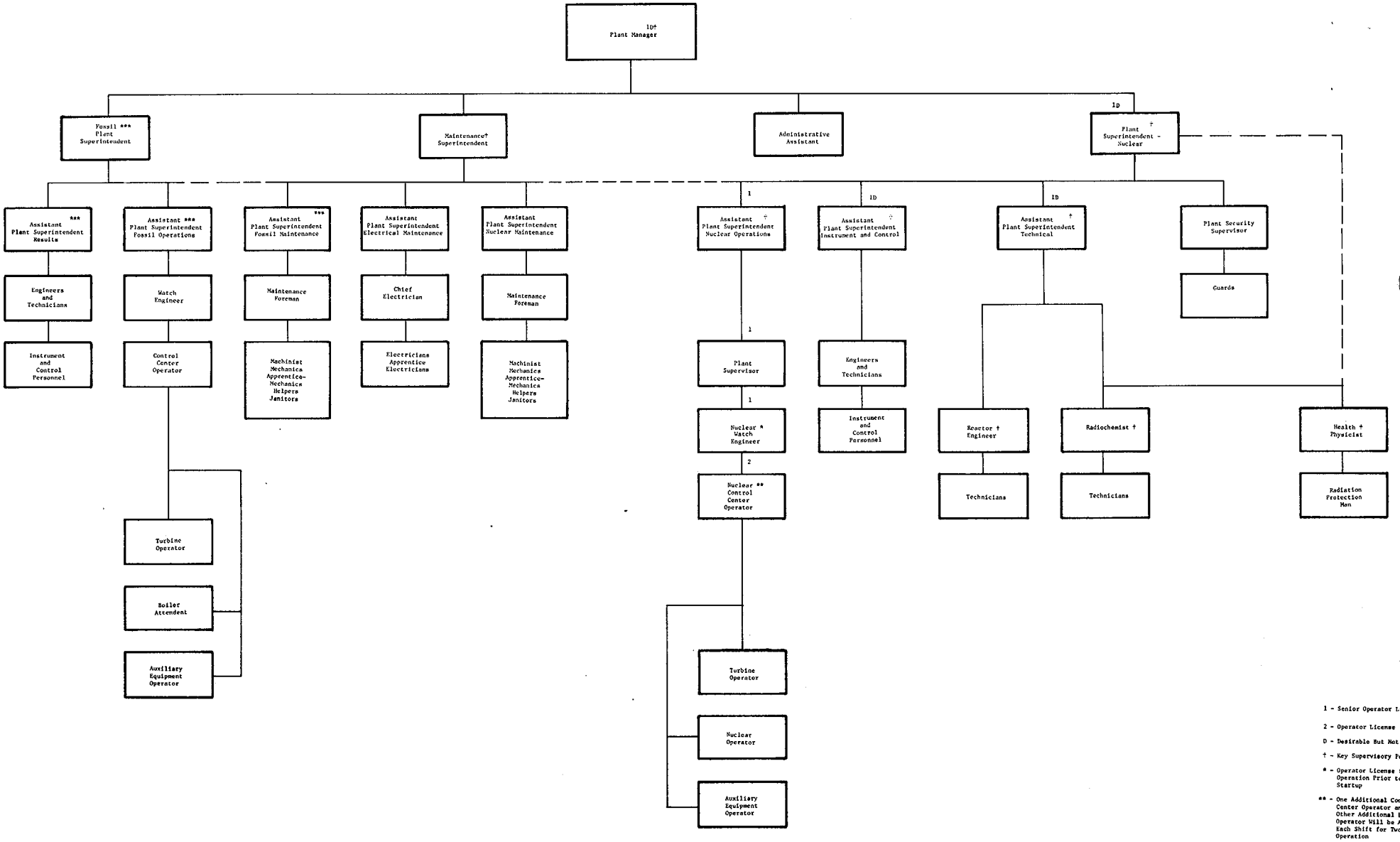
      The Plant Manager shall have ten years of responsible power plant experience of which a minimum of three years shall be nuclear power plant experience. A maximum of four years of the remaining seven years of experience may be fulfilled by academic training on a one-for-one time basis. This academic training shall be in an engineering or scientific field generally associated with power production. The Plant Manager shall have acquired the experience and training normally required for examination by the AEC for a Senior Reactor Operator's License whether or not the examination is taken. Where the Plant Superintendent - Nuclear meets the Senior Operator License requirements the Plant



MANAGEMENT ORGANIZATION CHART

FIGURE 6.1-1

3/16/73



1 - Senior Operator License

2 - Operator License

D - Desirable But Not Required

† - Key Supervisory Personnel

* - Operator License for Unit 3 Operation Prior to Unit 4 Startup

** - One Additional Control Center Operator and One Other Additional Licensed Operator Will be Added to Each Shift for Two Unit Operation

*** - Not Part of These Technical Specifications

PLANT ORGANIZATION CHART
FIG. 6.1-2

3/16/73

Manager need only have one of his ten years be nuclear plant experience and need not be eligible for AEC examination.

b. Plant Superintendent - Nuclear

The Plant Superintendent - Nuclear shall have ten years of responsible power plant experience of which a minumum of three years shall be nuclear power plant experience. A maximum of four years of the remaining seven years of experience may be fulfilled by academic training on a one-for-one time basis. This academic training shall be in an engineering or scientific field generally associated with power production The Plant Superintendent - Nuclear shall have acquired the experience and training normally required for examination by the AEC for a Senior Reactor Operator's License whether or not the examination is taken. Where the Plant Manager meets the Senior Operator License requirement the Plant Superintendent - Nuclear need only have one of his ten years be nuclear plant experience and need not be eligible for AEC examination.

c. Assistant Plant Superintendent - Nuclear Operations

The Assistant Plant Superintendent - Nuclear Operations shall have a minimum of eight years of responsible power plant experience of which a minimum of three years shall be nuclear power plant experience. A maximum of two years of the remaining five years of power plant experience may be fulfulled by satisfactory completion of academic or related technical training on a one-for-one time basis. He shall hold a Senior Reactor Operator's License.

d. Assistant Plant Superintendent - Technical

The Assistant Plant Superintendent - Technical should have a minimum of eight years in responsible positions, of which one year shall be nuclear power plant experience. A maximum of four years of the remaining seven years of experience should be fulfilled by satisfactory completion of academic training.

e. Maintenance Superintendent

The Maintenance Superintendent shall have a minimum of seven years of responsible power plant experience or applicable industrial experience, a minimum of one year of which shall be nuclear power plant experience. A maximum of two years of the remaining six years of power plant or industrial experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis. He should have non-destructive testing familiarity, craft knowledge and an understanding of electrical, pressure vessel and piping codes.

f. Assistant Plant Superintendent - Nuclear Maintenance

The Assistant Plant Superintendent - Nuclear Maintenance shall have a minimum of seven years of responsible power plant experience or applicable industrial experience, a minimum of one year of which shall be nuclear power plant experience. A maximum of two years of the remaining six years of power plant or industrial experience may be fulfilled by satisfactory completion of academic or related technical training on a one-for-one time basis. He should have non-destructive testing familiarity, craft knowledge and an understanding of pressure vessel and piping codes.

g. Radiochemist

The Radiochemist shall have a minimum of five years experience in chemistry of which a minimum of one year shall be in radiochemistry. A minimum of two years of this five years experience should be related technical training. A maximum of four years of the five years experience may be fulfilled by related technical or academic training.

h. Assistant Plant Superintendent - Instrument and Control

The Assistant Plant Superintendent - Instrument and Control shall have a minimum of five years experience in instrumentation and control, of which a minimum of six months shall be in nuclear instrumentation and control. A minimum of two years of this five years experience should be related technical training. A maximum of four years of this five years experience may be fulfilled by related technical or academic training.

i. Reactor Engineer

The Reactor Engineer shall have a minimum of a Bachelor Degree in Engineering or the Physical Sciences and two years experience in such areas as reactor physics, core measurements, core heat transfer, and core physics testing programs.

j. Health Physicist

The Health Physicist shall have a minimum of five years experience in radiation protection at a nuclear reactor facility. A minimum of two years of this five years experience should be related technical training. A maximum of four years of this five years experience may be fulfilled by related technical or academic training.

k. Plant Supervisors

The Plant Supervisors shall have a minimum of a high school diploma or equivalent and four years of responsible power plant experience of which a minimum of one year shall be nuclear power plant experience. A maximum of two years of the remaining three years of power plant experience may be fulfilled by academic or related technical training on a one-for-one time basis. He shall hold an AEC Senior Operator License.

l. Watch Engineers

The Watch Engineers shall have a high school diploma or equivalent and four years of power plant experience of which a minimum of one year shall be nuclear power plant experience. He shall hold an AEC Senior Operator License. During initial startup of the first unit, he may hold an AEC Operator License in lieu of a Senior Operator License.

m. Control Center Operators

The Control Center Operators shall have a high school diploma or equivalent and two years of power plant experience of which a minimum of one year shall be nuclear power plant experience. He shall hold an AEC Reactor Operator License.

n. Un-licensed Operators

The un-licensed operators shall have a high school diploma or equivalent and should possess a high degree of manual dexterity and mature judgment.

o. Technicians

Technicians in responsible positions shall have a minimum of two years of working experience in their specialty and should have a minimum of one year of related technical training in addition to their experience.

p. Maintenance Personnel

Maintenance personnel in responsible positions shall have a minimum of three years experience in one or more crafts. They should possess a high degree of manual dexterity and ability and should be capable of learning and applying basic skills in maintenance operations.

2. The operating organization shall be comprised and shall function as follows:

   a. The number of licensed and unlicensed operating personnel assigned to each operating shift is shown in Table 6.1-1.

   b. One licensed operator shall be in the control room at all times when there is fuel in either reactor.

   c. A licensed Senior Operator shall be on site at all times when there is fuel in either reactor.

   d. Two licensed operators shall be in the control room during start-up and shutdown of either unit and during recovery from trips caused by transients or emergencies.

   e. When there is fuel in both reactors, the Plant Supervisor shall take direct charge of the affected unit during the following operations:

      1. Startup and approach to power.
      2. Recovery from an unplanned or unscheduled shutdown or significant reduction in power.
      3. Refueling (except 1 and 2 above take precedence).

      A licensed Senior Operator shall be in direct charge of the second unit during the above operations and co-ordinate his activities with the Plant Supervisor as necessary.

   f. All licensed shift operating personnel will be required to hold applicable AEC license on both Units 3 and 4 when Unit 4 becomes operational.

g. A licensed operator will be responsible for implementing radiation protection procedures on each shift.

h. Shift operating personnel assigned to Units No. 3 & 4 other than the Plant Supervisors will not perform regularly assigned duties related to Units No. 1 & 2 operation.

3. A training program which meets the provisions of ANSI 18.1 dated March 8, 1971, shall be established.

6.1.4 Organizational units for the review and audit of plant operations shall be constituted and have the responsibilities and authorities outlined below:

1. Plant Nuclear Safety Committee

a. Membership

1. Chairman: Plant Superintendent - Nuclear
2. Vice Chairman: Assistant Plant Superintendent - Nuclear Operations
3. Assistant Plant Superintendent - Technical
4. Assistant Plant Superintendent - Nuclear Maintenance
5. Assistant Plant Superintendent - Instrument and Control
6. Health Physicist
7. Secretary: Reactor Engineer
8. Radiochemist

b. Qualifications:
The qualifications of the regular members of the Plant Nuclear Safety Committee with regard to the combined experience and technical specialties of the individual members shall be maintained at a level equal to those described in 6.1.3.

c. Consultants:

Additional personnel with expertise in specific areas such as radiochemistry, reactor engineering, and health physics may serve as consultants to the Plant Nuclear Safety Committee.

d. Meeting frequency: Monthly, and as required, on call of the Chairman.

6.2 ACTION TO BE TAKEN IN THE EVENT OF AN ABNORMAL OCCURRENCE IN A NUCLEAR UNIT

6.2.1 Any abnormal occurrence shall be reported immediately to the Manager of Power Resources - Nuclear and Director of Power Resources and promptly reviewed by the Plant Nuclear Safety Committee.

6.2.2 The Plant Nuclear Safety Committee shall prepare a separate report for each abnormal occurrence. This report shall include an evaluation of the cause of the occurrence, a record of the corrective action taken, and recommendations for appropriate action to prevent or reduce the probability of a recurrence.

6.2.3 Copies of all such reports shall be submitted to the Manager of Power Resources - Nuclear, the Director of Power Resources, and to the Chairman of the Company Nuclear Review Board for review and approval of any recommendations.

6.2.4 The Director of Power Resources shall report the circumstances of any abnormal occurrence to the AEC as specified in Section 6.6, "Plant Reporting Requirements."

6.3 ACTION TO BE TAKEN IF A SAFETY LIMIT IS EXCEEDED

6.3.1 If a safety limit is exceeded, the reactor shall be shut down and reactor operation shall only be resumed in accordance with the authorization within 10 CFR 50.36 (c) (1) (i).

6.3.2 An immediate report shall be made to the Manager of Power Resources - Nuclear, Director of Power Resources and the Chairman of the Company Nuclear Review Board.

6.3.3 The Director of Power Resources shall promptly report the circumstances to the AEC as specified in Section 6.6, "Plant Reporting Requirements."

6.3.4 A complete investigation of the occurrence including an analysis of the circumstances leading up to and resulting from the occurrence together with recommendations to prevent a recurrence shall be prepared by the Plant Nuclear Safety Committee. This report shall be submitted to the Manager of Power Resources - Nuclear, the Director of Power Resources and the Chairman of the Company Nuclear Review Board. Appropriate analyses or reports will be submitted to the AEC by the Director of Power Resources as specified in Section 6.6, "Plant Reporting Requirements."

encountered, such equipment is capable of providing a degree of protection at least equal to the protection factors listed in Table 6.4-1 and has been approved by the Bureau of Mines for those concentrations and intended use.

6. Respiratory protective equipment shall be selected and used in such a manner that peak concentrations of airborne radioactive material inhaled by an individual wearing the equipment do not exceed the pertinent values specified in Appendix B, Table I of 10CFR, Part 20.

7. Protection factors shall not be assigned in excess of those listed in Table 6.4-1.

8. If, in the future, 10CFR20, Section 103, shall assign protection factors for respiratory and other protective equipment, the provisions of paragraph 6.4.2.c, shall be superseded by the provisions of 10CFR20, Section 103.

6.4.3 All procedures described in 6.4.1 above, and changes thereto, shall be reviewed by the Plant Nuclear Safety Committee and approved by the Plant Superintendent - Nuclear prior to implementation, except as provided in 6.4.4 below.

6.4.4 Temporary changes to procedures in 6.4.1 above, which do not change the intent of the original procedure may be made, provided such changes are approved by two members of the plant management staff, at least one of whom shall hold a Senior Reactor Operator License. Such changes shall be documented and subsequently reviewed by the Plant Nuclear Safety Committee and approved by the Plant Superintendent - Nuclear.

6.4.5 Practice of site evacuation exercises shall be conducted annually following emergency procedures and including a check of communications with off-site support groups. Notification lists and rosters shall be continually updated. The Emergency Plan and implementing procedures shall be reviewed and updated at least annually.

6.4.6 An industrial security program shall be maintained throughout the life of the plant in accordance with the provisions of the Plant Security Plan. Annual review of the Plant Security Plan will be performed.

6.4.7 Investigations of all attempted or actual security infractions shall be conducted by the Plant Security Supervisor, in cooperation with any Federal, State, or Local agencies involved, and a report filed with the Director of Power Resources, Manager of Power Resources - Nuclear, Plant Superintendent - Nuclear and Chairman of the Company Nuclear Review Board.

6.4.8 Any actual or attempted introduction into the Generating Station Area of any dangerous weapon, explosive or material capable of producing injury or damage to persons or property, or that in any way could seriously affect the safe operation of the generating units, shall be reported immediately upon detection to the Captain of the Guard.

6.4.9 Drills for portions of emergency procedures described in 6.4.1 subsections d and i shall be conducted semiannually.