



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555

May 26, 1989

Docket Nos. 50-327
and 50-328

Mr. Oliver D. Kingsley, Jr.
Senior Vice President, Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

SUBJECT: CORRECTIONS TO AMENDMENT NOS. 111 AND 101 (TS 88-25) (TAC R00501/
R00502) - SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2

By letter dated April 3, 1989, we issued Amendment No. 111 to Facility Operating License DPR-77 and Amendment No. 101 to Facility Operating License DPR-79, for Sequoyah Nuclear Plant, Units 1 and 2. In these amendments, pages 3/4 4-4a and 3/4 4-4b (Unit 1), and pages 3/4 4-8 and 3/4 4-8a (Unit 2) had errors which did not involve the changes discussed in the amendments. The corrected pages are enclosed. The error was that these pages included surveillance requirement 4.4.3.2.3 which had been deleted from both units in Amendments 105 and 94, dated March 9, 1989, respectively.

In Amendment 109 dated April 3, 1989 to Operating License DPR-77 for Sequoyah, Unit 1, we issued page 3/4 11-14 with an incorrect amendment number on the page. This page was an overleaf page and not part of the changes discussed in the amendment. The corrected page is enclosed.

In Amendment 104, dated May 5, 1989, to Operating License DPR-79 for Sequoyah, Unit 2, we issued Page 3/4 3-4 with the Power Range Neutron Flux Interlock listed as item 22.D, instead of item 22.F as requested by the licensee, in Table 3.3-1. This correction to item 22.F is needed (1) to have the Unit 2 Technical Specifications consistent with the Unit 1 Technical Specifications and (2) to have the licensee's current surveillance instructions correctly identify the interlock in the Unit 2 Technical Specifications. This correction does not affect the safety evaluation issued by the staff to approve Amendment No. 104.

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CP
JH

Mr. Oliver D. Kingsley, Jr.

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We apologize for any inconvenience caused by these errors. If you have any questions, please contact Jack Donohew, Sequoyah Project Manager at 301/492-0704.

Sincerely,

Original signed by Rajender Auluck

for Suzanne Black, Assistant Director
for Projects
TVA Projects Division
Office of Nuclear Reactor Regulation

Enclosures:

1. Corrected pages for Sequoyah Unit 1
2. Corrected pages for Sequoyah Unit 2

cc w/ enclosures
See next page

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Mr. Oliver D. Kingsley, Jr.

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CORRECTED PAGES

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 4-4a
3/4 4-4b
3/4 11-14

INSERT

3/4 4-4a,
3/4 4-4b
3/4 11-14

REACTOR COOLANT SYSTEM

RELIEF VALVES - OPERATING

LIMITING CONDITION FOR OPERATION

3.4.3.2 Two power relief valves (PORVs) and their associated block valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one or more PORV(s) inoperable, but capable of RCS pressure control, within 1 hour either restore the PORV(s) to OPERABLE status or close the associated block valve(s); otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one PORV inoperable and incapable of RCS pressure control, within 1 hour either restore the PORV to OPERABLE status or close the associated block valve and remove power from the block valve; restore the PORV to OPERABLE status within the following 72 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With both PORVs inoperable and incapable of RCS pressure control, within 1 hour either restore each of the PORVs to OPERABLE status or close their associated block valves and remove power from the block valves and be in HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours.
- d. With one or more block valve(s) inoperable, within 1 hour: (1) restore the block valve(s) to OPERABLE status, or close the block valve(s) and remove power from the block valve(s), or close the PORV(s) and remove power from its associated solenoid valve(s); and (2) apply the ACTION b. or c. above, as appropriate, for the isolated PORV(s).
- e. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE at least once per 18 months by:

- a. Performance of a CHANNEL CALIBRATION, and
- b. by operating the valve through one complete cycle of full travel.

4.4.3.2.2 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.3.2.3 (Deleted.)

4.4.3.2.4 In addition to the requirements of Specification 4.0.5 the repair welds and adjoining areas of the pressurizer relief line shall be examined, using improved UT detection and evaluation procedures which have been demonstrated to be effective in detecting IGSCC, prior to entering MODE 4 whenever the plant has been in COLD SHUTDOWN for 72 hours or more if the examination has not been performed in the previous 6 months.

In the event these 6-month period examinations find the piping free of unacceptable indications for 3 successive inspections, the inspection interval shall be extended to 36 month intervals (± 12 months to coincide with a scheduled refueling outage).

In the event these 36-month period examinations find the piping free of unacceptable indications for 3 successive inspections, the inspection interval shall be extended to 80-month periods.

TABLE 4.11-2 (Continued)

TABLE NOTATION

- j. Venting - Applicable in MODES 1, 2, 3, and 4; the containment will be vented to the containment annulus and then to the auxiliary building via containment annulus fans. The lower containment compartment shall be sampled daily when venting is to occur to account for the radioactivity being discharged from the venting process.

CORRECTED PAGES

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 3-4
3/4 4-8
3/4 4-8a

INSERT

3/4 3-4
3/4 4-8
- -

TABLE 3.3-1 (Continued)

REACTOR TRIP SYSTEM INSTRUMENTATION

| <u>FUNCTIONAL UNIT</u> | <u>TOTAL NO. OF CHANNELS</u> | <u>CHANNELS TO TRIP</u> | <u>MINIMUM CHANNELS OPERABLE</u> | <u>APPLICABLE MODES</u> | <u>ACTION</u> |
|----------------------------------------------|----------------------------------|-----------------------------|------------------------------------------|-----------------------------|---------------|
| 19. Safety Injection Input from ESF | 2 | 1 | 2 | 1, 2 | 12 |
| 20. Reactor Trip Breakers | | | | | |
| A. Startup and Power Operation | 2 | 1 | 2 | 1, 2 | 12, 15 |
| B. Shutdown | 2 | 1 | 2 | 3*, 4* and 5* | 16 |
| 21. Automatic Trip Logic | | | | | |
| A. Startup and Power Operation | 2 | 1 | 2 | 1, 2 | 12 |
| B. Shutdown | 2 | 1 | 2 | 3*, 4* and 5* | 16 |
| 22. Reactor Trip System Interlocks | | | | | |
| A. Intermediate Range Neutron Flux, P-6 | 2 | 1 | 2 | 2, and* | 8a |
| B. Power Range Neutron Flux, P-7 | 4 | 2 | 3 | 1 | 8b |
| C. Power Range Neutron Flux, P-8 | 4 | 2 | 3 | 1 | 8c |
| D. Power Range Neutron Flux, P-10 | 4 | 2 | 3 | 1, 2 | 8d |
| E. Turbine Impulse Chamber Pressure, P-13 | 2 | 1 | 2 | 1 | 8b |
| F. Power Range Neutron Flux, P-9 | 4 | 2 | 3 | 1 | 8e |
| G. Reactor Trip, P-4 | 2 | 1 | 2 | 1, 2, and * | 14 |

SEQUOYAH UNIT 2

3/4 3-4

Amendment No. 46, 48, 104
(Correction letter of 8-24-87)

REACTOR COOLANT SYSTEM

RELIEF VALVES - OPERATING

LIMITING CONDITION FOR OPERATION

3.4.3.2 All power operated relief valves (PORVs) and their associated block valves shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With one or more PORV(s) inoperable, but capable of RCS pressure control, within 1 hour either restore the PORV(s) to OPERABLE status or close the associated block valve(s); otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With one PORV inoperable and incapable of RCS pressure control, within 1 hour either restore the PORV to OPERABLE status or close the associated block valve and remove power from the block valve; restore the PORV to OPERABLE status within the following 72 hours or be in HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With both PORVs inoperable and incapable of RCS pressure control, within 1 hour either restore each of the PORVs to OPERABLE status or close their associated block valves and remove power from the block valves and be in HOT STANDBY within the next 6 hours and COLD SHUTDOWN within the following 30 hours.
- d. With one or more block valve(s) inoperable, within 1 hour:
(1) restore the block valve(s) to OPERABLE status, or close the block valve(s) and remove power from the block valve(s), or close the PORV(s) and remove power from its associated solenoid valve(s); and (2) apply the ACTION b. or c. above, as appropriate, for the isolated PORV(s).
- e. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.4.3.2.1 In addition to the requirements of Specification 4.0.5, each PORV shall be demonstrated OPERABLE at least once per 18 months by:

- a. Performance of a CHANNEL CALIBRATION, and
- b. Operating the valve through one complete cycle of full travel.

4.4.3.2.2 Each block valve shall be demonstrated OPERABLE at least once per 92 days by operating the valve through one complete cycle of full travel.