



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

September 28, 1992

Docket Nos. 50-327
and 50-328

Tennessee Valley Authority
ATTN: Dr. Mark O. Medford, Vice President
Nuclear Assurance, Licensing & Fuels
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Dr. Medford:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M83583 AND M83584) (TS 92-06)

The Commission has issued the enclosed Amendment No. 163 to Facility Operating License No. DPR-77 and Amendment No. 153 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated May 28, 1992, which was modified by your letter dated July 14, 1992.

The changes reflect a restructuring within the Tennessee Valley Authority's Nuclear Power Organization affecting the Independent Safety Engineering manager and updates the Plant Operations Review Committee membership.

However, the proposed change to substitute the references to ANSI N18.1-1971, the March 28, 1980 NRC letter, and Regulatory Guide 1.8, with a reference to the TVA Nuclear Quality Assurance Plan in Specification 6.3.1 has been denied. Similarly, the proposed substitution of the reference to ANSI N18.1-1971 in Specification 6.4.1 with a reference to the TVA Quality Assurance Plan has been denied. The staff has determined that the references should be retained to ensure appropriate enforceability of the provisions of the applicable standards which include important details for the proper implementation of these programs.

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A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice. Enclosed is the Notice of Partial Denial which has been sent to the Office of the Federal Register for publication.

Sincerely,

Original signed by

Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Amendment No. 163 to License No. DPR-77
- 2. Amendment No. 153 to License No. DPR-79
- 3. Safety Evaluation
- 4. Notice of Partial Denial

cc w/enclosures:
See next page

WAPB 9/1
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OFC:	PDII-4/LA	PDII-4/PM	LHRB	OGC	PDII-4/D
NAME:	MSanders <i>ms</i>	DLaBarge:as	JWermiel	<i>CPW</i>	FHebdon
DATE:	8/31/92	9/14/92	9/11/92	9/12/92	9/28/92

DOCUMENT NAME: TAC83583.AMM

del 9/23/92

Tennessee Valley Authority
ATTN: Dr. Mark O. Medford

cc:

Mr. John B. Waters, Chairman
Tennessee Valley Authority
ET 12A
400 West Summit Hill Drive
Knoxville, Tennessee 37902

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Nuclear Operations
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Mr. M. J. Burzynski, Manager
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Sequoyah Nuclear Plant

County Judge
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U.S.N.R.C.
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Soddy Daisy, Tennessee 37379

AMENDMENT NO. 163 FOR SEQUOYAH UNIT NO. 1 - DOCKET NO. 50-327 and
AMENDMENT NO. 153 FOR SEQUOYAH UNIT NO. 2 - DOCKET NO. 50-328
DATED: September 28, 1992

Distribution

Docket File

NRC PDR

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SQN Reading File

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G. Hill P1-130 (4 per docket)

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J. Calvo 14-E-4

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OPA 2-G-5

OC/LFMB MNBB-9112

J. Wermiel 10-D-24



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 163
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 28, 1992, and modified by letter dated July 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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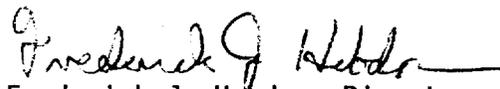
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 163, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 28, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 163

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 6-5
3/4 6-6
3-4 6-23

INSERT

3/4 6-5
3/4 6-6
3/4 6-23

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING (ISE)

FUNCTION

6.2.3.1 The ISE shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The ISE shall be composed of at least 3 dedicated full-time engineers located onsite. These engineers will be supplemented as necessary by full-time engineers shared among all TVA nuclear sites to achieve an equivalent staffing of 5 full-time engineers performing the ISE functions applicable to Sequoyah.

RESPONSIBILITIES

6.2.3.3 The ISE shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

6.2.3.4 The ISE shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Manager, Nuclear Experience Review/Independent Safety Engineering.

6.2.4 SHIFT TECHNICAL ADVISOR (STA)

6.2.4.1 The STA shall serve in an advisory capacity to the Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions except for the Site Radiological Control Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.0 The Senior Vice President, Nuclear Power is responsible for the safe operation of all TVA power plants.

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Plant Manager
Member:	Operations Manager
Member:	Site Radiological Control Manager
Member:	Maintenance Manager
Member:	Technical Support Manager
Member:	Quality Audit and Monitoring Manager
Member:	Nuclear Engineering Representative

ADMINISTRATIVE CONTROLS

6.10.2 The following records shall be retained for the duration of the Unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the facility staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required for lifetime retention by the Nuclear Quality Assurance Plan.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PORC, SQN RARC, and the NSRB.
- l. Records of analyses required by the radiological environmental monitoring program.
- m. Records of secondary water sampling and water quality.
- n. Records of the service life monitoring of all safety-related hydraulic and mechanical snubbers, required by T/S 3.7.9, including the maintenance performed to renew the service life.
- o. Records for Environmental Qualification which are covered under the provisions of Paragraph 2.c.(12)(b) of License No. DPR-77.
- p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 153
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 28, 1992, and modified by letter dated July 14, 1992, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

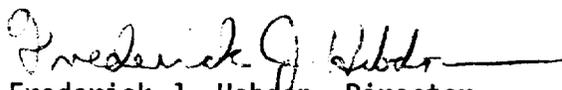
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 153, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: September 28, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 153

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 6-5
3/4 6-6
3-4 6-24

INSERT

3/4 6-5
3/4 6-6
3/4 6-24

ADMINISTRATIVE CONTROLS

6.2.3 INDEPENDENT SAFETY ENGINEERING (ISE)

FUNCTION

6.2.3.1 The ISE shall function to examine plant operating characteristics, NRC issuances, industry advisories, Licensee Event Reports and other sources which may indicate areas for improving plant safety.

COMPOSITION

6.2.3.2 The ISE shall be composed of at least 3 dedicated full-time engineers located onsite. These engineers will be supplemented as necessary by full-time engineers shared among all TVA nuclear sites to achieve an equivalent staffing of 5 full-time engineers performing the ISE functions applicable to Sequoyah.

RESPONSIBILITIES

6.2.3.3 The ISE shall be responsible for maintaining surveillance of plant activities to provide independent verification* that these activities are performed correctly and that human errors are reduced as much as practical.

AUTHORITY

6.2.3.4 The ISE shall make detailed recommendations for revised procedures, equipment modifications, or other means of improving plant safety to the Manager, Nuclear Experience Review/Independent Safety Engineering.

6.2.4 SHIFT TECHNICAL ADVISOR (STA)

6.2.4.1 The STA shall serve in an advisory capacity to the Shift Supervisor on matters pertaining to the engineering aspects of assuring safe operation of the unit.

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions except for the Site Radiological Control Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

*Not responsible for sign-off function.

ADMINISTRATIVE CONTROLS

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Plant Manager and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55 and shall include familiarization with relevant industry operational experience.

6.5 REVIEW AND AUDIT

6.5.0 Senior Vice President, Nuclear Power is responsible for the safe operation of all TVA power plants.

6.5.1 PLANT OPERATIONS REVIEW COMMITTEE (PORC)

FUNCTION

6.5.1.1 The PORC shall function to advise the Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The PORC shall be composed of the:

Chairman:	Plant Manager
Member:	Operations Manager
Member:	Site Radiological Control Manager
Member:	Maintenance Manager
Member:	Technical Support Manager
Member:	Quality Audit and Monitoring Manager
Member:	Nuclear Engineering Representative

ADMINISTRATIVE CONTROLS

6.10.2 The following records shall be retained for the duration of the Unit Operating License:

- a. Records and drawing changes reflecting unit design modifications made to systems and equipment described in the Final Safety Analysis Report.
- b. Records of new and irradiated fuel inventory, fuel transfers and assembly burnup histories.
- c. Records of radiation exposure for all individuals entering radiation control areas.
- d. Records of gaseous and liquid radioactive material released to the environs.
- e. Records of transient or operational cycles for those unit components identified in Table 5.7-1.
- f. Records of reactor tests and experiments.
- g. Records of training and qualification for current members of the facility staff.
- h. Records of in-service inspections performed pursuant to these Technical Specifications.
- i. Records of Quality Assurance activities required for lifetime retention by the Nuclear Quality Assurance Plan.
- j. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59.
- k. Records of meetings of the PORC, SQN RARC, and the NSRB.
- l. Records of analyses required by the radiological environmental monitoring program.
- m. Records of secondary water sampling and water quality.
- n. Records of the service life monitoring of all safety-related hydraulic and mechanical snubbers, required by T/S 3.7.9, including the maintenance performed to renew the service life.
- o. Records for environmental qualification which are covered under the provisions of paragraph 2.C.(10)(b) of license No. DPR-79.
- p. Records of reviews performed for changes made to the OFFSITE DOSE CALCULATION MANUAL and the PROCESS CONTROL PROGRAM.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555

ENCLOSURE 3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.163 TO FACILITY OPERATING LICENSE NO. DPR-77
AND AMENDMENT NO.153 TO FACILITY OPERATING LICENSE NO. DPR-79
TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-327 AND 50-328

1.0 INTRODUCTION

By application dated May 28, 1992, and amended by letter dated July 14, 1992, the Tennessee Valley Authority (TVA or the licensee) proposed an amendment to the Technical Specifications (TS) for Sequoyah Nuclear Plant (SQN) Units 1 and 2. The proposed changes would revise Specification 6.2.3.4 to reflect a restructuring of personnel titles within TVA's Nuclear Power organization. Specifically, TVA proposed changing the title of the person to whom the Independent Safety Engineering (ISE) reports from Manager of Nuclear Managers Review Group to Manager, Nuclear Reviews. By letter dated July 14, 1992, TVA proposed that this title be changed to Manager, Nuclear Experience Review/Independent Safety Engineering.

In addition, the licensee has proposed a change to Specifications 6.3.1 and 6.4.1 to indicate that the staff qualification and training program requirements are specified in the Nuclear Quality Assurance Plan. The change to Specification 6.10.2i would reflect the current TVA organizational nomenclature for the Nuclear Quality Assurance (QA) Plan and the change to Specification 6.5.1.2 would update the title of the Quality Assurance Manager serving on the Plant Operations Review Committee (PORC).

2.0 EVALUATION

As part of an organizational change in September 1991, TVA brought the corporate Nuclear Experience Review function into the Nuclear Managers Review Group (NMRG). At that time the name of the group was changed from NMRG to Nuclear Reviews to more accurately reflect the expanded organizational responsibilities. This change did not affect the ISE, since the same reporting relationships were maintained.

Since then, organizational changes within the management structure of TVA's Nuclear Power organization have been made to provide a sharper focus on oversight and assessment functions. This action resulted in the need to change Specification 6.2.3.4 to show the ISE reporting to the Manager, Nuclear Experience Review/Independent Safety Engineering. With this change, the ISE will continue to perform independent on-site reviews within the corporate Nuclear Assurance organization. It also maintains compliance with NUREG-0737

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guidance and the Standard TS by ensuring that ISE reports to a high-level corporate official in a technically-oriented position who is not in the management chain for power production, and who is not an integral part of the QA organization; i.e., the Manager, Nuclear Experience Review/Independent Safety Engineering.

In an effort to make the TS consistent with both the Nuclear Quality Assurance Plan and current regulatory guidance, the licensee proposed changes to Specification 6.3.1, FACILITY STAFF QUALIFICATIONS. The proposed changes would substitute the references to ANSI N18.1-1971, the March 28, 1980 NRC letter, and Regulatory Guide 1.8 with a reference to the TVA Nuclear Quality Assurance Plan. These changes reflect the requirements of Regulatory Guide (RG) 1.8, Revision 2, for all new personnel qualifying for positions identified in Regulatory Position C.1 after January 1, 1990. Personnel qualified for these positions before this date would continue to meet the requirements of RG 1.8, Revision 1-R. As specified in Regulatory Position C.2, all other positions will meet the requirements of the American National Standards Institute/American Nuclear Society Standard N18.1-1971. These requirements are also currently reflected in the TVA Nuclear Quality Assurance Plan.

However, the NRC staff has determined that parts of the proposed change to Specification 6.3.1 should not be removed in order to ensure that appropriate enforceability of the programs of the applicable standards (which include important details for the proper implementation of these programs) is retained. Therefore, the staff has denied: (a) removal of the reference to ANSI N18.1-1971; (b) removal of the reference to the Site Radiological Control Manager qualification specification to Regulatory Guide 1.8, September 1975; and (c) insertion of the reference to the TVA Nuclear Quality Assurance Plan. The staff has, however, determined that removal of the statement "the supplemental requirements specified in Section A and C of Enclosure 1 of the March 28, 1980 NRC letter to all licensees" is acceptable. As a result, Specification 6.3.1 will read: "Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Site Radiological Control Manager who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975." These conclusions and changes have been discussed with the TVA staff.

The licensee also proposed changes to Specification 6.4.1, TRAINING, to make the TS consistent with both the Nuclear Quality Assurance Plan and current regulatory guidance. This proposed change would reflect the 1987 revision to 10 CFR 55 that allowed facilities to substitute programs based on a systematic approach to training methodology and to conduct training using a plant-referenced simulator certified by the NRC, for the existing regulatory-based programs. This revision reflected the Commission's acceptance of industry initiatives and provides flexibility to licensees who maintain accreditation through the National Academy for Nuclear Training and who conduct training of operators on certified, plant-referenced simulators. It eliminated Appendix A of 10 CFR 55 and superseded the operator licensing requirements of NUREGs 0737 and 0094.

The training program structure and content guidance continue to be governed by the corporate-level training program described in the Nuclear Quality Assurance Plan using procedures contained in the Nuclear Power Training Manual. These program procedures, which are quality related, are subject to review in accordance with 10 CFR 50.59. They describe the programs that are accredited by the National Academy for Nuclear Training. Accreditation is not granted or maintained unless compliance with the academy-published guidelines is established. These guidelines are consistent with, and in many areas exceed, the previous regulatory requirements.

However, the NRC staff has determined that certain proposed changes to Specification 6.4.1 should be retained in order to ensure appropriate enforceability of the provisions of the applicable standards which include important details for the proper implementation of these programs. Therefore, the staff has denied the proposed change to remove the reference to ANSI N18.1-1971 and insert a reference to the TVA Nuclear Quality Assurance Plan. The staff has, however, determined that removal of the reference to the March 28, 1980 letter is acceptable. These conclusions and changes have been discussed with the TVA staff.

The proposed change to Specification 6.5.1.2 is administrative in nature and reflects the current organizational nomenclature. The change to the title of the Plant Operations Review Committee (PORC) member from Quality Engineering and Monitoring Supervisor, to Quality Audit and Monitoring Manager would not change the duties or responsibilities of the member, nor does it result in a change to the ability of the PORC to perform its designated functions.

The change to Specification 6.10.2.i is also administrative in nature since it reflects the current title given to the document that summarizes the Quality Assurance program. The purpose and requirements remain unchanged.

With the exception of the proposed changes to Specification 6.3.1 and 6.4.1 that were found to be unacceptable as described above, the staff has determined that the other proposed changes represent no change to the implementation of the associated regulations and are consistent with the appropriate regulatory guidance, or are administrative in nature. Therefore, the staff has determined that they are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

These amendments involve changes to administrative procedures or requirements. The Commission has previously issued proposed findings that the amendments involve no significant hazards consideration, and there has been no public comments on such findings (57 FR 30262 and 57 FR 34591).

Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. LaBarge

Date: September 28, 1992

UNITED STATES NUCLEAR REGULATORY COMMISSION
TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-327 AND 50-328
NOTICE OF PARTIAL DENIAL OF AMENDMENT TO FACILITY OPERATING LICENSE
AND OPPORTUNITY FOR HEARING

The U.S. Nuclear Regulatory Commission (the Commission) has denied a portion of a request by the Tennessee Valley Authority (TVA or the licensee) for an amendment to Facility Operating License Nos. DPR-77 and DPR-79 issued to the licensee for operation of the Sequoyah Nuclear Plant, Units 1 and 2, located in Soddy Daisy, Tennessee. Notice of Consideration of Issuance of Amendment to Facility Operating License and Proposed No Significant Hazards Consideration and Determination and Opportunity for Hearing was published in the FEDERAL REGISTER on July 8, 1992 (57 FR 30262) and renoticed on August 5, 1992 (57 FR 34591).

The purpose of the licensee's amendment request was to revise the Technical Specifications (TS) to reflect a restructuring within TVA's Nuclear Power Organization affecting the Independent Safety Engineering manager, change the reference document that contains the facility staff qualification and training requirements, and update Plant Operations Review Committee membership.

The staff has determined that the proposed change to substitute the references to ANSI N18.1-1971, the March 28, 1980 NRC letter, and Regulatory Guide 1.8, with a reference to the TVA Nuclear Quality Assurance Plan in

Specification 6.3.1 cannot be granted. Similarly, the proposed substitution of the references to ANSI N18.1-1971 in Specification 6.4.1 with a reference to the TVA Quality Assurance Plan cannot be granted. The licensee was notified of the Commission's denial by letter dated September 28, 1992.

By November 2, 1992, the licensee may demand a hearing with respect to the denial described above. Any person whose interest may be affected by this proceeding may file a written petition for leave to intervene.

A request for hearing or petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC, 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, by the above date.

A copy of any petitions should also be sent to the Office of the General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, E11 B33, Knoxville, Tennessee 37902, attorney for the licensee.

For further details with respect to this action, see (1) the application for amendment dated May 28, 1992, which was modified by letter dated July 14, 1992, and (2) the Commission's letter to the licensee dated September 28, 1992.

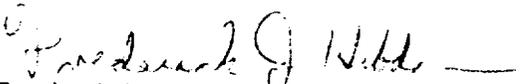
These documents are available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Chattanooga-Hamilton County Library, 1001 Broad Street, Chattanooga, Tennessee 37402.

A copy of item (2) may be obtained upon request addressed to the U.S. Nuclear

Regulatory Commission, Washington, DC, 20555, Attention: Document Control Desk.

Dated at Rockville, Maryland, this 28th day of September, 1992.

FOR THE NUCLEAR REGULATORY COMMISSION


Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation