

March 15, 1989

Docket Nos. 50-327/328

Mr. Oliver D. Kingsley, Jr.  
Senior Vice President, Nuclear Power  
Tennessee Valley Authority  
6N 38A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

Dear Mr. Kingsley:

SUBJECT: REPLACEMENT PAGES FOR AMENDMENT NO. 102 (UNIT 1) (TAC R00503),  
AMENDMENT NO. 91 (UNIT 2) (TAC R00504) AND AMENDMENT NO. 92  
(UNIT 2) (TAC R00043)

Due to administrative error, the amendment numbers were entered inadvertently on pages 3/4 6-4 and B 3/4 6-2 of Amendment No. 102 for Unit 1 and on page B 3/4 6-2 of Amendment No. 91 of Unit 2. These pages were not changed by Amendment Nos. 102 or 91, respectively. Amendment number 93 was typed on the Attachment to License for Amendment No. 92. The corrected pages are enclosed.

We apologize for any inconvenience these administrative errors may have caused.

Sincerely,

Original signed by

Suzanne Black, Assistant Director  
for Projects  
TVA Projects Division  
Office of Nuclear Reactor Regulation

Enclosures:

- 1. Page 3/4 6-4 (Unit 1)
- 2. Pages B 3/4 6-2 Units 1 and 2)
- 3. Attachment to License  
Amendment No. 92

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Mr. Oliver D. Kingsley, Jr.

-2-

Sequoyah Nuclear Plant

cc:

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Dr. Henry Myers, Science Advisor  
Committee on Interior  
and Insular Affairs  
U.S. House of Representatives  
Washington, D.C. 20515

Tennessee Valley Authority  
Rockville Office  
11921 Rockville Pike  
Suite 402  
Rockville, Maryland 20852

## CONTAINMENT SYSTEMS

### SURVEILLANCE REQUIREMENTS (Continued)

- e. The combined bypass leakage rate to the auxiliary building shall be determined to be less than or equal to  $0.25 L_a$  by applicable Type B and C tests at least once per 24 months except for penetrations which are not individually testable; penetrations not individually testable shall be determined to have no detectable leakage when tested with soap bubbles while the containment is pressurized to  $P_a$  (12 psig) during each Type A test.
- f. Air locks shall be tested and demonstrated OPERABLE per Surveillance Requirement 4.6.1.3.
- g. Leakage from isolation valves that are sealed with fluid from a seal system may be excluded, subject to the provisions of Appendix J, Section III.C.3, when determining the combined leakage rate provided the seal system and valves are pressurized to at least  $1.10 P_a$  (13.2 psig) and the seal system capacity is adequate to maintain system pressure (or fluid head for the containment spray system and RHR spray system valves at penetrations 48A, 48B, 49A and 49B) for at least 30 days.
- h. Type B tests for penetrations employing a continuous leakage monitoring system shall be conducted at  $P_a$  (12 psig) at intervals no greater than once per 3 years.
- i. All test leakage rates shall be calculated using observed data converted to absolute values. Error analyses shall be performed to select a balanced integrated leakage measurement system.
- j. The provisions of Specification 4.0.2 are not applicable.

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## CONTAINMENT SYSTEMS

### BASES

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containment peak pressure does not exceed the maximum allowable internal pressure of 12 psig during LOCA conditions.

#### 3/4.6.1.5 AIR TEMPERATURE

The limitations on containment average air temperature ensure that 1) the containment air mass is limited to an initial mass sufficiently low to prevent exceeding the maximum allowable internal pressure during LOCA conditions and 2) the ambient air temperature does not exceed that temperature allowable for the continuous duty rating specified for equipment and instrumentation located within containment.

The containment pressure transient is sensitive to the initially contained air mass during a LOCA. The contained air mass increases with decreasing temperature. The lower temperature limits of 100°F for the lower compartment, 85°F for the upper compartment, and 60°F when less than or equal to 5% of RATED THERMAL POWER will limit the peak pressure to an acceptable value. The upper temperature limit influences the peak accident temperature slightly during a LOCA; however, this limit is based primarily upon equipment protection and anticipated operating conditions. Both the upper and lower temperature limits are consistent with the parameters used in the accident analyses.

#### 3/4.6.1.6 CONTAINMENT VESSEL STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment steel vessel will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to ensure that the vessel will withstand the maximum pressure of 12 psig in the event of a LOCA. A visual inspection in conjunction with Type A leakage tests is sufficient to demonstrate this capability.

#### 3/4.6.1.7 SHIELD BUILDING STRUCTURAL INTEGRITY

This limitation ensures that the structural integrity of the containment shield building will be maintained comparable to the original design standards for the life of the facility. Structural integrity is required to provide 1) protection for the steel vessel from external missiles, 2) radiation shielding in the event of a LOCA, and 3) an annulus surrounding the steel vessel that can be maintained at a negative pressure during accident conditions.

## CONTAINMENT SYSTEMS

### BASES

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#### INTERNAL PRESSURE (Continued)

containment peak pressure does not exceed the maximum allowable internal pressure of 12 psig during LOCA conditions.

#### 3/4.6.1.5 AIR TEMPERATURE

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ATTACHMENT TO LICENSE AMENDMENT NO. 92

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 6-14

INSERT

3/4 6-14

Corrected 3/13/89