

Docket Nos.: 50-327
and 50-328

October 2, 1986

Mr. S. A. White
Manager of Nuclear Power
Tennessee Valley Authority
6N 38A Lookout Place
1101 Market Street
Chattanooga, Tennessee 37402-2801

Dear Mr. White:

Subject: Issuance of Amendment No. 48 to Facility Operating License
No. DPR-77 and Amendment No. 40 to Facility Operating
License No. DPR-79 - Sequoyah Nuclear Plant, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 48 to Facility Operating License No. DPR-77 and Amendment No. 40 to Facility Operating License No. DPR-79. These amendments are in response to your request dated May 25, 1984, and supplemented July 11, 1986.

The amendments change the Technical Specifications (TS) to delete the requirement to perform the airlock door seal leakage test by the pressure decay method for 15 minutes and add a requirement that the seal leakage be determined by precision flow methods for at least two minutes. The amendments are effective as of their date of issuance. This letter should not be construed as an authorization to commence operations prior to the Tennessee Valley Authority appropriately addressing the concerns identified in the 50.54(f) letter dated September 17, 1985.

A copy of the related safety evaluation supporting Amendment No. 48 to Facility Operating License DPR-77 and Amendment No. 40 to Facility Operating License DPR-79 is enclosed.

Notice of issuance will be included in the Commission's next bi-weekly Federal Register notice.

Sincerely,

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PDR ADDCK 05000327
P PDR

Carl R. Stahle, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Enclosures:

1. Amendment No. 48 to DPR-77
2. Amendment No. 40 to DPR-79
3. Safety Evaluation

cc w/enclosures: See next page

PWR#4/DPWR-A
MDuncan/mac
09/ /86

PWR#4/DPWR-A
JHolonich
09/19/86

PWR#4/DPWR-A
CStahle
09/15/86

PWR#4/DPWR-A
BJYoungblood
09/17/86

Mr. S. A. White
Tennessee Valley Authority

Sequoyah Nuclear Plant

cc:

Tennessee Department of Public
Health
ATTN: Director, Bureau of
Environmental Health Services
Cordell Hull Building
Nashville, Tennessee 37219

Regional Administrator, Region II
U.S. Nuclear Regulatory Commission,
101 Marietta Street, N.W., Suite 2900
Atlanta, Georgia 30323

W. C. Drotleff
ATTN: J. A. Raulston
Tennessee Valley Authority
400 West Summit Hill Drive, W12 A12
Knoxville, Tennessee 37902

Mr. Michael H. Mobley, Director
Division of Radiological Health
T.E.R.R.A. Building
150 9th Avenue North
Nashville, Tennessee 37203

Mr. Bob Faas
Westinghouse Electric Corp.
P.O. Box 355
Pittsburgh, Pennsylvania 15230

County Judge
Hamilton County Courthouse
Chattanooga, Tennessee 37402

R. L. Gridley
Tennessee Valley Authority
5N 157B Lookout Place
Chattanooga, Tennessee 37402-2801

M. R. Harding
Tennessee Valley Authority
Sequoyah Nuclear Plant
P.O. Box 2000
Soddy Daisy, Tennessee 37379

Resident Inspector/Sequoyah NPS
c/o U.S. Nuclear Regulatory Commission
2600 Igou Ferry Road
Soddy Daisy, Tennessee 37379

October 2, 1986

AMENDMENT NO.48 TO FACILITY OPERATING LICENSE NO. DPR-77 - Sequoyah Nuclear Plant
Unit 1
AMENDMENT NO. 40 TO FACILITY OPERATING LICENSE NO. DPR-79 - Sequoyah Nuclear Plant
Unit 2

DISTRIBUTION w/ enclosures:

Docket No. 50-327/328
NRC PDR
Local PDR
NSIC
PRC System
PD#4 Reading File
C. Stahle
M. Duncan
OGC-Beth MNBB-9604
R. Diggs, ADM
T. Barnhart (8)
E.L. Jordan, DEQA:I&E
L.J. Harmon, I&E
J. Youngblood
H. Thompson, Jr.
J. Holonich
N. Thompson
E. Butcher
W. Jones
H. Denton
J. Taylor
G. Zech
B. Hayes
S. Weise. RII
L. Spessard
H. L. Whitener, RII
R. Perch



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 48
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated May 25, 1984, and supplemented July 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

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(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 48 are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

151

B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment"
Appendix A Technical
Specification Changes

Date of Issuance: October 2, 1986

PWR#4/DPWR-A
MDuncan/mac
09/10/86

PWR#4/DPWR-A
JHJonich
09/10/86

PWR#4/DPWR-A
CStahle
09/10/86

OGC/BETH
SST
09/15/86

PWR#4/DPWR-A
BJYoungblood
09/11/86

9/26/86

ATTACHMENT TO LICENSE AMENDMENT NO. 48

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Amended
Page

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CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying seal leakage less than or equal to $0.01 L_a$ as determined by precision flow measurements when measured for at least two minutes with the volume between the door seals at a pressure greater than or equal to 6 psig,
- b. By conducting an overall air lock leakage test at not less than P_a (12 psig) and by verifying the overall air lock leakage rate is within its limit:#
 1. At least once per six months, and
 2. Prior to establishing CONTAINMENT INTEGRITY if opened when CONTAINMENT INTEGRITY was not required when maintenance has been performed on the air lock that could affect the air lock sealing capability.*
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

#The provisions of Specification 4.0.2 are not applicable.

*Exemption to Appendix "J" of 10 CFR 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 40
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 2 (the facility) Facility Operating License No. DPR-79 filed by the Tennessee Valley Authority (licensee), dated May 25, 1984, and supplemented July 11, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 40 are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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B. J. Youngblood, Director
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment
Appendix A Technical
Specification Changes

Date of Issuance: October 2, 1986

PWR#4/DPWR-A
MDuncan/mac
09/10/86

~~PWR#4/DPWR-A~~
JHeinrich
09/10/86

PWR#4/DPWR-A
CStahle
09/10/86

OGC/BETH
SEB
09/10/86

~~PWR#4/DPWR-A~~
BJYoungblood
09/29/86

W. King
9/26/86

ATTACHMENT TO LICENSE AMENDMENT NO. 40

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

Amended
Page

3/4 6-8

CONTAINMENT SYSTEMS

SURVEILLANCE REQUIREMENTS

4.6.1.3 Each containment air lock shall be demonstrated OPERABLE:

- a. After each opening, except when the air lock is being used for multiple entries, then at least once per 72 hours, by verifying seal leakage less than or equal to $0.01 L_a$ as determined by precision flow measurements when measured for at least two minutes with the volume between the door seals at a pressure greater than or equal to 6 psig,
- b. By conducting an overall air lock leakage test at not less than P_a (12 psig) and by verifying the overall air lock leakage rate is within its limit:
 1. At least once per six months, and
 2. Prior to establishing CONTAINMENT INTEGRITY if opened when CONTAINMENT INTEGRITY was not required when maintenance has been performed on the air lock that could affect the air lock sealing capability.*
- c. At least once per 6 months by verifying that only one door in each air lock can be opened at a time.

#The provisions of Specification 4.0.2 are not applicable.

*Exemption to Appendix "J" of 10 CFR 50.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO.48 TO FACILITY OPERATING LICENSE DPR-77
AND AMENDMENT NO.40 TO FACILITY OPERATING LICENSE DPR-79
TENNESSEE VALLEY AUTHORITY

INTRODUCTION

By letters dated May 25, 1984 and July 11, 1986, the Tennessee Valley Authority (TVA) requested amendments to the Technical Specifications (TS) for the Sequoyah Nuclear Plant, Units 1 and 2.

EVALUATION

The proposed change would delete the requirement in TS 4.6.1.3.a to perform the airlock door seal leakage test by the pressure decay method for 15 minutes and add a requirement that the seal leakage be determined by precision flow methods for at least two minutes. The licensee submitted test data to show that the pressure decay method yields leakage results in the order of 10 to 50 times less conservative than the make-up flow method. The staff agrees that due to the small test volume involved (about 0.067 SCF) the pressure drop in the volume can occur too fast for accurate measurement. Further, the proposed TS change is consistent with the revised NRC position on testing airlock door seals in accordance with NUREG-0452, Revision 5 (Draft), and with the requirements of Appendix J to 10 CFR 50.

Based on the above information, the staff finds the proposed amendments to Sequoyah Units 1 and 2 TS acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the use of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding.

Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

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CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register on September 28, 1984 (49 FR 38410) and consulted with the state of Tennessee. No public comments were received, and the state of Tennessee did not have any comments.

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: Carl R. Stahle, PWR#4, DPWR-A
Joe Holonich, PWR#4, DPWR-A
H. Whitener, Region 2

Dated: October 2, 1986