

July 24, 1992

Docket Nos. 50-327
and 50-328

Tennessee Valley Authority
ATTN: Dr. Mark O. Medford, Vice President
Nuclear Assurance, Licensing & Fuels
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Dr. Medford:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M82245 AND 82246) (TS 91-16)

The Commission has issued the enclosed Amendment No. 160 to Facility Operating License No. DPR-77 and Amendment No. 150 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated November 27, 1991.

The amendments revise the actions required when certain 6.9kv Shutdown Board Engineered Safety Feature Actuation System and Emergency Diesel Generator loss-of-power instrumentation is inoperable.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

David E. LaBarge, Senior Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 160 to License No. DPR-77
2. Amendment No. 150 to License No. DPR-79
3. Safety Evaluation

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cc w/enclosures:
See next page

NAME	PDII-4/LA	PDII-4/PM	SELBY	OGC	PDII-4/D
OFFICE	MSanders	DLaBarge:dw/as	FRosa	Bachmann	FHebdon
DATE:	6/23/92	6/23/92	7/7/92	7/10/92	7/24/92

DOCUMENT NAME: SQN91-16

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PDR ADDCK 05000327
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Tennessee Valley Authority
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 160
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated November 27, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 160, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 24, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 160

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-21
3/4 3-23
3/4 3-23a

INSERT

3/4 3-21
3/4 3-23
3/4 3-23a

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
7. LOSS OF POWER					
a. 6.9 kv Shutdown Board --Loss of Voltage					
1. Start Diesel Generators	2/shutdown board	1 loss of voltage on any shutdown board	2/shutdown board	1, 2, 3, 4	35*
2. Load Shedding	2/shutdown board	1/shutdown board	2/shutdown board	1, 2, 3, 4	34*
b. 6.9 kv Shutdown Board Degraded Voltage					
1. Voltage Sensors	3/shutdown board	2/shutdown board	2/shutdown board	1, 2, 3, 4	34*
2. Diesel Generator Start and Load Shedding Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	34*
3. SI/Degraded Voltage Enable Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	34*
8. ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INTERLOCKS					
a. Pressurizer Pressure - P-11/Not P-11	3	2	2	1, 2, 3	22a
b. Deleted					
c. Steam Generator Level P-14	3/loop	2/loop any loop	3/loop	1, 2	22c

SEQUOYAH - UNIT 1

3/4 3-21

Amendment No. 41, 129, 141

TABLE 3.3-3 (Continued)

- ACTION 21 - With less than the Minimum Number of Channels OPERABLE, declare the associated auxiliary feedwater pump inoperable, and comply with the ACTION requirements of Specification 3.7.1.2.
- ACTION 22 With less than the Minimum Number of Channels OPERABLE, declare the interlock inoperable and verify that all affected channels of the functions listed below are OPERABLE or apply the appropriate ACTION statement(s) for those functions. Functions to be evaluated are:
- a. Safety Injection
 - Pressurizer Pressure
 - Steam Line Pressure
 - Negative Steam Line Pressure Rate
 - b. Deleted
 - c. Turbine Trip
 - Steam Generator Level High-High
 - Feedwater Isolation
 - Steam Generator Level High-High
- ACTION 23 - With the number of OPERABLE channels one less than the Total Number of Channels, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.
- ACTION 24 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- ACTION 25 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the ACTION required by Specification 3.7.1.5.
- ACTION 34 -
- a. With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 - b. With the number of OPERABLE channels less than the Total Number of Channels by more than one, declare the associated 6,900-volt shutdown board inoperable, and comply with the action requirements of Specification 3.8.2.1 or 3.8.2.2 as applicable.

TABLE 3.3-3 (Continued)

- ACTION 35 - With the number of OPERABLE channels less than the Total Number of Channels by one or more, declare the associated diesel generator set inoperable, and comply with the action requirements of Specification 3.8.1.1 or 3.8.1.2 as applicable.
- ACTION 36 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
- a. The inoperable channel is placed in the tripped condition within 6 hours.
 - b. For the affected protection set, the Trip Time Delay for one affected steam generator (T_S) is adjusted to match the Trip Time Delay for multiple affected steam generators (T_M) within 4 hours.
 - c. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.2.1.1.
- ACTION 37 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected protection set, the Trip Time Delays (T_S and T_M) threshold power level for zero seconds time delay is adjusted to 0% RTP.
- ACTION 38 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected protection set, the Steam Generator Water Level - Low-Low (EAM) channels trip setpoint is adjusted to the same value as Steam Generator Water Level - Low-Low (Adverse).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Tennessee Valley Authority (the licensee) dated November 27, 1991, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No.150 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 24, 1992

ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-21
3/4 3-23
3/4 3-23a

INSERT

3/4 3-21
3/4 3-23
3/4 3-23a

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
7. LOSS OF POWER					
a. 6.9 kv Shutdown Board --Loss of Voltage					
1. Start Diesel Generators	2/shutdown board	1 loss of voltage on any shutdown board	2/shutdown board	1, 2, 3, 4	35*
2. Load Shedding	2/shutdown board	1/shutdown board	2/shutdown board	1, 2, 3, 4	34*
b. 6.9 kv Shutdown Board Degraded Voltage					
1. Voltage Sensors	3/shutdown board	2/shutdown board	2/shutdown board	1, 2, 3, 4	34*
2. Diesel Generator Start and Load Shedding Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	34*
3. SI/Degraded Voltage Enable Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	34*

SEQUOYAH - UNIT 2

3/4 3-21

Amendment No. 18, 132, 150

TABLE 3.3-3 (Continued)

- ACTION 21 - With less than the Minimum Number of Channels OPERABLE, declare the associated auxiliary feedwater pump inoperable, and comply with the ACTION requirements of Specification 3.7.1.2.
- ACTION 22 With less than the Minimum Number of Channels OPERABLE, declare the interlock inoperable and verify that all affected channels of the functions listed below are OPERABLE or apply the appropriate ACTION statement(s) for those functions. Functions to be evaluated are:
- a. Safety Injection
 - Pressurizer Pressure
 - Steam Line Pressure
 - Negative Steam Line Pressure Rate
 - b. Deleted
 - c. Turbine Trip
 - Steam Generator Level High-High
 - Feedwater Isolation
 - Steam Generator Level High-High
- ACTION 23 - With the number of OPERABLE channels one less than the Total Number of Channels, be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours; however, one channel may be bypassed for up to 2 hours for surveillance testing per Specification 4.3.2.1.1.
- ACTION 24 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.
- ACTION 25 - With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or declare the associated valve inoperable and take the ACTION required by Specification 3.7.1.5.
- ACTION 34 -
- a. With the number of OPERABLE channels one less than the Total Number of Channels, restore the inoperable channel to OPERABLE status within 48 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
 - b. With the number of OPERABLE channels less than the Total Number of Channels by more than one, declare the associated 6,900-volt shutdown board inoperable, and comply with the action requirements of Specification 3.8.2.1 or 3.8.2.2 as applicable.

TABLE 3.3-3 (Continued)

- ACTION 35 - With the number of OPERABLE channels less than the Total Number of Channels by one or more, declare the associated diesel generator set inoperable, and comply with the action requirements of Specification 3.8.1.1 or 3.8.1.2 as applicable.
- ACTION 36 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided the following conditions are satisfied:
- a. The inoperable channel is placed in the tripped condition within 6 hours.
 - b. For the affected protection set, the Trip Time Delay for one affected steam generator (T_S) is adjusted to match the Trip Time Delay for multiple affected steam generators (T_M) within 4 hours.
 - c. The Minimum Channels OPERABLE requirement is met; however, the inoperable channel may be bypassed for up to 4 hours for surveillance testing of other channels per Specification 4.3.1.1.1.
- ACTION 37 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected protection set, the Trip Time Delays (T_S and T_M) threshold power level for zero seconds time delay is adjusted to 0% RTP.
- ACTION 38 - With the number of OPERABLE channels one less than the Total Number of Channels, STARTUP and/or POWER OPERATION may proceed provided that within 6 hours, for the affected protection set, the Steam Generator Water Level - Low-Low (EAM) channels trip setpoint is adjusted to the same value as Steam Generator Water Level - Low-Low (Adverse).



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555

ENCLOSURE 3

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 160 TO FACILITY OPERATING LICENSE NO. DPR-77
AND AMENDMENT NO.150 TO FACILITY OPERATING LICENSE NO. DPR-79

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-327 AND 50-328

1.0 INTRODUCTION

By application dated November 27, 1991, the Tennessee Valley Authority (the licensee) submitted a request for changes to the Sequoyah Nuclear Plant, Units 1 and 2 Technical Specifications (TS). The proposed amendments would address the actions required in the event that the loss-of-power instrumentation associated with the 6.9-kilovolt (kv) shutdown boards and the diesel generators is inoperable. The proposed change would: (1) add two new action statements (No. 34 and 35) to Table 3.3-3, "Engineered Safety Feature Actuation System Instrumentation;" and (2) change the references in Item 7 (Loss of Power), Items 7.a.2 and 7.b, from Action Statement 20 to Action Statement 34, and Item 7.a.1 from reference to Action Statement 20 to Action Statement 35.

Table 3.3-3, Items 7.a.2 and 7.b address the 6.9-kv shutdown board load shedding and degraded voltage instrumentation. The addition of Action Statement 34 would result in retaining the present required actions specified in Action Statement 20 in the event that the number of operable channels is one less than the Total Number of Channels listed in the table. However, in the event that the number of operable channels is less than the Total Number of Channels listed in the table by more than one, new Action Statement 34.b would allow continued operation provided that the associated 6.9-kv shutdown board is declared inoperable and the actions specified in Specification 3.8.2.1 or 3.6.2.2 (as applicable, depending on the operating mode of the plant) are complied with.

Item 7.a.1 addresses operability of the 6.9-kv shutdown board diesel generator start instrumentation. In the event that the number of operable channels is less than the Total Number of Channels by one or more, the new Action Statement 35 would require that the diesel generator be declared inoperable

and Specification 3.8.1.1 or 3.8.1.2 (as applicable, depending on the operating mode of the plant) be complied with.

2.0 EVALUATION

Power to each Shutdown Board is supplied from one of three sources -- the unit main generator (the normal source), an offsite bus (the alternate source), and a diesel generator (the standby source). The shutdown boards, in turn, supply power to electrical devices that are important to safety. Transfer from the normal source or the alternate source to the standby source, tripping of certain load breakers (load shedding), startup of the emergency diesel generators and other Emergency Safeguards Function (ESF) equipment, and energizing of timers to control the sequencing of these events, occur when relay instrumentation senses a loss of voltage or low voltage on a 6.9-kv shutdown board. A two-out-of-three logic design for detecting a degraded voltage condition actuates two parallel and redundant sets of timers and relays, either one of which will cause initiation of the protective actions.

The installed test switches and test blocks provided for testing the voltage sensors do not have the capability to disable only one channel and allow performance of all required tests without actuating the ESF unnecessarily. The tests require disabling of both sets of timers and relays. For EDG start instrumentation, the single test switch disables all channels that would initiate a loss of voltage start signal. In order to perform the tests with only one channel inoperable, multiple wire lifts and jumpers would be necessary, resulting in a significant potential for inadvertent EDG starts and ESF actuations. Design changes to provide additional circuitry to allow testing a single channel, while maintaining operability of the companion channel, would be very complex, expensive, and could create test performances that could also challenge ESF actuation.

Presently, Action Statement 20 requires that if the number of operable channels is one less than the Total Number of Channels specified in Table 3.3-3, the inoperable channel must be returned to the operable status within 48 hours or else initiate a plant shutdown. If more than one channel is inoperable, this action statement would not apply and the requirements of TS 3.0.3 would be applicable, which would require that an immediate plant shutdown be initiated.

Since it is necessary to disable both relay channels in order to test the loss of voltage instrumentation for the respective shutdown board or EDG, a literal interpretation of the TS would require that a plant shutdown be initiated whenever the TS-required loss of voltage surveillance tests were performed. This interpretation of the TS requirement is more restrictive than the present TS requirement for inoperability of a single shutdown board or EDG. Since the testing of this instrumentation only affects the respective shutdown board or EDG, this aspect of the requirement is overly restrictive and the consequences

of the loss of the instrumentation inconsistent with the loss of the shutdown board or the EDG.

The proposed TS changes are intended to incorporate action statements that are consistent with the present shutdown board and EDG operability requirements and to clarify the actions required when the loss of power instrumentation and timers are inoperable. The proposed TS change requires that the shutdown board be declared inoperable for more than one load shedding or degraded voltage channel inoperable and maintains the existing action requirement if only one channel is inoperable. For the EDGs, the proposed change would require that failure of any loss of power start channel would require declaring the EDG inoperable and compliance with the associated actions for an inoperable EDG.

For the load shedding and the shutdown board degraded voltage ESF actuation channels, the required actions for a loss of one instrument channel remains unchanged (only the number of the Action Statement would change from "20" to "34a"). If both instrument channels become inoperable, Action Statement 34b would require that the associated shutdown board be declared inoperable and that the present TS requirements for inoperable shutdown boards in Specifications 3.8.2.1 or 3.8.2.2, as applicable for the plant operating mode, be enforced.

For the diesel generators, this would be accomplished by revising the Action Statement from "20" to "35," such that if one or both of the ESF loss of voltage actuation instrument channels are inoperable, the associated diesel generator is declared inoperable. The new statement then requires compliance with the present TS diesel generator operability specification (3.8.1.1 or 3.8.1.2, as applicable for the plant operating mode).

In summary, the Action 20 changes that create the proposed new Actions 34 and 35 ensure the application of actions consistent with the present TS requirement for inoperability of the shutdown board and EDGs, for loss-of-power instrumentation when multiple inoperabilities occur, and prevent unnecessary TS-required shutdown to perform TS-required testing. These proposed changes serve to clarify and strengthen the technical specification requirements. Hence, the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change a requirement with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (57 FR 2601). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: D. LaBarge

Date: July 24, 1992

July 24, 1992

Docket Nos. 50-327
and 50-328

Tennessee Valley Authority
ATTN: Dr. Mark O. Medford, Vice President
Nuclear Assurance, Licensing & Fuels
3B Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Dr. Medford:

SUBJECT: ISSUANCE OF AMENDMENTS (TAC NOS. M82245 AND 82246) (TS 91-16)

The Commission has issued the enclosed Amendment No. 160 to Facility Operating License No. DPR-77 and Amendment No. 150 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated November 27, 1991.

The amendments revise the actions required when certain 6.9kv Shutdown Board Engineered Safety Feature Actuation System and Emergency Diesel Generator loss-of-power instrumentation is inoperable.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by

David E. LaBarge, Senior Project Manager
Project Directorate II-4
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 160 to License No. DPR-77
2. Amendment No. 150 to License No. DPR-79
3. Safety Evaluation

cc w/enclosures:
See next page

NAME	PDII-4/LA	PDII-4/PM	SELBY	EGC	PDII-4/D
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DATE:	6/23/92	6/23/92	7/7/92	7/12/92	7/24/92

DOCUMENT NAME: SQN91-16

AMENDMENT NO.160 FOR SEQUOYAH UNIT NO. 1 - DOCKET NO. 50-327 and
AMENDMENT NO.150 FOR SEQUOYAH UNIT NO. 2 - DOCKET NO. 50-328
DATED: July 24, 1992

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