



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

September 3, 1985

Docket Nos: 50-327  
and 50-328

Mr. H. G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 41 to Facility Operating License  
No. DPR-77 and Amendment No. 33 to Facility Operating  
License No. DPR-79 - Sequoyah Nuclear Plant, Units 1 and 2

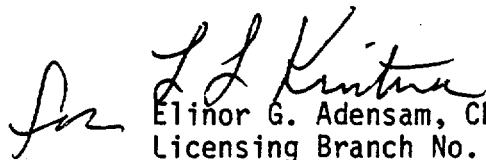
The Nuclear Regulatory Commission has issued the enclosed Amendment No. 41 to Facility Operating License No. DPR-77 and Amendment No. 33 to Facility Operating License No. DPR-79. These amendments are in response to your request dated May 21, 1985.

The amendments change the Technical Specifications to delete references to three loop power operation. The amendments are effective as of their date of issuance.

A copy of the related safety evaluation supporting Amendment No. 41 to Facility Operating License DPR-77 and Amendment No. 33 to Facility Operating License DPR-79 is enclosed.

Notice of issuance will be included in the Commission's next monthly Federal Register notice.

Sincerely,

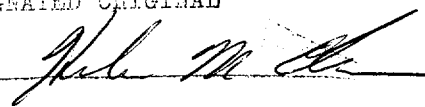
  
Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Enclosures:

1. Amendment No. 41 to DPR-77
2. Amendment No. 33 to DPR-79
3. Safety Evaluation

cc w/enclosures:  
See next page

DESIGNATED ORIGINAL

Certified By 

8509130410 850903  
PDR ADDCK 05000327  
P PDR

September 3, 1985

AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE NO. DPR-77 - Sequoyah Nuclear Plant  
AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE NO. DPR-79 - Sequoyah Nuclear Plant

DISTRIBUTION w/enclosures:

✓ Docket No. 50-327/328

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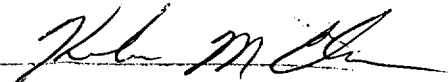
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SEQUOYAH

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 41  
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated May 21, 1985 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

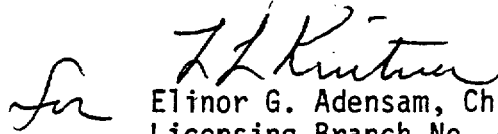
The Technical Specifications contained in Appendix A, as revised through Amendment No. 41, are hereby incorporated into the license.

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PDR ADOCK 05000327  
P PDR

The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Attachment:  
Appendix A Technical  
Specification Changes

Date of Issuance: September 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 41

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended  
Page

2-1  
2-3  
3/4 1-21  
3/4 1-23  
3/4 2-19  
3/4 3-2  
3/4 3-15  
3/4 3-16  
3/4 3-17  
3/4 3-18  
3/4 3-19  
3/4 3-20  
3/4 3-21

## 2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

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### 2.1 SAFETY LIMITS

#### REACTOR CORE

2.1.1 The combination of THERMAL POWER, pressurizer pressure, and the highest operating loop coolant temperature ( $T_{avg}$ ) shall not exceed the limits shown in Figure 2.1-1.

APPLICABILITY: MODES 1 and 2.

#### ACTION:

Whenever the point defined by the combination of the highest operating loop average temperature and THERMAL POWER has exceeded the appropriate pressurizer pressure line, be in HOT STANDBY within 1 hour.

#### REACTOR COOLANT SYSTEM PRESSURE

2.1.2 The Reactor Coolant System pressure shall not exceed 2735 psig.

APPLICABILITY: MODES 1, 2, 3, 4 and 5.

#### ACTION:

MODES 1 and 2

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, be in HOT STANDBY with the Reactor Coolant System pressure within its limit within 1 hour.

MODES 3, 4 and 5

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, reduce the Reactor Coolant System pressure to within its limit within 5 minutes.

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## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD INSERTION LIMITS

#### LIMITING CONDITION FOR OPERATION

---

3.1.3.6 The control banks shall be limited in physical insertion as shown in Figure 3.1-1.

APPLICABILITY: MODES 1\* and 2\*#.

ACTION:

With the control banks inserted beyond the above insertion limits, except for surveillance testing pursuant to Specification 4.1.3.1.2, either:

- a. Restore the control banks to within the limits within two hours, or
- b. Reduce THERMAL POWER within two hours to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the group position using the above figures, or
- c. Be in HOT STANDBY within 6 hours.

#### SURVEILLANCE REQUIREMENTS

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4.1.3.6 The position of each control bank shall be determined to be within the insertion limits at least once per 12 hours except during time intervals when the Rod Insertion Limit Monitor is inoperable, then verify the individual rod positions at least once per 4 hours.

\*See Special Test Exceptions 3.10.2 and 3.10.3.

#With  $K_{eff}$  greater than or equal to 1.0.

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TABLE 3.2-1DNB PARAMETERSLIMITS

<u>PARAMETER</u>	<u>4 Loops In Operation</u>
Reactor Coolant System $T_{avg}$	$\leq 583^{\circ}\text{F}$
Pressurizer Pressure	$\geq 2220 \text{ psia}^*$

\* Limit not applicable during either a THERMAL POWER ramp in excess of 5% RATED THERMAL POWER per minute or a THERMAL POWER step in excess of 10% RATED THERMAL POWER, physics test, or performance of surveillance requirement 4.1.1.3.b.

TABLE 3.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. Manual Reactor Trip	2	1	2	1, 2, and *	1
2. Power Range, Neutron Flux	4	2	3	1, 2	2 <sup>#</sup>
3. Power Range, Neutron Flux High Positive Rate	4	2	3	1, 2	2 <sup>#</sup>
4. Power Range, Neutron Flux, High Negative Rate	4	2	3	1, 2	2 <sup>#</sup>
5. Intermediate Range, Neutron Flux	2	1	2	1, 2, and *	3
6. Source Range, Neutron Flux					
A. Startup	2	1	2	2 <sup>##</sup> , and *	4
B. Shutdown	2	0	1	3, 4 and 5	5
7. Overtemperature Delta T Four Loop Operation	4	2	3	1, 2	6 <sup>#</sup>
8. Overpower Delta T Four Loop Operation	4	2	3	1, 2	6 <sup>#</sup>
9. Pressurizer Pressure--Low	4	2	3	1, 2	6 <sup>#</sup>
10. Pressurizer Pressure--High	4	2	3	1, 2	6 <sup>#</sup>
11. Pressurizer Water Level--High	3	2	2	1, 2	7 <sup>#</sup>

TABLE 3.3-3  
ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. SAFETY INJECTION, TURBINE TRIP AND FEEDWATER ISOLATION					
a. Manual Initiation	2	1	2	1, 2, 3, 4	20
b. Automatic Actuation Logic	2	1	2	1, 2, 3, 4	15
c. Containment Pressure-High	3	2	2	1, 2, 3	16*
d. Pressurizer Pressure - Low	3	2	2	1, 2, 3#	16*
e. Differential Pressure Between Steam Lines - High				1, 2, 3	
Four Loops Operating	3/steam line	2/steam line any steam line	2/steam line		16*
f. Steam Flow in Two Steam Lines-High				1, 2, 3 <sup>##</sup>	
Four Loops Operating	2/steam line	1/steam line any 2 steam lines	1/steam line		16*

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
COINCIDENT WITH EITHER T <sub>avg</sub> --Low-Low				1, 2, 3 <sup>##</sup>	
Four Loops Operating	1 T <sub>avg</sub> /loop	2 T <sub>avg</sub> any loops	1 T <sub>avg</sub> any 3 loops		16*
OR, COINCIDENT WITH Steam Line Pressure-Low				1, 2, 3 <sup>##</sup>	
Four Loops Operating	1 pressure/ loop	2 pressures any loops	1 pressure any 3 loops		16*
2. CONTAINMENT SPRAY					
a. Manual	2	1	2	1, 2, 3, 4	20
b. Automatic Actuation Logic	2	1	2	1, 2, 3, 4	15
c. Containment Pressure-- High-High	4	2	3	1, 2, 3	18
3. CONTAINMENT ISOLATION					
a. Phase "A" Isolation					
1) Manual	2	1	2	1, 2, 3, 4	20
2) From Safety Injection Automatic Actuation Logic	2	1	2	1, 2, 3, 4	15

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
b. Phase "B" Isolation					
1) Manual	2	1	2	1, 2, 3, 4	20
2) Automatic Actuation Logic	2	1	2	1, 2, 3, 4	15
3) Containment Pressure-High-High	4	2	3	1, 2, 3	18
c. Containment Ventilation Isolation					
1) Manual	2	1	2	1, 2, 3, 4	19
2) Automatic Isolation Logic	2	1	2	1, 2, 3, 4	15
3) Containment Gas Monitor Radioactivity-High	2	1	1	1, 2, 3, 4	19
4) Containment Purge Air Exhaust Monitor Radioactivity-High	2	1	1	1, 2, 3, 4	19
5) Containment Particu- late Activity High	2	1	1	1, 2, 3, 4	19

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
4. STEAM LINE ISOLATION					
a. Manual	1/steam line	1/steam line	1/operating steam line	1, 2, 3	25
b. Automatic Actuation Logic	2	1	2	1, 2, 3	23
c. Containment Pressure-- High-High	4	2	3	1, 2, 3	18
d. Steam Flow in Two Steam Lines--High				1, 2, 3	
Four Loops Operating	2/steam line	1/steam line any 2 steam lines	1/steam line		16*
COINCIDENT WITH EITHER T <sub>avg</sub> --Low-Low Four Loops Operating	1 T <sub>avg</sub> /loop	2 T <sub>avg</sub> any loop	1 T <sub>avg</sub> any 3 loops	1, 2, 3	16*
OR, COINCIDENT WITH Steam Line Pressure- Low				1, 2, 3	
Four Loops Operating	1 pressure/ loop	2 pressures any loops	1 pressure any 3 loops		16*



TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
5. TURBINE TRIP & FEEDWATER ISOLATION					
a. Steam Generator Water Level-- High-High	3/loop	2/loop in any oper- ating loop	2/loop in each oper- ating loop	1, 2, 3	16*
6. AUXILIARY FEEDWATER					
a. Manual Initiation	2	1	2	1, 2, 3	24
b. Automatic Actuation Logic	2	1	2	1, 2, 3	23
c. Main Stm. Gen. Water Level-Low-Low					
i. Start Motor Driven Pumps	3/stm. gen.	2/stm. gen. any stm gen.	2/stm. gen.	1, 2, 3	16*
ii. Start Turbine- Driven Pump	3/stm. gen.	2/stm. gen. any 2 stm. gen.	2/stm. gen.	1, 2, 3	16*
d. S.I. Start Motor-Driven Pumps and Turbine Driven Pump	See 1 above (all S.I. initiating functions and requirements)				

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
e. Station Blackout Start Motor-Driven Pump associated with the shutdown board and Turbine Driven Pump	2/shutdown board	1/shutdown board	2/shutdown board	1, 2, 3	20
f. Trip of Main Feedwater Pumps Start Motor-Driven Pumps and Turbine Driven Pump	1/pump	1/pump	1/pump	1, 2	20*
g. Auxiliary Feedwater Suction Pressure-Low	3/pump	2/pump	2/pump	1, 2, 3	20*
**7. LOSS OF POWER					
a. 6.9 kv Shutdown Board --Loss of Voltage					
1. Start Diesel Generators	2/shutdown board	1 loss of voltage on any shutdown board	2/shutdown board	1, 2, 3, 4	20*
2. Load Shedding	2/shutdown board	1/shutdown board	2/shutdown board	1, 2, 3, 4	20*

\*\*NOTE: This technical specification is to be implemented during the startup following the 2nd refueling outage or following completion of the modification, whichever is earlier.

TABLE 3.3-3 (Continued)ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>		<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
b.	6.9 kv Shutdown Board Degraded Voltage					
1.	Voltage Sensors	3/shutdown board	2/shutdown board	2/shutdown board	1, 2, 3, 4	20*
2.	Diesel Generator Start and Load Shedding Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	20*
3.	SI/Degraded Voltage Enable Timer	2/shutdown board	1/shutdown board	1/shutdown board	1, 2, 3, 4	20*



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 33  
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 2 (the facility) Facility Operating License No. DPR-79 filed by the Tennessee Valley Authority (licensee), dated May 21, 1985, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

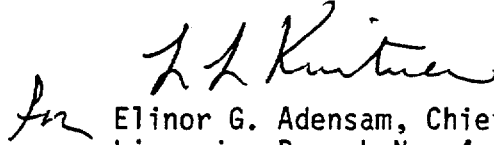
(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 33, are hereby incorporated into the license.

The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

  
Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Attachment:  
Appendix A Technical  
Specification Changes

Date of Issuance: September 3, 1985

ATTACHMENT TO LICENSE AMENDMENT NO. 33

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended  
Page

2-1  
2-3  
3/4 1-21  
3/4 1-23  
3/4 2-17  
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3/4 3-15  
3/4 3-16  
3/4 3-18  
3/4 3-19

## 2.0 SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS

### 2.1 SAFETY LIMITS

#### REACTOR CORE

2.1.1 The combination of THERMAL POWER, pressurizer pressure, and the highest operating loop coolant temperature ( $T_{avg}$ ) shall not exceed the limits shown in Figure 2.1-1.

APPLICABILITY: MODES 1 and 2.

#### ACTION:

Whenever the point defined by the combination of the highest operating loop average temperature and THERMAL POWER has exceeded the appropriate pressurizer pressure line, be in HOT STANDBY within 1 hour.

#### REACTOR COOLANT SYSTEM PRESSURE

2.1.2 The Reactor Coolant System pressure shall not exceed 2735 psig.

APPLICABILITY: MODES 1, 2, 3, 4 and 5.

#### ACTION:

MODES 1 and 2

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, be in HOT STANDBY with the Reactor Coolant System pressure within its limit within 1 hour.

MODES 3, 4 and 5

Whenever the Reactor Coolant System pressure has exceeded 2735 psig, reduce the Reactor Coolant System pressure to within its limit within 5 minutes.

This page deleted.



## REACTIVITY CONTROL SYSTEMS

### CONTROL ROD INSERTION LIMITS

#### LIMITING CONDITION FOR OPERATION

---

3.1.3.6 The control banks shall be limited in physical insertion as shown in Figure 3.1-1.

APPLICABILITY: Modes 1\* and 2\*#.

ACTION:

With the control banks inserted beyond the above insertion limits, except for surveillance testing pursuant to Specification 4.1.3.1.2, either:

- a. Restore the control banks to within the limits within two hours, or
- b. Reduce THERMAL POWER within two hours to less than or equal to that fraction of RATED THERMAL POWER which is allowed by the group position using the above figures, or
- c. Be in at least HOT STANDBY within 6 hours.

#### SURVEILLANCE REQUIREMENTS

---

4.1.3.6 The position of each control bank shall be determined to be within the insertion limits at least once per 12 hours except during time intervals when the Rod Insertion Limit Monitor is inoperable, then verify the individual rod positions at least once per 4 hours.

\*See Special Test Exceptions 3.10.2 and 3.10.3.

#With  $K_{eff}$  greater than or equal to 1.0.

This page deleted.

TABLE 3.2-1  
DNB PARAMETERS

<u>PARAMETER</u>	<u>LIMITS</u>
	4 Loops In <u>Operation</u>
Reactor Coolant System T <sub>avg</sub>	≤ 583°F
Pressurizer Pressure	≥ 2220 psia*

\*Limit not applicable during either a THERMAL POWER ramp in excess of 5% of RATED THERMAL POWER per minute or a THERMAL POWER step in excess of 10% of RATED THERMAL POWER, physics test, or performance of surveillance requirement 4.1.1.3.b.

TABLE 3.3-1

REACTOR TRIP SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. Manual Reactor Trip	2	1	2	1, 2, and *	1
2. Power Range, Neutron Flux	4	2	3	1, 2	2 <sup>#</sup>
3. Power Range, Neutron Flux High Positive Rate	4	2	3	1, 2	2 <sup>#</sup>
4. Power Range, Neutron Flux, High Negative Rate	4	2	3	1, 2	2 <sup>#</sup>
5. Intermediate Range, Neutron Flux	2	1	2	1, 2, and *	3
6. Source Range, Neutron Flux					
A. Startup	2	1	2	2 <sup>##</sup> , and *	4
B. Shutdown	2	0	1	3, 4 and 5	5
7. Overtemperature $\Delta T$ Four Loop Operation	4	2	3	1, 2	6 <sup>#</sup>
8. Overpower $\Delta T$ Four Loop Operation	4	2	3	1, 2	6 <sup>#</sup>
9. Pressurizer Pressure-Low	4	2	3	1, 2	6 <sup>#</sup>
10. Pressurizer Pressure--High	4	2	3	1, 2	6 <sup>#</sup>
11. Pressurizer Water Level--High	3	2	2	1, 2	7 <sup>#</sup>

TABLE 3.3-3

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
1. SAFETY INJECTION, TURBINE TRIP AND FEEDWATER ISOLATION					
a. Manual Initiation	2	1	2	1, 2, 3, 4	20
b. Automatic Actuation Logic	2	1	2	1, 2, 3, 4	15
c. Containment Pressure-High	3	2	2	1, 2, 3	16*
d. Pressurizer Pressure - Low	3	2	2	1, 2, 3#	16*
e. Differential Pressure Between Steam Lines - High				1, 2, 3	
Four Loops Operating	3/steam line	2/steam line any steam line	2/steam line		16*

TABLE 3.3-3 (Continued)  
ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
f. Steam Flow in Two Steam Lines-High				1, 2, 3 <sup>##</sup>	
Four Loops Operating	2/steam line	1/steam line any 2 steam lines	1/steam line		16*
COINCIDENT WITH EITHER T <sub>avg</sub> --Low-Low				1, 2, 3 <sup>##</sup>	
Four Loops Operating	1 T <sub>avg</sub> /loop	2 T <sub>avg</sub> any loops	1 T <sub>avg</sub> any 3 loops		16*
OR, COINCIDENT WITH Steam Line Pressure-Low				1, 2, 3 <sup>##</sup>	
Four Loops Operating	1 pressure/ loop	2 pressures any loops	1 pressure any 3 loops		16*

TABLE 3.3-3 (Continued)  
ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
c. Containment Ventilation Isolation					
1) Manual	2	1	2	1, 2, 3, 4	19
2) Automatic Isolation Logic	2	1	2	1, 2, 3, 4	15
3) Containment Gas Monitor Radioactivity-High	2	1	1	1, 2, 3, 4	19
4) Containment Purge Air Exhaust Monitor Radioactivity-High	2	1	1	1, 2, 3, 4	19
5) Containment Particulate Activity High	2	1	1	1, 2, 3, 4	19
4. STEAM LINE ISOLATION					
a. Manual	1/steam line	1/steam line	1/operating steam line	1, 2, 3	25
b. Automatic Actuation Logic	2	1	2	1, 2, 3	23
c. Containment Pressure--High-High	4	2	3	1, 2, 3	18
d. Steam Flow in Two Steam Lines--High				1, 2, 3	
Four Loops Operating	2/steam line	1/steam line any 2 steam lines	1/steam line		16*

TABLE 3.3-3 (Continued)

ENGINEERED SAFETY FEATURE ACTUATION SYSTEM INSTRUMENTATION

<u>FUNCTIONAL UNIT</u>	<u>TOTAL NO. OF CHANNELS</u>	<u>CHANNELS TO TRIP</u>	<u>MINIMUM CHANNELS OPERABLE</u>	<u>APPLICABLE MODES</u>	<u>ACTION</u>
COINCIDENT WITH EITHER T <sub>avg</sub> --Low-Low Four Loops Operating	1 T <sub>avg</sub> /loop	2 T <sub>avg</sub> any loop	1 T <sub>avg</sub> any 3 loops	1, 2, 3	16*
OR, COINCIDENT WITH Steam Line Pressure- Low				1, 2, 3	
Four Loops Operating	1 pressure/ loop	2 pressures any loops	1 pressure any 3 loops		16*
5. TURBINE TRIP & FEEDWATER ISOLATION					
a. Steam Generator Water Level-- High-High	3/loop	2/loop in any oper- ating loop	2/loop in each oper- ating loop	1, 2, 3	16*





UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 41 TO FACILITY OPERATING LICENSE DPR-77  
AND AMENDMENT NO. 33 TO FACILITY OPERATING LICENSE DPR-79  
TENNESSEE VALLEY AUTHORITY

INTRODUCTION

The Tennessee Valley Authority's (TVA's) submittal dated May 21, 1985, proposes to delete all references to three-loop power operations in the Technical Specifications of Sequoyah Nuclear Plant Units 1 and 2.

EVALUATION

When the Technical Specifications were originally issued to TVA, NRC left in place all references to three-loop operation. TVA stated that it has no plans to pursue approval of three-loop operation at Sequoyah. Additionally, this change was requested to eliminate the possibility of these Technical Specification requirements being inappropriately applied, resulting in confusion or potential violation. This change is administrative in nature and removes unused and unnecessary information from the Technical Specifications and is, therefore, acceptable.

ENVIRONMENTAL CONSIDERATION

These amendments involve changes in the installation of facility components located within the restricted area as defined in 10 CFR Part 20. The staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration, and there has been no public comment on such finding. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR Sec 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

CONCLUSION

The Commission made a proposed determination that the amendments involve no significant hazards consideration which was published in the Federal Register on July 31, 1985 (50 FR 31072) and consulted with the state of Tennessee. No public comments were received, and the state of Tennessee did not have any comments.

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We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Dated: September 3, 1985