

September 15, 1995

Mr. Oliver D. Kingsley, Jr.
President, TVA Nuclear and
Chief Nuclear Officer
Tennessee Valley Authority
6A Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

SUBJECT: ISSUANCE OF TECHNICAL SPECIFICATION AMENDMENTS FOR THE SEQUOYAH
NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M91941 AND M91942) (TS 95-11)

Dear Mr. Kingsley:

The Commission has issued the enclosed Amendment No.211 to Facility Operating License No. DPR-77 and Amendment No.201 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated August 7, 1995, which superseded your application dated April 6, 1995.

The amendments revise the time constant used in the overtemperature delta temperature and the overpower delta temperature trip equations of Technical Specification Table 2.2-1.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

David E. LaBarge, Sr. Project Manager
Project Directorate II-3
Division of Reactor Projects - I/I
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

- Enclosures: 1. Amendment No. 211 to License No. DPR-77
- 2. Amendment No. 201 to License No. DPR-79
- 3. Safety Evaluation

cc w/enclosures: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

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The amendments revise the time constant used in the overtemperature delta temperature and the overpower delta temperature trip equations of Technical Specification Table 2.2-1.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "D. LaBarge".

David E. LaBarge, Sr. Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-327 and 50-328

Enclosures: 1. Amendment No. 211 to
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2. Amendment No. 201 to
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3. Safety Evaluation

cc w/enclosures: See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 211
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated April 6, 1995, which was superseded by application dated August 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

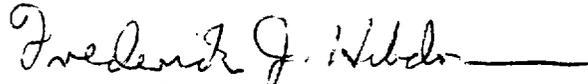
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 211, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 15, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 211

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

2-7
2-8
2-9
2-10

INSERT

2-7
2-8
2-9
2-10

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

<u>FUNCTIONAL UNIT</u>	<u>TRIP SETPOINT</u>	<u>ALLOWABLE VALUES</u>
21. Turbine Impulse Chamber Pressure - (P-13) Input to Low Power Reactor Trips Block P-7	≤ 10% Turbine Impulse Pressure Equivalent	≤ 12.4% Turbine Impulse Pressure Equivalent
22. Power Range Neutron Flux - (P-8) Low Reactor Coolant Loop Flow, and Reactor Trip	≤ 35% of RATED THERMAL POWER	≤ 37.4% of RATED THERMAL POWER
23. Power Range Neutron Flux - (P-10) - Enable Block of Source, Intermediate, and Power Range (low setpoint) Reactor Trips	≥ 10% of RATED THERMAL POWER	≥ 7.6% of RATED THERMAL POWER
24. Reactor Trip P-4	Not Applicable	Not Applicable
25. Power Range Neutron Flux - (P-9) - Blocks Reactor Trip for Turbine Trip Below 50% Rated Power	≤ 50% of RATED THERMAL POWER	≤ 52.4% of RATED THERMAL POWER

NOTATION

NOTE 1: Overtemperature $\Delta T \left(\frac{1 + \tau_4 S}{1 + \tau_5 S} \right) \leq \Delta T_o \{ K_1 - K_2 \left(\frac{1 + \tau_1 S}{1 + \tau_2 S} \right) [T - T'] + K_3 (P - P') - f_1(\Delta I) \}$

Where: $\frac{1 + \tau_4 S}{1 + \tau_5 S}$ = Lead-lag compensator on measured ΔT

τ_4, τ_5 = Time constants utilized in the lead-lag controller for ΔT , $\tau_4 \geq 5$ secs, $\tau_5 \leq 3$ sec.

ΔT_o = Indicated ΔT at RATED THERMAL POWER

K_1 ≤ 1.15

K_2 ≥ 0.011

TABLE 2.2-1 (Continued)REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTSNOTATION (Continued)

NOTE 1: (Continued)

$\frac{1 + \tau_1 S}{1 + \tau_2 S}$ = The function generated by the lead-lag controller for T_{avg} dynamic compensation

τ_1 , & τ_2 = Time constants utilized in the lead-lag controller for T_{avg} , $\tau_1 \geq 33$ secs.,
 $\tau_2 \leq 4$ secs.

T = Average temperature °F

T' ≤ 578.2°F (Nominal T_{avg} at RATED THERMAL POWER)

K_3 = 0.00055

P = Pressurizer pressure, psig

P' = 2235 psig (Nominal RCS operating pressure)

S = Laplace transform operator (sec^{-1})

and $f_1(\Delta I)$ is a function of the indicated difference between top and bottom detectors of the power-range nuclear ion chambers; with gains to be selected based on measured instrument response during plant startup tests such that:

- (i) for $q_t - q_b$ between - 29 percent and + 5 percent $f_1(\Delta I) = 0$ (where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER).

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION (Continued)

NOTE 1: (Continued)

- (ii) for each percent that the magnitude of $(q_t - q_b)$ exceeds -29 percent, the ΔT trip setpoint shall be automatically reduced by 1.50 percent of its value at RATED THERMAL POWER.
- (iii) for each percent that the magnitude of $(q_t - q_b)$ exceeds +5 percent, the ΔT trip setpoint shall be automatically reduced by 0.86 percent of its value at RATED THERMAL POWER.

NOTE 2: Overpower $\Delta T \left(\frac{1 + \tau_4 S}{1 + \tau_5 S} \right) \leq \Delta T_o \left\{ K_4 - K_5 \left(\frac{\tau_3 S}{1 + \tau_3 S} \right) T - K_6(T-T'') - f_2(\Delta I) \right\}$

Where: $\frac{1 + \tau_4 S}{1 + \tau_5 S}$ = as defined in Note 1

τ_4, τ_5 = as defined in Note 1

ΔT_o = as defined in Note 1

K_4 \leq 1.087

K_5 \geq 0.02/°F for increasing average temperature and 0 for decreasing average temperature

$\frac{\tau_3 S}{1 + \tau_3 S}$ = The function generated by the rate-lag controller for T_{avg} dynamic compensation

TABLE 2.2-1 (Continued)REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTSNOTATION (Continued)

NOTE 2: (Continued)

τ_3	=	Time constant utilized in the rate-lag controller for T_{avg} , $\tau_3 \geq 10$ secs.
K_6	\geq	0.0011 for $T > T''$ and $K_6 \geq 0$ for $T \leq T''$
T	=	as defined in Note 1
T''	=	Indicated T_{avg} at RATED THERMAL POWER (Calibration temperature for ΔT instrumentation, $\leq 578.2^\circ F$)
S	=	as defined in Note 1
$f_2(\Delta I)$	=	0 for all ΔI

NOTE 3: The channel's maximum trip setpoint shall not exceed its computed trip point by more than 1.9 percent ΔT span.

NOTE 4: The channel's maximum trip setpoint shall not exceed its computed trip point by more than 1.7 percent ΔT span.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 201
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated April 6, 1995, which was superseded by application dated August 7, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

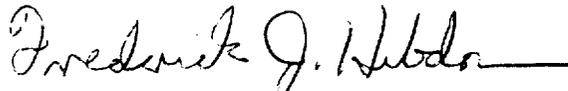
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 201, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 45 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: September 15, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 201

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

2-9
2-10
2-11

INSERT

2-9
2-10
2-11

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION

NOTE 1: Overtemperature $\Delta T \left(\frac{1 + \tau_4 S}{1 + \tau_5 S} \right) \leq \Delta T_o \{ K_1 - K_2 \left(\frac{1 + \tau_1 S}{1 + \tau_2 S} \right) [T - T'] + K_3 (P - P') - f_1(\Delta I) \}$

where: $\frac{1 + \tau_4 S}{1 + \tau_5 S}$ = Lag compensator on measured ΔT

τ_4, τ_5 = Time constants utilized in the lead-lag controller for ΔT , $\tau_4 \geq 5$ secs,
 $\tau_5 \leq 3$ secs

ΔT_o = Indicated ΔT at RATED THERMAL POWER

K_1 ≤ 1.15

K_2 ≥ 0.011

$\frac{1 + \tau_1 S}{1 + \tau_2 S}$ = The function generated by the lead-lag controller for T_{avg} dynamic compensation

$\tau_1, \& \tau_2$ = Time constants utilized in the lead-lag controller for T_{avg} , $\tau_1 \geq 33$ secs.,
 $\tau_2 \leq 4$ secs.

T = Average temperature °F

T' ≤ 578.2 °F (Nominal T_{avg} at RATED THERMAL POWER)

K_3 = 0.00055

P = Pressurizer pressure, psig

P' = 2235 psig (Nominal RCS operating pressure)

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION (Continued)

NOTE 1: (Continued)

S = Laplace transform operator. sec⁻¹

and $f_1(\Delta I)$ is a function of the indicated difference between top and bottom detectors of the power-range nuclear ion chambers; with gains to be selected based on measured instrument response during plant startup tests such that:

- (i) for $q_t - q_b$ between - 29 percent and + 5 percent $f_1(\Delta I) = 0$ (where q_t and q_b are percent RATED THERMAL POWER in the top and bottom halves of the core respectively, and $q_t + q_b$ is total THERMAL POWER in percent of RATED THERMAL POWER).
- (ii) for each percent that the magnitude of $(q_t - q_b)$ exceeds -29 percent, the ΔT trip setpoint shall be automatically reduced by 1.50 percent of its value at RATED THERMAL POWER.
- (iii) for each percent that the magnitude of $(q_t - q_b)$ exceeds +5 percent, the ΔT trip setpoint shall be automatically reduced by 0.86 percent of its value at RATED THERMAL POWER.

NOTE 2: Overpower $\Delta T \left(\frac{1 + \tau_4 S}{1 + \tau_5 S} \right) \leq \Delta T_o \{ K_4 - K_5 \left(\frac{\tau_3 S}{1 + \tau_3 S} \right) T - K_6 [T - T^*] - f_2(\Delta I) \}$

where: $\frac{1 + \tau_4 S}{1 + \tau_5 S}$ = as defined in Note 1

TABLE 2.2-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION TRIP SETPOINTS

NOTATION (Continued)

NOTE 2: (Continued)

τ_4, τ_5 = as defined in Note 1

ΔT_o = as defined in Note 1

K_4 \leq 1.087

K_5 \geq 0.02/°F for increasing average temperature and 0 for decreasing average temperature

$\frac{\tau_3 S}{1 + \tau_3 S}$ = The function generated by the rate-lag controller for T_{avg} dynamic compensation

τ_3 = Time constant utilized in the rate-lag controller for T_{avg} , $\tau_3 \geq 10$ secs.

K_6 \geq 0.0011 for $T > T''$ and $K_6 \geq 0$ for $T \leq T''$

T = as defined in Note 1

T'' = Indicated T_{avg} at RATED THERMAL POWER (Calibration temperature for ΔT instrumentation, $\leq 578.2^\circ\text{F}$)

S = as defined in Note 1

$f_2(\Delta I)$ = 0 for all ΔI



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 211 TO FACILITY OPERATING LICENSE NO. DPR-77

AND AMENDMENT NO. 201 TO FACILITY OPERATING LICENSE NO. DPR-79

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-327 AND 50-328

1.0 INTRODUCTION

By letter dated August 7, 1995, Tennessee Valley Authority (TVA) requested changes to the Sequoyah Nuclear Plant (SQN) Units 1 and 2 Technical Specifications (TS) to revise the numerical value for the time constant τ_4 found in the overtemperature delta temperature (OT Δ T) and overpower delta temperature (OP Δ T) trip equations of TS Table 2.2-1. The τ_4 value would be changed from 12 seconds to 5 seconds. In addition, the equality signs for the τ and K constant numerical values would be changed to either a "less than or equal to" or a "greater than or equal to" function, as applicable. This application superseded a similar TS amendment that was proposed by letter dated April 6, 1995, to move these constants to the Core Operating Limits Report.

SQN has experienced OP Δ T turbine runback alarms on individual channels resulting in partial runback signals. These occurrences could result in turbine runbacks or reactor trips during functional testing at power. The proposed changes to the τ_4 constant numerical values result from a reanalysis to provide additional margin to the trip setpoints and minimize the potential for turbine runback and reactor trip signals. In the interim, SQN Unit 2 is presently operating at a maximum power of 99.5 percent of rated thermal power in order to reduce the potential for inadvertent turbine runback and reactor trip signals because of the spurious OP Δ T channel actuations.

2.0 EVALUATION

The SQN safety analyses that rely on the OP Δ T and OT Δ T trips for primary protection are the uncontrolled rod cluster assembly bank (RCCA) withdrawal at power, loss of external load and/or turbine trip, uncontrolled boron dilution, accidental depressurization of the reactor coolant system, steamline break with coincident rod withdrawal at power, and steamline break mass/energy outside containment. These transients were reanalyzed or evaluated by Westinghouse Electric Corporation (W) with the proposed changes and found to adhere to all of the safety analysis acceptance criteria described in the plant's Final Safety Analysis Report (FSAR). Therefore, the staff concludes that the proposed revision of the τ_4 constant numerical value is acceptable.

The revision of the equality signs for the constant numerical values of the OT Δ T and OP Δ T equations will provide the flexibility to set these values more

conservatively than the limits utilized in the safety analysis. These changes, which are consistent with the W improved Standard TS, NUREG-1431, do not change these functions. Therefore, the staff finds the proposed changes to the equality signs acceptable.

Based on the review described above, the staff finds the proposed modifications to the SQN Units 1 and 2 TS to revise the OTAT and OPAT equation constant numerical value τ_4 from 12 seconds to 5 seconds, and the change of the equality signs for the τ and K constant numerical values to either a "less than or equal to" or a "greater than or equal to" function, acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendment. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendment changes a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (60 FR 42187). Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Laurence Kopp

Dated: September 15, 1995

Mr. Oliver D. Kingstey, Jr.
Tennessee Valley Authority

SEQUOYAH NUCLEAR PLANT

cc:

Mr. O. J. Zeringue, Sr. Vice President
Nuclear Operations
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

TVA Representative
Tennessee Valley Authority
11921 Rockville Pike
Suite 402
Rockville, MD 20852

Dr. Mark O. Medford, Vice President
Engineering & Technical Services
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Regional Administrator
U.S. Nuclear Regulatory Commission
Region II
101 Marietta Street, NW., Suite 2900
Atlanta, GA 30323

Mr. D. E. Nunn, Vice President
New Plant Completion
Tennessee Valley Authority
3B Lookout Place
1101 Market Street
Chattanooga, TN 37402-2801

Mr. William E. Holland
Senior Resident Inspector
Sequoyah Nuclear Plant
U.S. Nuclear Regulatory Commission
2600 Igou Ferry Road
Soddy Daisy, TN 37379

Mr. R. J. Adney, Site Vice President
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

Mr. Michael H. Mobley, Director
Division of Radiological Health
3rd Floor, L and C Annex
401 Church Street
Nashville, TN 37243-1532

General Counsel
Tennessee Valley Authority
ET 11H
400 West Summit Hill Drive
Knoxville, TN 37902

County Judge
Hamilton County Courthouse
Chattanooga, TN 37402-2801

Mr. P. P. Carrier, Manager
Corporate Licensing
Tennessee Valley Authority
4G Blue Ridge
1101 Market Street
Chattanooga, TN 37402-2801

Mr. Ralph H. Shell
Site Licensing Manager
Sequoyah Nuclear Plant
Tennessee Valley Authority
P.O. Box 2000
Soddy Daisy, TN 37379

AMENDMENT NO. 211 FOR SEQUOYAH UNIT NO. 1 - DOCKET NO. 50-327 and
AMENDMENT NO. 201 FOR SEQUOYAH UNIT NO. 2 - DOCKET NO. 50-328
DATED: September 15, 1995

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Docket Files

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S. Varga

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G. Hill

T-5-C-3(2 per docket)

C. Grimes

0-11-E-22

L. Kopp

ACRS(4)

E. Merschoff

RII

M. Lesser

RII