



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 10, 1990

Docket Nos. 50-327
and 50-328

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

SUBJECT: CORRECTIONS TO AMENDMENT NOS. 144 and 125 (TS 90-12) AND AMENDMENT NO. 142 (TS 90-07) (TAC NOS. 76772, 76774, AND 76775) - SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2

By letter dated August 1, 1990, we issued Amendment No. 144 to Facility Operating License DPR-77 and Amendment No. 125 to Facility Operating License DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. The first page of the letter had a statement about restrictions on the burnup of fuel assemblies which may be placed in the spent fuel pool storage racks which is incorrect, and Page 3/4 9-1 of the amended Unit 1 Technical Specifications had an error. The correct statement that fuel assemblies with enrichments greater than 4.0 w/o and burnups less than 7500 MWD/MTU may be placed in spent fuel pool storage rack locations that face adjacent cells filled with water or fuel assemblies with at least 22,000 MWD/MTU of burnup is in the amended Technical Specifications and Safety Evaluation enclosed with the letter dated August 1, 1990. The criticality analysis submitted in the licensee's letter dated February 14, 1990 justified the burnup values of 6,750 and 20,000 MWD/MTU that were stated in the letter; however, the licensee proposed the values of 7,500 and 22,000 MWD/MTU for the Technical Specifications for additional conservatism. The error on Page 3/4 9-1 does not have anything to do with the amendment and is in the statement of "20,000 ppm boron" in the action statement for Limiting Condition For Operation 3.9.1 instead of the correct statement "20,000 ppm boron." The corrected pages are enclosed.

In the letter dated July 27, 1990, we issued Amendment No. 142 to Facility Operating License DPR-77 for the Sequoyah Nuclear Plant, Unit 1. The amended Page 3/4 3-63 of the Unit 1 Technical Specifications was inadvertently issued without the amendment number on the page. The corrected page is enclosed.

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Mr. Oliver D. Kingsley, Jr.

-2-

We apologize for any inconvenience caused by these errors. If you have any questions, please contact Jack Donohew, Sequoyah Project Manager, at 301-492-1308.

Sincerely,

Original signed by

Frederick J. Hebdon, Director
Project Directorate II-4
Division of Reactor Projects I/II
Office of Nuclear Reactor Regulation

Enclosures:
Corrected Pages

cc w/enclosure:
See next page

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Mr. Oliver D. Kingsley, Jr.

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Sequoyah

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 1, 1990

Docket Nos. 50-327
and 50-328

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

SUBJECT: INCREASE FUEL ENRICHMENT TO 5.0 WEIGHT PERCENT (TAC NOS. 76074,
76075, 76774, 76775) (TS 90-12) - SEQUOYAH NUCLEAR PLANT, UNITS 1
AND 2

The Commission has issued the enclosed Amendment No. 144 to Facility Operating License No. DPR-77 and Amendment No. 125 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated May 4, 1990.

The amendments modify the Sequoyah Nuclear Plant, Units 1 and 2, Technical Specifications (TSs) to increase the maximum enrichment of fuel allowed on the site from 4.0 to 5.0 weight percent (w/o) Uranium 235. Changes have been made to Section 5.0, Design Features, and Surveillance Requirement 4.9.1.4 on the boron concentration in the spent fuel storage pool has been added to the TSs. As stated in TS 5.6.1.2, new fuel with an enrichment greater than 4.5 weight percent may not be stored in the new fuel pit storage racks. This fuel may, based on these amendments, be stored in the spent fuel pool storage racks. Fuel assemblies with enrichments greater than 4.0 w/o and burnups less than 7500 MWD/MTU may be placed in spent fuel pool storage rack locations that face adjacent cells filled with water or fuel assemblies with at least 22,000 MWD/MTU of burnup. This amendment is based on the criticality analysis for the spent fuel storage pool which was submitted in your letter dated February 14, 1990.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

CORRECTED PAGES

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-63

3/4 9-1

INSERT

3/4 3-63

3/4 9-1

TABLE 3.3-11 (Continued)

FIRE DETECTION INSTRUMENTS

Fire Zone	Instrument Location	Minimum Instruments Operable			
		Ionization	Photoelectric	Thermal	Infrared
115	Waste Packaging Area El. 706	3			
116	Cask Loading Area El. 706	2			
117	Cask Loading Area El. 706	2			
118	New Fuel Storage Area El. 706	2			
119	New Fuel Storage Area El. 706	2			
120	Aux. Bldg. Gas Trtmt. Fltr. El. 714		1	1	
121	Aux. Bldg. Gas Trtmt. Fltr. El. 714		1	1	
122	Add. Eqpt. Bldg. El. 706 & 717.5	6			
123	Volume Cont. Tank Rm. 1A, El. 690	1	1		
124	Additional Equip. Bldg. El. 706	6			
125	Volume Cont. Tank Rm. 1A, El. 690	1	1		
126	ABGTS Rm. El. 714	2			
127	ABGTS Rm. El. 714	2			
128	ABGTS Rm. El. 714	2			
129	ABGTS Rm. El. 714	2			
130	Ventilation & Purge Air Rm. El. 714	3			
131	Ventilation & Purge Air Rm. El. 714	3			
132	Ventilation & Purge Air Rm. El. 714	3			
133	Ventilation & Purge Air Rm. El. 714	3			
134	Aux. Bldg. A5-A11, Col. U-W, El. 714	7			
135	Aux. Bldg. A5-A11, Col. U-W, El. 714	7			
136	Heating & Vent. Rm. El. 714	4			
137	Heating & Vent. Rm. El. 714	4			
138	Heating & Vent. Rm. El. 714	4			

SEQUOYAH - UNIT 1

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Amendment No. 97, 109, 142

3/4.9 REFUELING OPERATIONS

3/4.9.1 BORON CONCENTRATION

LIMITING CONDITION FOR OPERATION

3.9.1 With the reactor vessel head closure bolts less than fully tensioned or with the head removed, the boron concentration of all filled portions of the Reactor Coolant System and the refueling canal shall be maintained uniform and sufficient to ensure that the more restrictive of the following reactivity conditions is met:

- a. Either a K_{eff} of 0.95 or less, which includes a 1% delta k/k conservative allowance for uncertainties, or
- b. A boron concentration of greater than or equal to 2000 ppm, which includes a 50 ppm conservative allowance for uncertainties.

APPLICABILITY: MODE 6*

ACTION:

With the requirements of the above specification not satisfied, immediately suspend all operations involving CORE ALTERATIONS or positive reactivity changes and initiate and continue boration at greater than or equal to 10 gpm of a solution containing greater than or equal to 20,000 ppm boron or its equivalent until K_{eff} is reduced to less than or equal to 0.95 or the boron concentration is restored to greater than or equal to 2000 ppm, whichever is the more restrictive. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.9.1.1 The more restrictive of the above two reactivity conditions shall be determined prior to:

- a. Removing or unbolting the reactor vessel head, and
- b. Withdrawal of any full length control rod in excess of 3 feet from its fully inserted position within the reactor pressure vessel.

4.9.1.2 The boron concentration of the reactor coolant system and the refueling canal shall be determined by chemical analysis at least once per 72 hours.

*The reactor shall be maintained in MODE 6 whenever fuel is in the reactor vessel with the vessel head closure bolts less than fully tensioned or with the head removed.