

April 8, 1998

Mr. H. B. Barron
Vice President, McGuire Site
Duke Energy Corporation
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. MA0431 and MA0432)

Dear Mr. Barron:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 178 to Facility Operating License NPF-9 and Amendment No. 160 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated December 17, 1997.

The amendments revise TS Section 6.9.1.9 to reference updated or recently approved topical reports, which contain methodologies used to calculate cycle-specific limits contained in the Core Operating Limits Report. For several reports, the licensee indicated staff approval, but neglected to provide an "A" designation for the report number. Upon agreement by the licensee, the staff has made these appropriate editorial corrections. These topical reports have all been previously approved by the staff under licensing actions separate from the current amendment request.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY:

Frank Rinaldi, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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PDR ADDCK 05000369
P PDR

Docket Nos. 50-369 and 50-370

Enclosures:

1. Amendment No. 178 to NPF-9
2. Amendment No. 160 to NPF-17
3. Safety Evaluation

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cc w/encl: See next page

DOCUMENT NAME: G:\MCGUIRE\MA0431.WPD

OFFICE	PDII-2/PM	PDII-2/LA	OGC	PDII-2/D
NAME	F.RINALDI:cn	L.BERRY	OGC	H.BERKOW
DATE	3/13/98	3/13/98	3/31/98	4/1/98
COPY	YES NO	YES NO	YES NO	YES

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 8, 1998

Mr. H. B. Barron
Vice President, McGuire Site
Duke Energy Corporation
12700 Hagers Ferry Road
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AND 2 (TAC NOS. MA0431 and MA0432)

Dear Mr. Barron:

The Nuclear Regulatory Commission has issued the enclosed Amendment No.178 to Facility Operating License NPF-9 and Amendment No. 160 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated December 17, 1997.

The amendments revise TS Section 6.9.1.9 to reference updated or recently approved topical reports, which contain methodologies used to calculate cycle-specific limits contained in the Core Operating Limits Report. For several reports, the licensee indicated staff approval, but neglected to provide an "A" designation for the report number. Upon agreement by the licensee, the staff has made these appropriate editorial corrections. These topical reports have all been previously approved by the staff under licensing actions separate from the current amendment request.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script, appearing to read "Frank Rinaldi".

Frank Rinaldi, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosures:

1. Amendment No. 178 to NPF-9
2. Amendment No. 160 to NPF-17
3. Safety Evaluation

cc w/encls: See next page

McGuire Nuclear Station

cc:

Mr. Paul R. Newton
Legal Department (PBO5E)
Duke Energy Corporation
422 South Church Street
Charlotte, North Carolina 28242

County Manager of
Mecklenburg County
720 East Fourth Street
Charlotte, North Carolina 28202

Michael T. Cash
Regulatory Compliance Manager
Duke Energy Corporation
McGuire Nuclear Site
12700 Hagers Ferry Road
Huntersville, North Carolina 28078

J. Michael McGarry, III, Esquire
Winston and Strawn
1400 L Street, NW.
Washington, DC 20005

Senior Resident Inspector
c/o U.S. Nuclear Regulatory
Commission
12700 Hagers Ferry Road
Huntersville, North Carolina 28078

Mr. Peter R. Harden, IV
Account Sales Manager
Westinghouse Electric Corporation
Power Systems Field Sales
P. O. Box 7288
Charlotte, North Carolina 28241

Dr. John M. Barry
Mecklenburg County
Department of Environmental
Protection
700 N. Tryon Street
Charlotte, North Carolina 28202

Ms. Karen E. Long
Assistant Attorney General
North Carolina Department of
Justice
P. O. Box 629
Raleigh, North Carolina 27602

L. A. Keller
Manager - Nuclear Regulatory
Licensing
Duke Energy Corporation
526 South Church Street
Charlotte, North Carolina 28242

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission
Atlanta Federal Center
61 Forsyth Street, S.W., Suite 23T85
Atlanta, Georgia 30303

Elaine Wathen, Lead REP Planner
Division of Emergency Management
116 West Jones Street
Raleigh, North Carolina 27603-1335

Mr. Richard M. Fry, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health and Natural
Resources
3825 Barrett Drive
Raleigh, North Carolina 27609-7721

Mr. T. Richard Puryear
Owners Group (NCEMC)
Duke Energy Corporation
4800 Concord Road
York, South Carolina 29745



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 178
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Facility Operating License No. NPF-9 filed by the Duke Energy Corporation (licensee) dated December 17, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 178 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 8, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 178

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

6-21
6-22

Insert

6-21
6-22

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT

The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by NRC in:

1. WCAP-9272-P-A, "WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY," July 1985 (W Proprietary).

(Methodology for Specifications 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limit, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
2. WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL FQ SURVEILLANCE TECHNICAL SPECIFICATION", June 1983 (W Proprietary).

(Methodology for Specifications 3.2.1 - Axial Flux Difference (Relaxed Axial Offset Control) and 3.2.2 - Heat Flux Hot Channel Factor (W(Z) surveillance requirements for F_Q Methodology.)
3. WCAP-10266-P-A Rev. 2, "THE 1981 VERSION OF WESTINGHOUSE EVALUATION MODEL USING BASH CODE", March 1987 (W Proprietary).

(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
4. BAW-10168P-A, "B&W Loss-of-Coolant Accident Evaluation Model for Recirculating Steam Generator Plants," Rev. 1, SER dated January 1991; Rev. 2, SER dated August 22, 1996; Rev. 3, SER dated June 15, 1994. (B&W Proprietary).

(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
5. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March 1990 (DPC Proprietary).

(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
6. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November 1991 (DPC Proprietary).

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
7. DPC-NE-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June 1985

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, Specification 3.9.1 - RCS and Refueling Canal Boron Concentration, and Specification 3/4.9.12 - Spent Fuel Pool Boron Concentration.)

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT

8. DPC-NE-3002A, through Rev 2, "FSAR Chapter 15 System Transient Analysis Methodology," SER dated April 26, 1996.

(Methodology used in the system thermal-hydraulic analyses which determine the core operating limits)
9. DPC-NE-3000P-A, Rev. 1, "Thermal-Hydraulic Transient Analysis Methodology," SER dated December 27, 1995.

(Modeling used in the system thermal-hydraulic analyses)
10. DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," SER dated April 26, 1996.

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)
11. DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," SER dated February 20, 1997 (DPC Proprietary).

(Methodology for Specifications 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.2.1 - Axial Flux Difference (AFD), and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor $F\Delta H(X,Y)$.)
12. DPC-NE-2001P-A, Rev. 1, "Fuel Mechanical Reload Analysis Methodology for Mark-BW fuel," October 1990 (DPC Proprietary).

(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints.)
13. DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology," SER dated November 7, 1996 (DPC Proprietary).

(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, Specification 3.2.1 - Axial Flux Difference, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
14. DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," SER dated April 3, 1995 (DPC Proprietary).

(Methodology used for Specification 2.2.1 - Reactor Trip System Instrumentation setpoints).
15. BAW-10183P-A, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, as approved by SER dated July, 1995.

(Used for Specification 2.2.1, Reactor Trip System Instrumentation Setpoints).

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

DUKE ENERGY CORPORATION

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 160
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Energy Corporation (licensee) dated December 17, 1997, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 160, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: April 8, 1998

ATTACHMENT TO LICENSE AMENDMENT NO. 160

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove

6-21

6-22

Insert

6-21

6-22

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT

The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by NRC in:

1. WCAP-9272-P-A, "WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY," July 1985 (W Proprietary).

(Methodology for Specifications 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limit, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
2. WCAP-10216-P-A, "RELAXATION OF CONSTANT AXIAL OFFSET CONTROL FQ SURVEILLANCE TECHNICAL SPECIFICATION", June 1983 (W Proprietary).

(Methodology for Specifications 3.2.1 - Axial Flux Difference (Relaxed Axial Offset Control) and 3.2.2 - Heat Flux Hot Channel Factor (W(Z) surveillance requirements for F_Q Methodology.)
3. WCAP-10266-P-A Rev. 2, "THE 1981 VERSION OF WESTINGHOUSE EVALUATION MODEL USING BASH CODE", March 1987 (W Proprietary).

(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
4. BAW-10168P-A, "B&W Loss-of-Coolant Accident Evaluation Model for Recirculating Steam Generator Plants," Rev. 1, SER dated January 1991; Rev. 2, SER dated August 22, 1996; Rev. 3, SER dated June 15, 1994 (B&W Proprietary).

(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor.)
5. DPC-NE-2011PA, "Duke Power Company Nuclear Design Methodology for Core Operating Limits of Westinghouse Reactors," March 1990 (DPC Proprietary).

(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
6. DPC-NE-3001PA, "Multidimensional Reactor Transients and Safety Analysis Physics Parameter Methodology," November 1991 (DPC Proprietary).

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Rod Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
7. DPC-NE-2010A, "Duke Power Company McGuire Nuclear Station Catawba Nuclear Station Nuclear Physics Methodology for Reload Design," June 1985

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, Specification 3.9.1 - RCS and Refueling Canal Boron Concentration, and Specification 3/4.9.12 - Spent Fuel Pool Boron Concentration.)

ADMINISTRATIVE CONTROLS

CORE OPERATING LIMITS REPORT

8. DPC-NE-3002A, Through Rev. 2, "FSAR Chapter 15 System Transient Analysis Methodology," SER dated April 26, 1996.
(Methodology used in the system thermal-hydraulic analyses which determine the core operating limits)
9. DPC-NE-3000P-A, Rev. 1, "Thermal-Hydraulic Transient Analysis Methodology," SER dated December 27, 1995.
(Modeling used in the system thermal-hydraulic analyses)
10. DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P," SER dated April 26, 1996.
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient.)
11. DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01," SER dated February 20, 1997 (DPC Proprietary).
(Methodology for Specifications 2.2.1 - Reactor Trip System Instrumentation Setpoints, 3.2.1 - Axial Flux Difference (AFD), and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor $F_{\Delta H}(X,Y)$.)
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(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints.)
13. DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology," SER dated November 7, 1996 (DPC Proprietary).
(Methodology for Specification 2.2.1 - Reactor Trip System Instrumentation Setpoints, Specification 3.2.1 - Axial Flux Difference, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.)
14. DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," SER dated April 3, 1995 (DPC Proprietary).
(Methodology used for Specification 2.2.1 - Reactor Trip System Instrumentation setpoints).
15. BAW-10183P-A, Fuel Rod Gas Pressure Criterion, B&W Fuel Company, as approved by SER dated July, 1995.
(Used for Specification 2.2.1, Reactor Trip System Instrumentation Setpoints).

The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, ECCS limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements thereto, shall be provided upon issuance, for each reload cycle, to the NRC Document Control Desk with copies to the Regional Administrator and Resident Inspector.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 178 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 160 TO FACILITY OPERATING LICENSE NPF-17

DUKE ENERGY CORPORATION

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated December 17, 1997, Duke Energy Corporation (the licensee) submitted a request for changes to the McGuire Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would revise TS Section 6.9.1.9 to reference updated or recently approved topical reports, which contain methodologies used to calculate cycle-specific limits contained in the Core Operating Limits Report (COLR). These topical reports have all been previously approved by the staff under licensing actions separate from the current amendment request.

2.0 EVALUATION

Generic Letter 88-16 provided guidance on removing cycle-specific parameters that are calculated using NRC-approved methodologies from the TS. The parameters are to be replaced in the TS with a reference to the approved report. By letter dated December 17, 1997, the licensee proposed changes that reflect the current versions of these reports.

The licensee has proposed to update the listing for the COLR in TS Section 6.9.1.9, as follows:

Topical Report BAW-10168P-A, "B&W Loss-of-Coolant Accident Evaluation Model for Recirculating Steam Generator Plants" - The licensee proposed to add Revision 2 (approved by letter dated August 22, 1996, R. Jones, NRC, to J. H. Taylor) and Revision 3 (approved by letter dated June 15, 1994, M. Virgilio, NRC, to J. H. Taylor) to the reference. The licensee's amendment request inadvertently omitted the "-A" designation at the end of the topical report number. The staff's correction of this typographical error in the TS page is purely editorial, and does not change the staff's original proposed no significant hazards determination. Since these revisions were previously approved for the McGuire Nuclear Station, Units 1 and 2, their addition to Specification 6.9.1.9 is acceptable.

Topical Report DPC-NE-3002A, "FSAR Chapter 15 System Transient Analysis Methodology" - The licensee proposed to add "Through Rev. 2" to this reference. Revision 2 was approved by

letter dated April 26, 1996, H. N. Berkow, NRC, to M. S. Tuckman, formally Duke Power Company (DPC). The licensee's amendment request inadvertently omitted the "A" designation at the end of the topical report number. The staff's correction of this typographical error in the TS page is purely editorial, and does not change the staff's original proposed no significant hazards determination. Since these revisions were previously approved for the McGuire Nuclear Station, Units 1 and 2, their addition to Specification 6.9.1.9 is acceptable.

Topical Report DPC-NE-3000P-A, Rev. 1, "Thermal-Hydraulic Transient Analysis Methodology" - The proposed change is considered editorial. It clarifies the approval date as "December 27, 1995," from "December 1995." The licensee's submittal inadvertently omitted the "-A" designation at the end of the topical report number. The staff's correction of this typographical error in the TS page is purely editorial, and does not change the staff's original proposed no significant hazards determination. Since this revision was previously approved for the McGuire Nuclear Station, Units 1 and 2, its addition to Specification 6.9.1.9 is acceptable.

Topical Report DPC-NE-1004A, Rev. 1, "Nuclear Design Methodology Using CASMO-3/SIMULATE-3P" - The licensee proposed to replace the original with Revision 1, which was approved by letter dated April 26, 1996, R. E. Martin, NRC, to M. S. Tuckman, DPC. Since Revision 1 was previously approved for the McGuire Nuclear Station, Units 1 and 2, its replacement is acceptable.

Topical Report DPC-NE-2004P-A, Rev. 1, "Duke Power Company McGuire and Catawba Nuclear Stations Core Thermal-Hydraulic Methodology using VIPRE-01" - Revision 1 was approved by letter, P. S. Tam, NRC, to M. S. Tuckman, DPC, dated February 20, 1997 (filed under TACs M97139 and M97140). Since Revision 1 was previously approved for the McGuire Nuclear Station, Units 1 and 2, its replacement is acceptable.

Topical Report DPC-NE-2005P-A, Rev. 1, "Thermal Hydraulic Statistical Core Design Methodology" - Revision 1 was approved by letter, H. N. Berkow, NRC, to M. S. Tuckman, DPC, dated November 7, 1996. Since Revision 1 was previously approved for the McGuire Nuclear Station, Units 1 and 2, its replacement is acceptable. Also, the designation "NE-" was added to the topical report number, which is consistent with other DPC topical report numbering.

Topical Report DPC-NE-2008P-A, "Fuel Mechanical Reload Analysis Methodology Using TACO3," April 3, 1995 - Currently, Babcock and Wilcox (B&W) Topical Report BAW-10162P-A is referenced. By letter, H. N. Berkow, NRC, to M. S. Tuckman, DPC, dated April 3, 1995, the staff approved transfer of the fuel performance code TACO3 from B&W to DPC for reload licensing applications. The licensee's request for amendment inadvertently omitted the "P-A" designation at the end of the topical report number. The staff's correction of this typographical error in the amended TS page is purely editorial, and does not change the staff's original proposed no significant hazard determination. The proposed replacement of BAW-10162P-A with DPC-NE-2008P-A reflects the staff's previous approval, and is acceptable.

Topical Report BAW-10183P-A, "Fuel Rod Gas Pressure Criterion," B&W Fuel Company, July 1995 - The licensee proposed to reference the approved version, dated July 1995. The licensee's request for amendment inadvertently omitted the "A-" designation at the end of the

topical report number. The staff's correction of this typographical error in the amended TS page is purely editorial, and does not change the staff's original proposed no significant hazards determination. This change, as corrected, reflects the approved status of the topical report and is acceptable.

The use of NRC-approved methodologies will ensure that values for cycle-specific parameters are determined consistent with applicable levels of the plant safety analysis. Therefore, the proposed changes are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: Frank Rinaldi

Date: April 8, 1998