

September 15, 1981

Docket No. 50-328

Mr. H. G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 2 to Facility Operating License DPR-79 - Sequoyah Nuclear Plant, Unit 2

The U. S. Nuclear Regulatory Commission has issued the enclosed Amendment No. 2 to Facility Operating License No. DPR-79 to the Tennessee Valley Authority for the Sequoyah Nuclear Plant, Unit 2, located in Hamilton County, Tennessee. Amendment No. 2 authorizes operation of the Sequoyah Nuclear Plant, Unit 2, at 100 percent power (3411 megawatts thermal) upon completion of the construction item identified in Attachment 1 to the amended license.

Also enclosed is a copy of a related Federal Register Notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Darrell G. Eisenhower, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

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Enclosures:

- 1. Amendment No. 2 to DPR-79
- 2. Federal Register Notice

cc w/enclosures:  
See next page

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TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

FACILITY OPERATING LICENSE

License No. DPR-79  
Amendment No. 2

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
  - A. The application for license filed by the Tennessee Valley Authority (the licensee or TVA) complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I, and all required notifications to other agencies or bodies have been duly made;
  - B. Construction of the Sequoyah Nuclear Plant, Unit 2 (the facility), has been substantially completed in conformity with Provisional Construction Permit No. CPPR-73 and the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - C. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission;
  - D. There is reasonable assurance: (i) that the activities authorized by this amended operating license can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission set forth in 10 CFR Chapter I;
  - E. The Tennessee Valley Authority is technically and financially qualified to engage in the activities authorized by this amended operating license in accordance with the Commission's regulations set forth in 10 CFR Chapter I;
  - F. The Tennessee Valley Authority has satisfied the applicable provisions of 10 CFR Part 140, "Financial Protection Requirements and Indemnity Agreements", of the Commission's regulations;
  - G. The issuance of this amended license will not be inimical to the common defense and security or to the health and safety of the public;

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- H. After weighing the environmental, economic, technical, and other benefits of the facility against environmental and other costs and considering available alternatives, the issuance of Amendment 2, subject to the conditions for protection of the environment set forth herein, is in accordance with 10 CFR Part 50, Appendix D\*, of the Commission's regulations and all applicable requirements have been satisfied; and
  - I. The receipt, possession, and use of source, byproduct, and special nuclear material as authorized by this amended license will be in accordance with the Commission's regulations in 10 CFR Parts 30, 40 and 70.
2. Pursuant to approval by the Nuclear Regulatory Commission at a meeting on September 8, 1981, Amendment No. 2 amends Facility Operating License No. DPR-79 issued to the Tennessee Valley Authority in its entirety to read as follows:
- A. This amended license applies to the Sequoyah Nuclear Plant, Unit 2, a pressurized water nuclear reactor and associated equipment (the facility), owned by the Tennessee Valley Authority. The facility is located in Hamilton County, Tennessee, about 9.5 miles northeast of Chattanooga, and is described in TVA's Final Safety Analysis Report as supplemented and amended, and the Final Environmental Statement prepared by the Tennessee Valley Authority.
  - B. Subject to the conditions and requirements incorporated herein, the Commission hereby licenses the Tennessee Valley Authority:
    - (1) Pursuant to Section 104(b) of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities", to possess, use, and operate the facility at the designated location in Hamilton County, Tennessee, in accordance with the procedures and limitations set forth in this license;
    - (2) Pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
    - (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
  - (5) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This amended license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3411 megawatts thermal. Operation of the facility is subject to compliance with the construction item listed in Attachment 1.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B attached hereto are hereby incorporated in this license. The Tennessee Valley Authority shall operate the facility in accordance with the Technical Specifications.

(3) Initial Test Program

The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report, as amended, as being essential;
- b. Modification of test objectives, methods, or acceptance criteria for any test identified in Section 14 of TVA's Final Safety Analysis Report, as amended, as being essential;
- c. Performance of any test at a power level different from that described in the program; and

- d. Failure to complete any tests included in the described program (planned or scheduled) for power levels up to the authorized power level.

(4) Monitoring Settlement Markers (SER/SSER Section 2.6.3)

TVA shall continue to monitor the settlement markers along the ERCW conduit for the new ERCW intake structure for a period not less than three years from the date of this license. Any settlement greater than 0.5 inches that occurs during this period will be evaluated by TVA and a report on this matter will be submitted to the NRC.

(5) Tornado Missiles (Section 3.5)

Prior to startup after the first refueling of the facility, TVA shall reconfirm to the satisfaction of the NRC that adequate tornado protection is provided for the 480 V transformer ventilation systems.

(6) Design of Seismic Category Structures (Section 3.8)

Prior to startup following the first refueling, TVA shall evaluate all seismic Category I masonry walls to final NRC criteria and implement NRC required modifications that are indicated by that evaluation.

(7) Low Temperature Overpressure Protection (Section 5.2.2)

Prior to startup after the first refueling, TVA shall install an overpressure mitigation system which meets NRC requirements.

(8) Steam Generator Inspection (Section 5.3.1)

Prior to start-up after the first refueling, TVA shall install inspection ports in each steam generator or have an alternative for inspection that is acceptable to the NRC.

(9) Containment Isolation Systems (Section 6.2.4)

Prior to startup after the first refueling, TVA shall modify to the satisfaction of the NRC the one-inch chemical feed lines to the main and auxiliary feedwater lines for compliance with GDC 57.

(10) Environmental Qualification (Section 7.2.2)

- a. No later than June 30, 1982, TVA shall be in compliance with the requirements of NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electrical Equipment," for safety-related equipment exposed to a harsh environment.

- b. Complete and auditable records must be available and maintained at a central location which describe the environmental qualification method used for all safety-related electrical equipment in sufficient detail to document the degree of compliance with the DOR Guidelines or NUREG-0588. Such records should be updated and maintained current as equipment is replaced, further tested, or otherwise further qualified to document complete compliance by June 30, 1982.
- c. Within 90 days of receipt of the equipment qualification safety evaluation, the licensee shall either (i) provide missing documentation identified in Sections 3 and 4 of the equipment qualification safety evaluation which will demonstrate compliance of the applicable equipment with NUREG-0588, or (ii) commit to corrective actions which will result in documentation of compliance of applicable equipment with NUREG-0588 no later than June 30, 1982.

(11) Requirements For Modification To Or Addition Of Instrumentation And Controls (Section 7.3.2)

- a. Prior to startup after first refueling, TVA shall have installed instrument downscale failure alarms for the effluent monitoring instrumentation channels for radioactive gaseous and radioactive liquid effluents. Modifications to procedures and Technical Specifications 3.3.3.9 and 3.3.3.10 shall have been completed.
- b. Prior to startup after the first refueling, TVA shall have received NRC approval of an additional operable level of over/undervoltage protection including associated Technical Specifications. The level of protection from the effects of power transients on safety-related equipment provided by Part I of the staff's "Degraded Grid Voltage Position", or equivalent, is required.

(12) Diesel Generator Reliability (Section 8.3.1)

Prior to operation following the first refueling, TVA shall implement the following design and procedure modifications as outlined in Section 8.3.1 of SER Supplement No. 2. These include: (a) Moisture in Air Starting System; (b) Turbocharger Gear Drive Problem; and (c) Personnel Training.

(13) Fire Protection System (Section 9.5)

- a. TVA shall maintain in effect and fully implement all provisions of the approved fire protection plan and the NRC staff's Fire Protection Review in the Sequoyah Safety Evaluation Report and Supplements.
- b. TVA shall replace the control room ceiling panels with panels acceptable to NRC by September 30, 1981.
- c. TVA shall comply with Section III.G, III.J, III.L and III.O of Appendix R of 10 CFR 50, except where NRC has approved deviations, on a schedule consistent with that required for other operating reactors. By October 1, 1981, TVA shall submit a report that identifies and justifies differences between existing or proposed fire protection features and those features specified in Section III.G, III.J, III.L and III.O of Appendix R to 10 CFR Part 50.

(14) Compliance With Regulatory Guide 1.97

TVA shall implement modifications necessary to comply with Revision 2 of Regulatory Guide 1.97, "Instrumentation for Light Water Cooled Nuclear Power Plants to Assess Plant Conditions During and Following an Accident," dated December 1980, by June 1983.

(15) Corrosion of Carbon Steel Piping

TVA shall carry out a surveillance program on corrosion of carbon steel piping in accordance with TVA document SQRD-50-328/81-10 dated August 25, 1981, and procedures for implementation are to be submitted for NRC concurrence by October 15, 1981.

(16) NUREG-0737 Conditions (Section 22.2)

Each of the following conditions shall also be performed to the satisfaction of the NRC:

a. Shift Technical Advisor (Section 22.2, I.A.1.1)

TVA shall provide a fully-trained on-shift technical advisor to the shift supervisor.

b. Independent Safety Engineering Group (Section 22.2, I.B.1.2)

TVA shall have an onsite Independent Safety Engineering Group.

c. Procedures for Verifying Correct Performance of Operating Activities (Section 22.2, I.C.6)

Procedures shall be available to verify the adequacy of the operating activities.

d. Control Room Design (Section 22.2, I.D.1)

Prior to startup after first refueling of Unit 1 or Unit 2, whichever occurs first, TVA shall complete the detailed Control Room Design Review. As part of this review, TVA shall consider benefits of installing data recording and logging equipment in the control room to correct the deficiencies associated with the trending of important parameters on strip chart recorders used in the control room.

e. Training During Low-Power Testing (Section 22.2, I.G.1)

One experienced operator trained on Unit 1 low power testing for natural circulation operation shall be assigned to each shift on this facility. This requirement shall remain until TVA submits a report, and NRC agrees with findings, that an acceptable level of training and experience on Unit 2 has been attained.

f. Reactor Coolant System Vents (Section 22.2, II.B.1)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor coolant system and reactor vessel head highpoint vents that are remotely operable from the control room.

g. Post Accident Sampling (Section 22.2, II.B.3)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall complete corrective actions needed to provide the capability to promptly obtain and perform radioisotopic and chemical analyses of reactor coolant and containment atmosphere samples under degraded core conditions without excessive exposure.

h. Hydrogen Control Measures (Section 22.2, II.B.7)

(1) Prior to startup following the first refueling outage, the Commission must confirm that an adequate hydrogen control system for the plant is installed and will perform its intended function in a manner that provides adequate safety margins.



(2) During the interim period of operation, TVA shall continue a research program on hydrogen control measures and the effects of hydrogen burns on safety functions and shall submit to the NRC quarterly reports on that research program.

(a) TVA shall amend its research program on hydrogen control measures to include, but not be limited to, the following items:

- 1) Improved calculational methods for containment temperature and ice condenser response to hydrogen combustion.
- 2) Research to address the potential for local detonation.
- 3) Confirmatory tests on selected equipment exposed to hydrogen burns.
- 4) New calculations to predict differences between expected equipment temperature environments and containment temperatures.
- 5) Evaluate and resolve any anomalous results occurring during the course of its ongoing test program.

(b) A schedule for confirmatory tests shall be provided by TVA consistent with the requirement to meet Section (16)h.(2) of the license.

i. Relief and Safety Valve Test Requirements (Section 22.2, II.D.1)

TVA shall conform to the results of the EPRI test program. TVA shall provide documentation for qualifying (a) reactor coolant system relief and safety valves, (b) piping and supports, and (c) block valves in accordance with the review schedule given in SECY 81-491 as approved by the Commission.

j. Auxiliary Feedwater (Section 22.2, II.E.1.1)

Prior to exceeding five percent power, auxiliary feedwater pump endurance tests will be completed. A report shall be submitted to NRC within 30 days after all tests are completed.

k. Containment Isolation Dependability (Section 22.2, II.E.4.2)

TVA shall limit the purge valve openings to less than or equal to 50 degrees.

l. Additional Accident Monitoring Instrumentation (Section 22.2, II.F.1)

- (1) TVA shall install interim noble gas monitors at the first outage of sufficient duration.
- (2) At the first outage of sufficient duration, but no later than startup following the first refueling outage, TVA shall install the following qualified monitoring instrumentation:
  - (a) Integrated monitoring assembly which will accomplish particulate, iodine and noble gas monitoring.
  - (b) Containment high range radiation monitor.
  - (c) Containment pressure monitor.
  - (d) Containment water level monitor.
  - (e) Containment hydrogen monitor.

m. Instruments for Inadequate Core Cooling (Section 22.2, II.F.2)

- (1) By January 1, 1982, TVA shall install a backup indication for incore thermocouples. This display shall be in the control room and cover the temperature range of 200 F - 2000 F.
- (2) At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor vessel water level instrumentation and the system will meet seismic and environmental requirements.

n. Voiding in Reactor Coolant System (Section 22.2, II.K.2.17)

TVA is participating in the Westinghouse owners group effort on this item and shall conform to the results of this effort. The analysis will be submitted to NRC by January 1, 1982.

o. Sequential Auxiliary Feedwater Flow Analysis (Section 22.2, II.K.2.19)

TVA is participating in the Westinghouse owners group effort on this item and shall conform to the results of this effort. The analysis will be submitted to NRC by July 1, 1982.

p. Calculations for Small-Break LOCAs (Section 22.2, II.K.3.30 and II.K.3.31)

TVA is participating in the Westinghouse owners group effort for this item and shall conform to the results of this effort. The analysis for model justification will be submitted to NRC by January 1, 1982.

q. Upgrade Emergency Support Facilities (Section 22.2, III.A.1.2)

(1) The installation of the TSC shall be completed prior to startup after the first refueling. However, if an outage scheduled to last more than five weeks occurs after May 1, 1982, installation of the necessary modifications to the control room and plant instrumentation will be completed at that time and the TSC hardware installation will be completed within eleven weeks of the start of this scheduled outage.

(2) TVA shall maintain interim emergency support facilities (Technical Support Center, Operations Support Center and the Emergency Operations Facility) until the final facilities are complete.

r. Long-Term Emergency Preparedness (Section 22.2, III.A.2)

Additional implementation dates for the meteorological program are:

(1) Functional description of upgraded capabilities shall be provided by January 1, 1982.

(2) Installation of hardware and software shall be completed by July 1, 1982.

(3) Full operational capability is required by October 1, 1982.

s. Primary Coolant Outside Containment (Section 22.2, III.D.1.1)

Prior to exceeding 5 percent power level, TVA is required to complete the leak tests on Unit 2, and results are to be submitted within 30 days from the completion of the testing.

- D. Exemptions from certain requirements of Appendices G and J to 10 CFR Part 50 are described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report, Supplements No. 1 and No. 5. These exemptions are authorized by law and will not endanger life or property or the common defense and security and are otherwise in the public interest. Therefore, these exemptions are hereby granted. The facility will operate, to the extent authorized herein, in conformity with the application, as amended, the provisions of the Act, and the regulations of the Commission.

A temporary exemption from General Design Criterion 57 found in Appendix A to 10 CFR Part 50 is described in the Office of Nuclear Reactor Regulation's Safety Evaluation Report, Supplement No. 5, Section 6.2.4. This exemption is authorized by law and will not endanger life or property or the common defense and security and is otherwise in the public interest. The exemption, therefore, is hereby granted and shall remain in effect through the first refueling outage as discussed in Section 6.2.4 of Supplement 5 to the Safety Evaluation Report. The granting of the exemption is authorized with the issuance of the Facility Operating License. The facility will operate, to the extent authorized herein, in conformity with the application as amended, the provisions of the Act, and the regulations of the Commission.

- E. The licensee shall maintain in effect and fully implement all provisions of the Commission-approved physical security plan, guard training and qualification plan, and safeguards contingency plan, including amendments made pursuant to the authority of 10 CFR §50.54(p). The approved plans consist of 10 CFR §2.790(d) information collectively entitled: "Physical Security Plan for the Sequoyah Nuclear Plant", dated August 25, 1978, as revised on April 2, 1979, June 29, 1979, September 19, 1979, and December 16, 1980; "Sequoyah Nuclear Plant Security Personnel Training and Qualifications Plan" dated August 17, 1979, as revised January 24, 1980, May 21, 1980, October 1, 1980, March 9, 1981, and as amended by subsequent approved revisions; and the "Sequoyah Nuclear Plant Safeguards Contingency Plan", dated March 1, 1979, as revised September 1, 1979, April 15, 1980, December 21, 1980, March 30, 1981, and as amended by subsequent approved revisions.

In addition to all other commitments contained in the physical security plan, the licensee shall ensure that whenever an employee leaves the employment of TVA, all locks, keys, combinations, card keys, or related equipment used to control access to the Sequoyah Nuclear Plant protected area or vital areas to which that employee had access shall be changed.

F. Reactor Safety Methodology Applications Programs (Section 24.0)

TVA will provide a report prepared by the Kaman Sciences Corporation (KSC) on a full scale nuclear safety and availability analysis within six months from the date of the KSC report.

- G. This amended license is subject to the following additional condition for the protection of the environment:

Before engaging in additional construction or operational activities which may result in an environmental impact that was not evaluated by the Commission, Tennessee Valley Authority will prepare and record an environmental evaluation of such activity. When the evaluation indicates that such activity may result in a significant adverse environmental impact that was not evaluated, or that is significantly greater than that evaluated in the Final Environmental Statement prepared by the Tennessee Valley Authority and the Environmental Impact Appraisal prepared by the Commission in May 1979, the Tennessee Valley Authority shall provide a written evaluation of such activities and obtain prior approval from the Director, Office of Nuclear Reactor Regulation.

- H. TVA shall report any violations of the requirements contained in Sections 2.C(3) through 2.C(16), 2.E, 2.F, and 2.G of this license within 24 hours by telephone and confirmed by telegram, mailgram, or facsimile transmission to the Director of the Regional Office, or his designee, no later than the first working day following the violation with a written followup report within 14 days.
- I. TVA shall immediately notify the Commission of any accident at this facility which could result in an unplanned release of quantities of fission products in excess of allowable limits for normal operation established by the Commission.
- J. TVA shall have and maintain financial protection of such type and in such amounts as the Commission shall require in accordance with Section 170 of the Atomic Energy Act of 1954, as amended, to cover public liability claims.



K. This amended license is effective as of the date of issuance and shall expire May 27, 2010.

FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed by  
H. R. Denton

Harold R. Denton, Director  
Office of Nuclear Reactor Regulation

Attachments:

- 1. Attachment 1
- 2. Appendices A and B Technical Specifications

Date of Issuance: September 15, 1981

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ATTACHMENT 1

CONSTRUCTION ITEM TO BE COMPLETED

Prior to initial criticality, TVA shall complete applicable preoperational testing and resolve significant test deficiencies. TVA shall not proceed to initial criticality without NRC authorization.

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-328

TENNESSEE VALLEY AUTHORITY

NOTICE OF ISSUANCE OF AMENDMENT TO FACILITY OPERATING LICENSE

On June 25, 1981, the U. S. Nuclear Regulatory Commission (the Commission) issued Facility Operating License No. DPR-79, to Tennessee Valley Authority (licensee) authorizing operation of the Sequoyah Nuclear Plant, Unit 2 (the facility), at reactor core power levels not in excess of 170 megawatts thermal (5 percent power) in accordance with the provisions of the license and the Technical Specifications.

The Commission has now issued Amendment No. 2 to Facility Operating License No. DPR-79, which authorizes operation of the Sequoyah Nuclear Plant, Unit 2, at reactor core power levels not in excess of 3411 megawatts thermal (100 percent power) in accordance with the provisions of the amended license and the Technical Specifications.

The Sequoyah Nuclear Plant, Unit 2, is a pressurized water nuclear reactor located at the licensee's site in Hamilton County, Tennessee, about 9.5 miles northeast of Chattanooga. The amendment is effective as of the date of issuance.

~~As noted in the FEDERAL REGISTER (46 FR 34443), for the 5 percent license,~~

The application for the license complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the amended license. Prior public notice of the overall action involving the proposed issuance of an operating license was published in the FEDERAL REGISTER on March 25, 1974 (39 FR 11131).

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The Commission has determined that the issuance of this amendment will not result in any environmental impacts other than those evaluated in the Final Environmental Statement prepared by Tennessee Valley Authority and the Environmental Impact Appraisal prepared by the Commission because the activity authorized by the amended license is encompassed by the overall action evaluated in these documents.

For further details with respect to this action, see (1) Amendment No. 2 to Facility Operating License No. DPR-79, complete with Technical Specifications; (2) Facility Operating License No. DPR-79 dated June 25, 1981, authorizing five percent power; (3) the reports of the Advisory Committee on Reactor Safeguards dated December 11, 1979, July 15, 1980, September 8, 1980, and January 31, 1981; (4) the Commission's Safety Evaluation Report (NUREG-0011) dated March 1979, Supplement No. 1 dated February 1980, Supplement No. 2 dated August, 1980, Supplement No. 3 dated September 1980, Supplement No. 4 dated January 1981, and Supplement No. 5 dated June 1981; (5) the Final Safety Analysis Report and amendments thereto; (6) the Final Environmental Statement prepared by Tennessee Valley Authority in July 1974; (7) the Commission's Environmental Impact Appraisal dated July 18, 1980; and (9) Discussion of the Environmental Effects of the Uranium Fuel Cycle dated September 3, 1980.

These items are available for public inspection at the Commission's public Document Room, 1717 H Street, N.W., Washington, D. C., and the Chattanooga Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37402. A copy of Amendment No. 2 to Facility Operating License No. DPR-79 may

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be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing. A copy of item (4) may be purchased at current rates from the National Technical Information Service, Department of Commerce, 5285 Port Royal Road, Springfield, Virginia 22161, and through the NRC GPO sales program by writing to U. S. Nuclear Regulatory Commission, Attention: Sales Manager, Washington, D. C. 20555. GPO deposit account holders can call (301) 492-9530.

Dated at Bethesda, Maryland, this 15<sup>th</sup> of September 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

*sl*

Elinor G. Adensam, Acting Chief  
Licensing Branch No. 4  
Division of Licensing

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AMENDMENT NO. 2 TO FACILITY OPERATING LICENSE - SEQUOYAH NUCLEAR PLANT, UNIT 2

DISTRIBUTION:

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