Docket Nos. 50-327 and 50-328

> Mr. Oliver D. Kingsley, Jr. Senior Vice President, Nuclear Power Tennessee Valley Authority 6N 38A Lookout Place 1101 Market Street Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

ADDITION OF TWO SMOKE DETECTORS TO VOLUME CONTROL ROOM SUBJECT: (TAC NO. 76772) (TS 90-07) - SEQUOYAH NUCLEAR PLANT, UNIT 1

The Commission has issued the enclosed Amendment No. 142 to Facility Operating License No. DPR-77 for the Sequoyah Nuclear Plant, Unit 1. This amendment is in response to your application dated May 4, 1990.

This amendment modifies Table 3.3-11, Fire Detection Instruments, of the Sequoyah Unit 1 Technical Specifications (TSs). The changes add two smoke detectors which were installed in the volume control tank room during the recent Unit 1 Cycle 4 refueling outage.

In your application, you also proposed changes to Table 3.3-11 of the Sequoyah Unit 2 TSs. As you requested, these changes will be issued after these detectors are installed in the Unit 2 volume control tank room during the Unit 2 Cycle 4 refueling outage. This outage is scheduled to begin in October 1990. The enclosed Safety Evaluation for this amendment applies to both units.

The Notice of Issuance for this amendment will be included in the Commission's biweekly Federal Register notice.

Sincerely,

/s/

Jack N. Donohew, Project Manager Project Directorate II-4 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Enclosures:

Amendment No.142 to License No. DPR-77

Safety Evaluation

cc w/enclosures: See next page

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cc: Mr. Marvin Runyon, Chairman Tennessee Valley Authority ET 12A 7A

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# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

### SEQUOYAH NUCLEAR PLANT, UNIT 1

#### AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 142 License No. DPR-77

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 4, 1990, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission:
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment 2. and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

# (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 142, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

This license amendment is effective as of its date of issuance. 3.

FOR THE NUCLEAR REGULATORY COMMISSION

Frederick J. Hebdon, Director Project Directorate II-4

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical Specifications

Date of Issuance: July 27, 1990

# ATTACHMENT TO LICENSE AMENDMENT NO.142

# FACILITY OPERATING LICENSE NO. DPR-77

# DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change. Overleaf pages\* are provided to maintain document completeness.

REMOVE 3/4 6-3 INSERT 3/4 6-3

# TABLE 3.3-11 (Continued)

# FIRE DETECTION INSTRUMENTS

SEQUOYAH		FIRE DETECTION INSTRUMENTS								
3	Fire	Minimum Instruments Operable								
£	Zone Instrument Location		Ionization	Photoelectric						
1			2011/201011	THO COCTCC CTTC	- Incinai	Illiated				
TINU	115	Waste Packaging Area El. 706	3							
Ē	116	Cask Loading Area El. 706								
-	117	Cask Loading Area El. 706	$\bar{2}$							
•	118	New Fuel Storage Area El. 706	2 2 2							
	119	New Fuel Storage Area El. 706	2							
	120	Aux. Bldg. Gas Trtmt. Fltr. El. 714	_	1	1					
	121	Aux. Bldg. Gas Trtmt. Fltr. El. 714		1	1					
	122	Add. Eqpt. Bldg. El. 706 & 717.5	6							
	123	Volume Cont. Tank Rm. 1A, El. 690	1	1						
3/4 3-63 Amendment No. 97, 109	124	Additional Equip. Bldg. El. 706	6							
	125	Volume Cont. Tank Rm. 1A, E1. 690	1	1						
	126	ABGTS Rm. El. 714	2							
	127	ABGTS Rm. El. 714								
	128	ABGTS Rm. El. 714	2 2 2							
	129	ABGTS Rm. El. 714	2							
	130	Ventilation & Purge Air Rm. El. 714	3							
	131	Ventilation & Purge Air Rm. El. 714	3							
	132	Ventilation & Purge Air Rm. El. 714	3							
	133	Ventilation & Purge Air Rm. El. 714	3							
	134	Aux. Bldg. A5-A11, Col. U-W, El. 714	7							
	135	Aux. Bldg. A5-A11, Col. U-W, E1. 714	7							
	136	Heating & Vent. Rm. El. 714	4							
	137	Heating & Vent. Rm. El. 714	4							
	138	Heating & Vent. Rm. El. 714	4							



# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

### **ENCLOSURE 2**

# SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION SUPPORTING AMENDMENT NO. 142 TO FACILITY OPERATING LICENSE NO. DPR-77

# TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-327

## 1.0 INTRODUCTION

By letter dated May 4, 1990, the Tennessee Valley Authority (TVA) proposed changes to Section 3/4.3.3, Monitoring Instrumentation, of the Sequoyah Nuclear Plant, Units 1 and 2, Technical Specifications (TSs). These changes would add two smoke detectors to Table 3.3-11, Fire Detection Instruments. These smoke detectors were to be installed in the Cycle 4 refueling outage for each unit. This is TVA Change Request 90-07.

The two smoke detectors have been installed at Unit 1; therefore, the evaluation given below is on the proposed change to add these detectors to the Unit 1 TSs. The detectors for Unit 2 will be installed during the Unit 2 Cycle 4 refueling which is scheduled to begin in October 1990. The proposed change for Unit 2 will be the subject of a separate evaluation to be issued by the staff after the detectors are installed. The evaluation below discusses the proposed changes to the Unit 1 TSs, but applies also to the changes proposed for the Unit 2 TSs.

#### 2.0 EVALUATION

TVA proposed to revise Table 3.3-11 to reflect the addition of two smoke detectors in the volume control tank room during the Cycle 4 refueling outage for each unit. TVA stated that it is installing the two photoelectric smoke detectors in the room entry labyrinth to provide redundant fire protection in the entry labyrinth. The modification will provide detectors cross-zoned in the same area so that the failure of one smoke detector will not result in a loss of fire detection capability in the room entry labyrinth. TVA stated that the TSs contain surveillance requirements for fire detection instrumentation which protects safety-related equipment and, because the additional smoke detectors in the volume control tank room meet this criteria, it proposed that these detectors be included in the TSs.

The staff reviewed the proposed changes to Table 3.3-11 and agree that the two additional smoke detectors should be included in the table. This is an administrative change which does not revise any fire protection requirements in the TSs except to add the two additional detectors to Table 3.3-11 because the detectors have been installed. Therefore, the staff concludes that the proposed change is acceptable.

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# 3.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

### 4.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (55 FR 24004 on June 13, 1990 and consulted with the State of Tennessee. No public comments were received and the State of Tennessee did not have any comments.

The staff has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: Jack Donohew

Dated: July 27, 1990

AMENDMENT NO. FOR SEQUOYAH UNIT NO. 1 - DOCKET NO. 50-327 DATED: July 27, 1990

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DISTRIBUTION:
Docket File
NRC PDR
Local PDR
Plant Reading File
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cc: Plant Service List