

October 14, 1988

Docket Nos. 50-327/328

Mr. S. A. White  
Senior Vice President, Nuclear Power  
Tennessee Valley Authority  
6N 38A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

Dear Mr. White:

SUBJECT: DELETION OF TECHNICAL SPECIFICATION TABLE 4.4-5 (TAC R00151/R00152)  
(TS 87-06) SEQUOYAH NUCLEAR PLANTS, UNITS 1 AND 2

The Commission has issued the enclosed Amendment No. 87 to Facility Operating License No. DPR-77 for the Sequoyah Nuclear Plant, Unit 1. This amendment is in response to your application dated May 29, 1987.

This amendment revises the surveillance requirements (SR) on pressure/temperature limits in the Sequoyah Unit 1 Technical Specifications (TS). The changes delete (1) Table 4.4-5, "Reactor Vessel Material Surveillance Program - Withdrawal Schedule," and (2) references to the Table in SR 4.4.9.1.2.

This change has not been proposed for Sequoyah, Unit 2. This change would be an enhancement for the Unit 2 TS; therefore, you are requested to propose a similar change for Unit 2.

A copy of the Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's Bi-Weekly Federal Register Notice.

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

Original Signed by  
Suzanne Black, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Enclosures:

1. Amendment No. 87 to License No. DPR-77
2. Safety Evaluation

cc w/enclosures:  
See next page

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STS group and OGC enforcement did not disagree w/ th Action

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY  
DOCKET NO. 50-327  
SEQUOYAH NUCLEAR PLANT, UNIT 1  
AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 87  
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Tennessee Valley Authority (the licensee) dated May 29, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 87, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Suzanne Black, Assistant Director  
for Projects  
TVA Projects Division  
Office of Special Projects

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: October 14, 1987

ATTACHMENT TO LICENSE AMENDMENT NO. 87

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Revise the Appendix A Technical Specifications by removing the pages identified below and inserting the enclosed pages. The revised pages are identified by the captioned amendment number and contain marginal lines indicating the area of change.

REMOVE

3/4 4-23  
3/4 4-23a

INSERT

3/4 4-23  
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## REACTOR COOLANT SYSTEM

### 3/4.4.9 PRESSURE/TEMPERATURE LIMITS

## REACTOR COOLANT SYSTEM

### LIMITING CONDITION FOR OPERATION

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3.4.9.1 The Reactor Coolant System (except the pressurizer) temperature and pressure shall be limited in accordance with the limit lines shown on Figures 3.4-2 and 3.4-3 during heatup, cooldown, criticality, and inservice leak and hydrostatic testing with:

- a. A maximum heatup of 100°F in any one hour period.
- b. A maximum cooldown of 100°F in any one hour period.
- c. A maximum temperature change of less than or equal to 5°F in any one hour period during inservice hydrostatic and leak testing operations above the heatup and cooldown limit curves.

APPLICABILITY: At all times.

#### ACTION:

With any of the above limits exceeded, restore the temperature and/or pressure to within the limit within 30 minutes; perform an engineering evaluation to determine the effects of the out-of-limit condition on the structural integrity of the Reactor Coolant System; determine that the Reactor Coolant System remains acceptable for continued operations or be in at least HOT STANDBY within the next 6 hours and reduce the RCS  $T_{avg}$  and pressure to less than 200°F and 500 psig, respectively, within the following 30 hours.

### SURVEILLANCE REQUIREMENTS

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4.4.9.1.1 The Reactor Coolant System temperature and pressure shall be determined to be within the limits at least once per 30 minutes during system heatup, cooldown, and inservice leak and hydrostatic testing operations.

4.4.9.1.2 The reactor vessel material irradiation surveillance specimens shall be removed and examined, to determine change in material properties, in accordance with 10 CFR 50, Appendix H. The results of these examinations shall be used to update Figures 3.4-2 and 3.4-3.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF SPECIAL PROJECTS

SUPPORTING AMENDMENT NO. 87 TO FACILITY OPERATING LICENSE NO. DPR-77

TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT, UNIT 1

DOCKET NO. 50-327

1.0 INTRODUCTION

By a letter dated May 29, 1987, the Tennessee Valley Authority (TVA) proposed change (87-06) to the Technical Specifications (TS) for Sequoyah Unit 1. These changes would delete (1) Table 4.4-5, "Reactor Vessel Material Surveillance Program - Withdrawal Schedule," and (2) all references to the Table 4.4-5 in the Surveillance Requirement (SR) 4.4.9.1.2.

2.0 DISCUSSION

TVA is required to comply with 10 CFR Part 50, Appendix H, "Reactor Vessel Material Surveillance Program Requirements." The requirements of Appendix H provide for NRC approval of a proposed withdrawal schedule for the reactor vessel material irradiation surveillance specimens. Thus, the inclusion of Table 4.4-5 in SR 4.4.9.1.2 is redundant since this table is provided to satisfy the requirements of 10 CFR Part 50, Appendix H. The deletion of Table 4.4-5 is consistent with an improvement of the TS by removing specifications that are redundant to regulations, as recommended by the NRC Technical Specification Improvement Project and the Atomic Industrial Forum Subcommittee on TS Improvements. The regulation is Appendix H to 10 CFR Part 50 in this case.

The data of Table 4.4-5 would be placed in the Sequoyah Final Safety Analysis Report (FSAR) Section 5.4.3.7, "Reactor Vessel Material Surveillance Program Requirements." This would occur at the next scheduled annual FSAR update. Thus, the surveillance schedule would be available to NRC in an administratively controlled document that is reviewed and revised on a regular basis.

3.0 EVALUATION

TVA has proposed to delete Table 4.4.-5, "Reactor Vessel Material Surveillance Program - Withdrawal Schedule" and references to the Table in SR 4.4.9.1.2 due to the redundancy with the requirements of 10 CFR Part 50, Appendix H. The staff has reviewed the proposed TS amendment and finds the changes acceptable.

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#### 4.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change to a requirement with respect to the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and changes to the surveillance requirements. The staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement nor environmental assessment need be prepared in connection with the issuance of this amendment.

#### 5.0 CONCLUSION

The Commission made a proposed determination that the amendment involves no significant hazards consideration which was published in the Federal Register (53 FR 13018) on April 20, 1988 and consulted with the State of Tennessee on October 12, 1988. No public comments were received and the State of Tennessee did not have any comments

We have concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of this amendment will not be inimical to the common defense and security nor to the health and safety of the public.

Principal Contributor: B. K. Singh

Dated: October 14, 1988