

December 23, 1982

Docket Nos: 50-327
and 50-328

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 20 to Facility Operating License
No. DPR-77 and Amendment No. 10 to Facility Operating
License No. DPR-79 - Sequoyah Nuclear Plant, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 20 to
Facility Operating License No. DPR-77 and Amendment No. 10 to Facility Operating
License No. DPR-79.

The amendments change the maximum isolation time for containment isolation valves,
modify the surveillance requirements for testing of containment penetration pro-
tective fuses, and correct a typographical error in Table 4.4-5. The other changes
requested in your August 16, 1982, letter will be addressed in a future amendment.

A copy of the related safety evaluation supporting Amendment No. 20 to Facility
Operating License DPR-77 and Amendment No. 10 to Facility Operating License DPR-79
is enclosed. Also enclosed is a copy of the Federal Register Notice which has
been forwarded to the Office of the Federal Register for publication.

Sincerely,

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Enclosures:

1. Amendment No. 20 to DPR-77
2. Amendment No. 10 to DPR-79
3. Safety Evaluation
4. Federal Register Notice

cc w/enclosures:
See next page

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| SURNAME | MDuncan/hmc | MMiller | CStable | EAdensam | H. Park | | |
| DATE | 9/16/82 | 9/16/82 | 9/14/82 | 12/23/82 | 9/21/82 | | |

SEQUOYAH

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

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Tennessee Valley Authority
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Resident Inspector/Sequoyah NPS
c/o U.S. Nuclear Regulatory
Commission
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Director, Office of Urban
& Federal Affairs
108 Parkway Towers
404 James Robertson Way
Nashville, Tennessee 37219

Attorney General
Supreme Court Building
Nashville, Tennessee 37219

U.S. Environmental Protection
Agency
ATTN: EIS Coordinator
345 Courtland Street
Atlanta, Georgia 30308

Honorable Don Moore, Jr.
County Judge
Hamilton County Courthouse
Chattanooga, Tennessee 37402

Regional Administrator
Nuclear Regulatory Commission,
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

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TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.20
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated August 16, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 20, are hereby incorporated into the license.

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The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

51

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: December 23, 1982

As for TN 12/23/82
WBS 9/27
WBS 12/21/82

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| OFFICE | LA:DL:LB #4 | DL:LB #4 | OELD | DL:LB #4 | DL:LB #4 | CSB | PSB |
| SURNAME | <i>md</i> MDuncan/hmc | <i>MM</i> MMiller | <i>R. K...</i> | <i>C</i> CStahle | <i>E</i> EAdensam | <i>W</i> WButler | <i>M</i> MSrinivasan |
| DATE | 9/16/82 | 9/16/82 | 9/21/82 | 9/14/82 | 9/23/82 | 9/27/82 | 12/21/82 |

ATTACHMENT TO LICENSE AMENDMENT NO.20

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Amended
Page

3/4 6-20
3/4 8-16
3/4 4-23a

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TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (Seconds)</u> | |
|--------------------------------|-----------------|---|----|
| A. PHASE "A" ISOLATION (Cont.) | | | |
| 32. | FCV-62-63 | RCP Seals | 10 |
| 33. | FCV-62-72 | Letdown Line | 10 |
| 34. | FCV-62-73 | Letdown Line | 10 |
| 35. | FCV-62-74 | Letdown Line | 10 |
| 36. | FCV-62-77 | Letdown Line | 20 |
| 37. | FCV-63-23 | Accum to Hold Up Tank | 10 |
| 38. | FCV-63-64 | WDS N ₂ to Accum | 10 |
| 39. | FCV-63-71 | Accum ² to Hold Up Tank | 10 |
| 40. | FCV-63-84 | Accum to Hold Up Tank | 10 |
| 41. | FCV-68-305 | WDS N ₂ to PRT | 10 |
| 42. | FCV-68-307 | PRT to Gas Analyzer | 10 |
| 43. | FCV-68-308 | PRT to Gas Analyzer | 10 |
| 44. | FCV-70-85 | CCS from Excess Lt Dn Hx | 10 |
| 45. | FCV-70-143 | CCS to Excess Lt Dn Hx | 60 |
| 46. | FCV-77-9 | RCDT Pump Disch | 10 |
| 47. | FCV-77-10 | RCDT Pump Disch | 10 |
| 48. | FCV-77-16 | RCDT to Gas Analyzer | 10 |
| 49. | FCV-77-17 | RCDT to Gas Analyzer | 10 |
| 50. | FCV-77-18 | RCDT and PRT to V H | 10 |
| 51. | FCV-77-19 | RCDT and PRT to V H | 10 |
| 52. | FCV-77-127 | Floor Sump Pump Disch | 10 |
| 53. | FCV-77-128 | Floor Sump Pump Disch | 10 |
| 54. | FCV-81-12 | Primary Water Makeup | 10 |
| 55. | FCV-87-7 | UHI Test Line | 10 |
| 56. | FCV-87-8 | UHI Test Line | 10 |
| 57. | FCV-87-9 | UHI Test Line | 10 |
| 58. | FCV-87-10 | UHI Test Line | 10 |
| 59. | FCV-87-11 | UHI Test Line | 10 |
| 60. | FCV-26-240 | Fire Protection Isol. | 20 |
| 61. | FCV-26-243 | Fire Protection Isol. | 20 |

TABLE 4.4-5
REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM - WITHDRAWL SCHEDULE

| <u>CAPSULE NUMBER</u> | <u>VESSEL LOCATION</u> | <u>LEAD FACTOR</u> | <u>WITHDRAWAL TIME (EFPY)</u> |
|-----------------------|------------------------|--------------------|-------------------------------|
| T | 40° | 3.73 | 1 st REFUELING |
| U | 140° | 3.73 | 3 |
| X | 220° | 3.73 | 5 |
| Y | 320° | 3.73 | 9 |
| S | 4° | 1.09 | EOL |
| V | 176° | 1.09 | STBY |
| W | 184° | 1.09 | STBY |
| Z | 356° | 1.09 | STBY |

ELECTRICAL POWER SYSTEMS

SURVEILLANCE REQUIREMENTS (Continued)

- (c) For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 1 of the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.
2. By selecting and functionally testing a representative sample of at least 10% of each type of lower voltage circuit breakers. Circuit breakers selected for functional testing shall be selected on a rotating basis. The functional test shall consist of injecting a current input at the specified setpoint to each selected circuit breaker and verifying that each circuit breaker functions as designed. Circuit breakers found inoperable during functional testing shall be restored to OPERABLE status prior to resuming operation. For each circuit breaker found inoperable during these functional tests, an additional representative sample of at least 10% of all the circuit breakers of the inoperable type shall also be functionally tested until no more failures are found or all circuit breakers of that type have been functionally tested.
3. By selecting and functionally testing a representative sample of each type of fuse on a rotating basis. Each representative sample of fuses shall include at least 10% of all fuses of that type. The functional test shall consist of a non-destructive resistance measurement test which demonstrates that the fuse meets its manufacturer's design criteria. Fuses found inoperable during these functional tests shall be replaced with OPERABLE fuses prior to resuming operation. For each fuse found inoperable during these functional tests, an additional representative sample of at least 10% of all fuses of that type shall be functionally tested until no more failures are found or all fuses of that type have been functionally tested.*
- b. At least once per 60 months by subjecting each circuit breaker to an inspection and preventive maintenance in accordance with procedures prepared in conjunction with its manufacturer's recommendations.

*Surveillance requirement 4.8.3.1.a.3 may be suspended until the completion of Cycle 2 provided that the following surveillance requirement is implemented:

A fuse inspection and maintenance program will be maintained to ensure that:

1. The proper size and type of fuse is installed,
2. The fuse shows no sign of deterioration, and
3. The fuse connections are tight and clean.

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 2 (the facility) Facility Operating License No. DPR-79 filed by the Tennessee Valley Authority (licensee), dated August 16, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 10, are hereby incorporated into the license.

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The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

S/
Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: December 23, 1982

As of 12/23/82
CSB 9/27
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| OFFICE | LA:DL:LB #4 | DL:LB #4 | DL:LB #4 | OELD | DL:LB #4 | CSB | PSB |
| SURNAME | MDuncan/hmc | MMiller | CStahle | <i>R. [unclear]</i> | EAdensam | WButler | MSrinivasan |
| DATE | 9/16/82 | 9/16/82 | 9/16/82 | 9/21/82 | 9/27/82 | 9/27/82 | 11/ /82 |

ATTACHMENT TO LICENSE AMENDMENT NO. 10

FACILITY OPERATING LICENSE NO. DPR-79

DUCKET NO. 50-328

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Amended
Page

3/4 6-20

3/4 4-29

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TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

| <u>VALVE NUMBER</u> | <u>FUNCTION</u> | <u>MAXIMUM ISOLATION TIME (Seconds)</u> | |
|---------------------|-----------------------------|---|----|
| A. | PHASE "A" ISOLATION (Cont.) | | |
| 32. | FCV-62-63 | RCP Seals | 10 |
| 33. | FCV-62-72 | Letdown Line | 10 |
| 34. | FCV-62-73 | Letdown Line | 10 |
| 35. | FCV-62-74 | Letdown Line | 10 |
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| 37. | FCV-63-23 | Accum to Hold Up Tank | 10 |
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| 40. | FCV-63-84 | Accum to Hold Up Tank | 10 |
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| 42. | FCV-68-307 | PRT to Gas Analyzer | 10 |
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| 49. | FCV-77-17 | RCDT to Gas Analyzer | 10 |
| 50. | FCV-77-18 | RCDT and PRT to V H | 10 |
| 51. | FCV-77-19 | RCDT and PRT to V H | 10 |
| 52. | FCV-77-20 | N ₂ to RCDT | 10 |
| 53. | FCV-77-127 | Floor Sump Pump Disch | 10 |
| 54. | FCV-77-128 | Floor Sump Pump Disch | 10 |
| 55. | FCV-81-12 | Primary Water Makeup | 10 |
| 56. | FCV-87-7 | UHI Test Line | 10 |
| 57. | FCV-87-8 | UHI Test Line | 10 |
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| 59. | FCV-87-10 | UHI Test Line | 10 |
| 60. | FCV-87-11 | UHI Test Line | 10 |
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| 62. | FCV-26-243 | Fire Protection Isol. | 20 |

TABLE 4.4-5

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|----------------------------|----------------------------|------------------------|-------------------------------|
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| U | 140° | 3.73 | 3 |
| X | 220° | 3.73 | 5 |
| Y | 320° | 3.73 | 9 |
| S | 4° | 1.09 | EOL |
| V | 176° | 1.09 | STBY |
| W | 184° | 1.09 | STBY |
| Z | 356° | 1.09 | STBY |



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE DPR-77
AND AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE DPR-79
TENNESSEE VALLEY AUTHORITY

INTRODUCTION

On August 16, 1982, TVA requested a change to the Sequoyah Technical Specifications concerning maximum isolation time for the outboard containment isolation valve. The change in the maximum allowable stroke time would be increased from 10 seconds to 20 seconds to permit the inboard isolation valves to close before the outboard isolation valves.

By letter dated August 16, 1982, TVA also proposed a change to the Sequoyah Units 1 and 2 Technical Specifications regarding testing of containment protective fuses. TVA believes that the required testing of 10% of these fuses every 18 months is unnecessary. The proposed change suggests visually inspecting at least 10% of each fuse type.

EVALUATION

The maximum isolation times for containment isolation valves are found in Table 3.6-2 of Sequoyah Units 1 and 2 Technical Specifications. TVA has suggested increasing the maximum allowable stroke time of the outboard containment isolation valve (FCV-62-77) from 10 to 20 seconds. The increased stroke time permits the inboard isolation valves (FCV-62-72, 73 or 74) to close before the outboard isolation valve. This sequence prevents overpressurizing the letdown line between the containment isolation valves and lifting the relief valve 62-662. TVA's proposal is justified by a May 5, 1982, letter from Darrell G. Eisenhut to All Licensees of Operating Plants, "Engineering Evaluation of the H.B. Robinson Reactor Coolant System Leak on January 29, 1981," which supports the TVA closing sequence since overpressurizing the relief valve would be prevented. Additionally, the staff agrees that TVA's proposed order of containment isolation valve closing, i.e. inboard prior to outboard, may decrease relief valve bellow failure. Therefore, the maximum allowable stroke time of FCV-62-77 should be increased to 20 seconds.

At Sequoyah there are three types of protection fuses: 6900 and 480 volt fuses crimped inline and 480 volt fuses located in clip type holders. The licensee has indicated in regard to these fuses that: a) removal and insertion of fuses located in clip type holders may damage the holder's contact points; b) due to the seismic design of the holders, fuse removal will destroy the fuse itself; c) removal and replacement of inline fuses may compromise cable integrity, and d) removal of fuses that are crimped inline and wrapped with heat shrink insulation material will destroy the fuse itself.

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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE DPR-77
AND AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE DPR-79
TENNESSEE VALLEY AUTHORITY

INTRODUCTION

On August 16, 1982, TVA requested a change to the Sequoyah Technical Specifications concerning maximum isolation time for the outboard containment isolation valve. The change in the maximum allowable stroke time would be increased from 10 seconds to 20 seconds to permit the inboard isolation valves to close before the outboard isolation valves.

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EVALUATION

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At Sequoyah there are three types of protection fuses: 6900 and 480 volt fuses crimped inline and 480 volt fuses located in clip type holders. The licensee has indicated in regard to these fuses that: a) removal and insertion of fuses located in clip type holders may damage the holder's contact points; b) due to the seismic design of the holders, fuse removal will destroy the fuse itself; c) removal and replacement of inline fuses may compromise cable integrity, and d) removal of fuses that are crimped inline and wrapped with heat shrink insulation material will destroy the fuse itself.

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Due to inaccessibility, destructive removal is required for testing of fuses at the Sequoyah Plant; thus, the licensee has requested the subject surveillance requirement changed to require only visual inspection. In justification of the change, the licensee has quoted Gould Shawmut, a major fuse manufacturer, to the effect that "Under no condition can a current limiting fuse ever become less protective over life."

Based on the above and the proposed periodic visual inspection, the staff concludes that there is reasonable assurance that the fuses will perform their safety function over the next fuel cycle without performing the subject surveillance. Therefore, interim Technical Specification relief is being granted for Sequoyah Unit 1 until the completion of Cycle 2 as indicated through a Technical Specification amendment. The proposed change for Sequoyah Unit 2 will be considered at a later date.

Enclosure 6 of the August 16, 1982, letter deals with a typographical error contained in Table 4.4-5 involving the inadvertent interchange of vessel locations. The staff agrees that the correction should be made so that the location of capsule T is 40° and the location of capsule S is 4°.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: December 23, 1982

Principal Contributors: Melanie Miller, Licensing Branch No. 4, DL
Carl Stahle, Licensing Branch No. 4, DL
John Knox, PSB

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UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-327 AND 50-328

TENNESSEE VALLEY AUTHORITY

NOTICE OF ISSUANCE OF AMENDMENTS

FACILITY OPERATING LICENSE NOS. DPR-77 AND DPR-79

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 20 to Facility Operating License No. DPR-77 and Amendment No. 10 to Facility Operating License No. DPR-79, issued to Tennessee Valley Authority (licensee) for the Sequoyah Nuclear Plant, Units 1 and 2 (the facilities) located in Hamilton County, Tennessee. These amendments change the maximum isolation time for containment isolation valves, modify the surveillance requirements for testing of containment penetration protective fuses, and correct a typographical error in Table 4.4-5. The amendments are effective as of their dates of issuance.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5(d)(4) environmental impact statements, or negative declarations and environmental impact appraisals need not be prepared in connection with issuance of these amendments.

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For further details with respect to this action, see (1) Tennessee Valley Authority letter dated August 16, 1982, (2) Amendment No. 20 to Facility Operating License No. DPR-77 with Appendix A Technical Specification page changes; (3) Amendment No. 10 to Facility Operating License No. DPR-79 with Appendix A Technical Specification page changes; and (4) the Commission's related Safety Evaluation.

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Chattanooga Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37402. A copy of Amendment No. 20 and Amendment No. 10 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 23rd day of December 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

g/
Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

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| OFFICE | LA:DL:LB #4 | DL:LB #4 | DL:LB #4 | OELD | DL:LB #4 | | |
| SURNAME | MDuncan/hmc | MMiller | CStahle | <i>[Signature]</i> | EAdensam | | |
| DATE | 9/16/82 | 9/16/82 | 9/16/82 | 9/21/82 | 9/21/82 | | |

September 16, 1982

Docket Nos: 50-327
and 50-328

MEMORANDUM FOR: Thomas M. Novak, Assistant Director
for Licensing
Division of Licensing

THRU: Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

FROM: Carl Stahle, Project Manager
Licensing Branch No. 4
Division of Licensing

SUBJECT: ISSUANCE OF AMENDMENT NO. 20 TO FACILITY OPERATING
LICENSE DPR-77 AND AMENDMENT NO. 10 TO FACILITY
OPERATING LICENSE DPR-79, SEQUOYAH NUCLEAR PLANT,
UNITS 1 AND 2

Regarding the issuance of subject amendments, there is no known public
correspondence or irreversible impact associated with this subject.

5
Carl Stahle, Project Manager
Licensing Branch No. 4
Division of Licensing

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PDR ADOCK 05000327
P PDR

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|---------|-------------|----------|----------|----------|--|--|--|
| OFFICE | LA:DL:LB #4 | DL:LB #4 | DL:LB #4 | DL:LB #4 | | | |
| SURNAME | MDuncan/hmc | MNovak | CStahle | EAdensam | | | |
| DATE | 9/16/82 | 9/16/82 | 9/14/82 | 9/14/82 | | | |

AMENDMENT NO. 20 TO FACILITY OPERATING LICENSE DPR-77 - SEQUOYAH UNIT 1
AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE DPR-79 - SEQUOYAH UNIT 2

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✓ Docket No. 50-327/328
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