

MAY 30 1975

Dockets Nos. 50-250
and 50-251

Florida Power and Light Company
ATTN: Dr. Robert E. Uhrig
Vice President
P. O. Box 013100
Miami, Florida 33101

Gentlemen:

The Commission has issued the enclosed Amendment No. 7 to Facility Operating License No. DPR-31 and Amendment No. 6 to Facility Operating License No. DPR-41 for Turkey Point Nuclear Generating Units 3 and 4. These amendments include Change No. 19 to the joint Technical Specifications and are in response to your request dated September 20, 1974, and staff discussions.

These amendments provide an alternate method for testing the operation of the circuit breakers associated with the residual heat removal pump motors during the periodic testing of the Safety Injection System. The other changes proposed in your September 20, 1974 request are still under review.

Copies of the related Safety Evaluation and the Federal Register Notice are also enclosed.

Sincerely,

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Enclosures:

1. Amendment No. 7
2. Amendment No. 6
3. Safety Evaluation
4. Federal Register Notice

cc: See next page

OFFICE	ORB#3	ORB#3	ORB#3 XXXXXXXX	AD:DRL/ORs	OELD	ORB#3
SURNAME	RIngram/dg	JGuibert/dg	GLear <i>GL</i>	KGoller <i>GL</i>	WCHANDLER	GLear
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Florida Power & Light Company

MAY 30 1975

cc: w/enclosure

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Environmental & Urban Affairs Library
Florida International University
Miami, Florida 33199

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-250

TURKEY POINT NUCLEAR GENERATING UNIT 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 7
License No. DPR-31

1. The Nuclear Regulatory Commission (the Commission) has found that:

- A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
- B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
- C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
- D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-31 is hereby amended to read as follows:

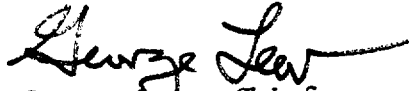
" B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 19."



3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "George Dear", with a long horizontal flourish extending to the right.

George Dear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 19 to
Technical Specifications

Date of Issuance: MAY 30 1975

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

FLORIDA POWER AND LIGHT COMPANY

DOCKET NO. 50-251

TURKEY POINT NUCLEAR GENERATING UNIT 4

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. DPR-41

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Florida Power and Light Company (the licensee) dated September 20, 1974, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations; and
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.
2. Accordingly, the license is amended by a change to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 3.B of Facility License No. DPR-41 is hereby amended to read as follows:

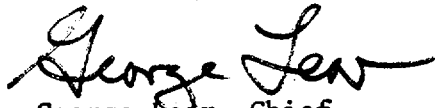
" B. Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, as revised by issued changes thereto through Change No. 19."



3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "George Lear". The signature is fluid and cursive, with the first name "George" and last name "Lear" clearly distinguishable.

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing

Attachment:
Change No. 19 to
Technical Specifications

Date of Issuance: MAY 30 1975

ATTACHMENT TO LICENSE AMENDMENTS NOS. 7 AND 6
CHANGE NO. 19 TO THE TECHNICAL SPECIFICATIONS
FACILITY OPERATING LICENSES NOS. DPR-31 AND DPR-41
DOCKETS NOS. 50-250 AND 50-251

Revise Appendix A by deleting page 4.5-1 and inserting the attached revised page bearing the same number. The changed area on the revised page is shown by a marginal line.

Applicability: Applies to testing of the Safety Injection System.

Objective: To verify that the subject systems will respond promptly and perform their design functions.

Specifications: 1. SYSTEM TESTS

- a. System tests shall be performed at each refueling shutdown. The test shall be performed in accordance with the following procedure:

With the Reactor Coolant System pressure equal to or less than 350 psig and temperature equal to or less than 350F, a test safety injection signal will be applied to initiate operation of the system. The breakers for the residual heat removal pump motors will be tested either in the test position or by actual residual heat removal pump motor operation resulting from the test safety injection signal. 19

- b. The test will be considered satisfactory if control panel indication and visual observations indicate that all components have received the safety injection signal in the proper sequence and timing, appropriate breakers shall open and close, and all automatic valves shall complete their travel.

2. COMPONENT TESTS

a. Pumps

1. The safety injection pumps and residual heat removal pumps shall be started at intervals not greater than one month.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 7 TO LICENSE NO. DPR-31 AND
AMENDMENT NO. 6 TO LICENSE NO. DPR-41

(CHANGE NO. 19 TO TECHNICAL SPECIFICATIONS)

FLORIDA POWER & LIGHT COMPANY

TURKEY POINT NUCLEAR GENERATING UNITS 3 & 4

Introduction

By letter dated September 20, 1974, Florida Power & Light Company (the licensee) requested a change to the Technical Specifications, subsection 4.5, specification 1.a, for Turkey Point Nuclear Generating Units 3 & 4. (The wording of the proposed change was modified with mutual concurrence between the licensee and the NRC staff on April 30, 1975.) The proposed change would provide an alternate method for testing the operation of the circuit breakers associated with the Residual Heat Removal (RHR) pump motors during the periodic testing of the Safety Injection System.

Discussion

Technical Specification 4.5.1.a for Turkey Point Units 3 & 4 requires that a test of the Safety Injection System (SIS) be performed during each refueling shutdown to verify that the SIS will respond promptly and perform its design function. The present procedure for the conduct of the test is described in Specification 4.5.1.a:

"With the Reactor Coolant System pressure equal to or less than 350 psig and temperature equal to or less than 350° F, a test safety injection signal will be applied to initiate operation of the system. The breakers for the residual heat removal pump motors will be in the test position for this test."

The criteria for evaluating the results of the test are specified in Specification 4.5.1.b:

"The test will be considered satisfactory if control panel indication and visual observations indicate that all components have received the safety injection signal in the proper sequence and timing, appropriate breakers shall open and close, and all automatic valves shall complete their travel."



The design of the SIS provides for the automatic actuation of the Residual Heat Removal pumps upon their receipt of a safety injection actuation signal. In accordance with the procedure for conducting the SIS test as stated above, the circuit breakers for the RHR pump motors are required to be in the test position. With this procedure in effect, verification of a proper response of the RHR pump motor breakers to an SIS actuation signal is achieved when the circuit breakers close. With the circuit breakers in the test position, power is not provided to the pump motors when the breakers are closed. This procedure permits testing and verification of the SIS actuation signal to the RHR pump breakers even though the RHR pumps may be inoperable due to maintenance or other conditions.

The licensee proposes that Technical Specification 4.5.1.a be changed to read:

"System tests shall be performed at each refueling shutdown. The test shall be performed in accordance with the following procedure:

With the Reactor Coolant System pressure equal to or less than 350 psig and temperature equal to or less than 350° F, a test safety injection signal will be applied to initiate operation of the system. The breakers for the residual heat removal pump motors will be tested either in the test position or by actual residual heat removal pump motor operation resulting from the test safety injection signal."

This proposed change would provide an alternate method for testing the RHR pump motor breakers by placing the circuit breakers in the normal position prior to insertion of the test safety injection signal. Verification of a proper response is obtained when the circuit breakers close and the associated RHR pump motors operate.

Evaluation

The alternate method of testing RHR pump motor circuit breakers proposed by the licensee allows for the verification of breaker response to an SIS actuation signal. This is accomplished with no degradation of the validity or extent of the existing acceptable test method. In addition, during those plant conditions, when the RHR pumps are capable of being operated, which is the normal condition, the alternate method provides a more comprehensive check by requiring actual pump motor operation.

We conclude that the alternate test method is acceptable because it does not reduce the requirements of the presently specified test procedure and therefore, does not reduce the margin of safety or increase the risk or consequences of previously analyzed accidents.

Conclusion

We have concluded, based on the considerations discussed above, that:
(1) because the change does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the change does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: MAY 30 1975

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKETS NOS. 50-250 AND 50-251

FLORIDA POWER AND LIGHT COMPANY

NOTICE OF ISSUANCE OF AMENDMENTS TO FACILITY
OPERATING LICENSES

Notice is hereby given that the U. S. Nuclear Regulatory Commission (the Commission) has issued Amendments Nos. 7 and 6 respectively to Facility Operating Licenses Nos. DPR-31 and DPR-41 to Florida Power and Light Company which revised Technical Specifications for operation of the Turkey Point Nuclear Generating Units 3 & 4, located in Dade County, Florida. The amendments are effective as of the date of issuance.

These amendments provide an alternate method for testing the operation of the circuit breakers associated with the residual heat removal pump motors during the periodic testing of the Safety Injection System.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments is not required since the amendments do not involve a significant hazards consideration.

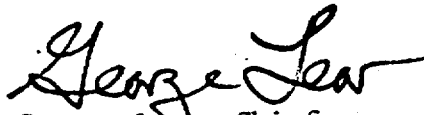
For further details with respect to these actions, see (1) the application for amendments dated September 20, 1974, (2) Amendment No. 7 to License No. DPR-31 and Amendment No. 6 to License No. DPR-41 with Change No. 19 and (3) the Commission's related Safety Evaluation. All

of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N. W., Washington, D. C. and after May 23, 1975 at the Environmental & Urban Affairs Library, Florida International University, Miami, Florida.

A copy of items (2) and (3) may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Reactor Licensing.

Dated at Bethesda, Maryland this MAY 30 1975

FOR THE NUCLEAR REGULATORY COMMISSION

A handwritten signature in dark ink, appearing to read "George Lear". The signature is fluid and cursive, with the first name "George" and last name "Lear" clearly distinguishable.

George Lear, Chief
Operating Reactors Branch #3
Division of Reactor Licensing