September 8, 1981

Docket Nos: 50-327
and 50-328

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 9 to Facility Operating License No. DPR-77 and Amendment No. 1 to Facility Operating License No. DPR-79 - Sequoyah Nuclear Plant, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 9 to Facility Operating License No. DPR-77 and Amendment No. 1 to Facility Operating License No. DPR-79.

These amendments extend the date by which ceiling tiles in the control rooms must be replaced, authorize replacement of certain flow rate monitors no later than the next refueling instead of September 1, 1981, and delete Appendix J requirements on an RHR supply line isolation valve.

A copy of the related safety evaluation supporting Amendment No. 9 to Facility Operating License DPR-77 and Amendment No. 1 to Facility Operating License DPR-79 is enclosed. Also enclosed is a copy of the Federal Register Notice which has been forwarded to the Office of the Federal Register for publication.

cp
1

Sincerely,

S/

Carl Stahle, Project Manager
Licensing Branch No. 4
Division of Licensing

Enclosures:
1. Amendment No. 9 to DPR-77
2. Amendment No. 1 to DPR-79
3. Safety Evaluation
4. Federal Register Notice

cc w/enclosures:
See next page

OFFICE	LA:DL:LB #4	DL:LB #4	DL:LB #4				
SURNAME	MDuncan/hmc	CStahle	EAdensam				
DATE	8/31/81	8/ /81	9/ /81				

SEQUOYAH

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

cc: Herbert S. Sanger, Jr., Esq.
General Counsel
Tennessee Valley Authority
400 Commerce Avenue
E 11B 33
Knoxville, Tennessee 37902

Mr. H. N. Culver
Tennessee Valley Authority
400 Commerce Avenue, 249A HBB
Knoxville, Tennessee 37902

Mr. Bob Faas
Westinghouse Electric Corp.
P.O. Box 355
Pittsburgh, Pennsylvania 15230

Mr. Mark Burzynski
Tennessee Valley Authority
400 Chestnut Street, Tower II
Chattanooga, Tennessee 37401

Mr. J. F. Cox
Tennessee Valley Authority
400 Commerce Avenue, W10C131C
Knoxville, Tennessee

Resident Inspector/Sequoyah NPS
c/o U.S. Nuclear Regulatory
Commission
2600 Igou Ferry Road
Soddy Daisy, Tennessee 37379

Director, Office of Urban
& Federal Affairs
108 Parkway Towers
404 James Robertson Way
Nashville, Tennessee 37219

Attorney General
Supreme Court Building
Nashville, Tennessee 37219

U.S. Environmental Protection
Agency
ATTN: EIS Coordinator
345 Courtland Street
Atlanta, Georgia 30308

U.S. Environmental Protection
Agency
ATTN: Ms. F. Munter
Office of Federal Activities
Room W-535, Waterside Mall
401 M Street, S.W.
Washington, D. C. 20460

Honorable Don Moore, Jr.
County Judge
Hamilton County Courthouse
Chattanooga, Tennessee 37402

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 9
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) having found that:

   A. The applications for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated August 14, August 20, and August 25, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended as follows:

   A. Add paragraph 2.C.(16)(c) to read:

      (c) By September 30, 1981, TVA shall replace the control room ceiling panels with panels acceptable to NRC.

   B. The Appendix A Technical Specifications are changed as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

OFFICE ▶							
SURNAME ▶							
DATE ▶							

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 9, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: September 8, 1981

OFFICE	LA:DL:LB #4	DL:LB #4	ETSB	CSB	OELD	DL:LB #4	AD:L/DL
SURNAME	MDuncan/hmc	CStahle	JBoegli	JPulsipher		EAdensam	RTedesco
DATE	8/31/81	8/31/81	8/31/81	8/31/81	8/9/81	9/3/81	9/4/81

ATTACHMENT TO LICENSE AMENDMENT NO. 9

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Overleaf Page	Amended Page
3/4 3-78	3/4 3-77
3/4 6-5	3/4 6-6

OFFICE ▶							
SURNAME ▶							
DATE ▶							

TABLE 3.3-13 (Continued)

TABLE NOTATION

* At all times.

** During waste gas disposal system operation.

*** During shield building exhaust system operation.

ACTION 40 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment for up to 14 days provided that prior to initiating the release:

a. At least two independent samples of the tank's contents are analyzed, and

b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valve lineup;

Otherwise, suspend release of radioactive effluents via this pathway.

ACTION 41 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days[1] provided the flow rate is estimated at least once per 4 hours.

ACTION 42 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 8 hours and these samples are analyzed for noble gas gross activity within 24 hours.

ACTION 43 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of this waste gas disposal system may continue for up to 7 days provided grab samples are collected at least once per 4 hours and analyzed within the following 4 hours. With the hydrogen and oxygen monitors inoperable, be in at least HOT STANDBY within 6 hours.

ACTION 44 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue for up to 30 days provided that within 4 hours after the channel has been declared inoperable samples are continuously collected with auxiliary sampling equipment as required in Table 4.11-2.

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[1] Until the replacement of the applicable flow rate monitors, but in no case later than before startup following the first refueling outage, this 30-day time limit is waived for effluent releases via the shield building exhaust, auxiliary building, and service building.

TABLE 3.6-1 (Continued)

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

PENETRATION		RELEASE LOCATION
X-84A	Pressurizer Relief Tank Gas Sample	Auxiliary Area
X-85A	Excess Letdown Heat Exchanger	Auxiliary Area
X-90	Control Air	Auxiliary Area
X-93	Accumulator Sample	Auxiliary Area
X-94ABC	Radiation Sample	Auxiliary Area
X-95ABC	Radiation Sample	Auxiliary Area
X-96C	Hot Leg Sample	Auxiliary Area
X-98	ILRT	Auxiliary Area
X-110	UHI	Auxiliary Area
X-114	Ice Condenser	Auxiliary Area
X-115	Ice Condenser	Auxiliary Area
X-400	Hydrogen Purge	Auxiliary Area

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 1
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) having found that:

   A. The applications for amendment to the Sequoyah Nuclear Plant, Unit 2 (the facility) Facility Operating License No. DPR-79 filed by the Tennessee Valley Authority (licensee), dated August 14, August 20, and August 25, 1981, comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended as follows:

   A. Change paragraph 2.C.(6)(c) to read:

      (c) TVA shall replace the control room ceiling panels with panels acceptable to NRC by September 30, 1981.

   B. The Appendix A Technical Specifications are changed as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 1, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

S/

Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: September 8, 1981

OFFICE	LA:DL:LB #4	DL:LB #4	BTSB	CSB	OELD	DL:LB #4	AD:L/DL
SURNAME	MDuncan/hmc	CSchulte	JBoegli	JPolsinelli		EAdensam	RTedesco
DATE	8/3/81	8/31/81	8/31/81	8/3/81	8/ /81	8/3/81	9/4/81

ATTACHMENT TO LICENSE AMENDMENT NO. 1

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Overleaf Page	Amended Page
3/4 3-80	3/4 3-79
3/4 6-5	3/4 6-6

OFFICE ▶							
SURNAME ▶							
DATE ▶							

TABLE 3.3-13 (Continued)

TABLE NOTATION

* At all times.

** During waste gas disposal system operation.

*** During shield building exhaust system operation.

ACTION 40 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, the contents of the tank(s) may be released to the environment for up to 14 days provided that prior to initiating the release:

a. At least two independent samples of the tank's contents are analyzed, and

b. At least two technically qualified members of the Facility Staff independently verify the release rate calculations and discharge valve lineup;

Otherwise, suspend release of radioactive effluents via this pathway.

ACTION 41 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days1/ provided the flow rate is estimated at least once per 4 hours.

ACTION 42 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via this pathway may continue for up to 30 days provided grab samples are taken at least once per 8 hours and these samples are analyzed for noble gas gross activity within 24 hours.

ACTION 43 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, operation of this waste gas disposal system may continue for up to 7 days provided grab samples are collected at least once per 4 hours and analyzed within the following 4 hours. With the hydrogen and oxygen monitors inoperable, be in at least HOT STANDBY within 6 hours.

ACTION 44 - With the number of channels OPERABLE less than required by the Minimum Channels OPERABLE requirement, effluent releases via the affected pathway may continue for up to 30 days provided that within 4 hours after the channel has been declared inoperable samples are continuously collected with auxiliary sampling equipment as required in Table 4.11-2.

1/Until the replacement of the applicable flow rate monitors, but in no case later than before startup following the first refueling outage, this 30-day time limit is waived for effluent releases via the shield building exhaust, auxiliary building, and service building.

TABLE 3.6-1 (Continued)

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

PENETRATION		RELEASE LOCATION
X-84A	Pressurizer Relief Tank Gas Sample	Auxiliary Area
X-85A	Excess Letdown Heat Exchanger	Auxiliary Area
X-90	Control Air	Auxiliary Area
X-93	Accumulator Sample	Auxiliary Area
X-94ABC	Radiation Sample	Auxiliary Area
X-95ABC	Radiation Sample	Auxiliary Area
X-96C	Hot Leg Sample	Auxiliary Area
X-98	ILRT	Auxiliary Area
X-110	UHI	Auxiliary Area
X-114	Ice Condenser	Auxiliary Area
X-115	Ice Condenser	Auxiliary Area
X-400	Hydrogen Purge	Auxiliary Area

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE DPR-77

AND AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE DPR-79

TENNESSEE VALLEY AUTHORITY

INTRODUCTION

The licensee proposed three separate changes to the operating licenses for Sequoyah Units 1 and 2 which are as follows:

a) Installation of the new ceiling tile will be completed by September 30, 1981 instead of September 1, 1981 (TVA letter dated 8/20/81).

b) Replacement of certain flow rate monitors no later than the next refueling instead of September 1, 1981 (TVA letter dated 8/14/81).

c) Deletion of Appendix J requirements on an RHR supply line isolation valve (TVA letter dated 8/25/81).

EVALUATION

a) Ceiling Tile

In Supplement No. 5 to the SER the staff determined that new ceiling tile was needed in the control room prior to exceeding 5 percent of full power. Subsequent to the issuance of Supplement No. 5 the staff accepted a September 1, 1981, completion date and conditioned the Unit 2 low power license accordingly. Installation of the tile is now expected by September 30, 1981, due to procurement delays. This is acceptable to the staff since no additional safety hazards are introduced for the extended period of time.

b) Replacement of Certain Flow Rate Monitors

The note on Table 3.3-13, Radiological Gaseous Effluent Monitoring Instrumentation, in the Technical Specifications states that the identified flow rate monitors are to be replaced by September 1, 1981. In the interim, acceptable plant procedures are used to estimate the exhaust flow instead of these flow rate monitors. Procurement problems, however, make it impossible for TVA to comply with the September 1 deadline. TVA expects to receive the monitors by March 1982, and they will install them at the next convenient outage at the facility.

In any case, the monitors will be installed no later than prior to the next startup after the first refueling. This is acceptable to the NRC staff because the existing procedures are acceptable. The staff concludes that no additional safety risks will be incurred by continuing use of the procedural measures.

c) Deletion of Appendix J Requirement

The licensee requests that the RHR supply line containment isolation valve (74-2) be removed from the local leakage rate testing program required by Appendix J to 10 CFR 50. The licensee contends that this line is not a potential path for containment leakage during an accident.

The NRC staff concurs with the TVA analysis that local leakage rate testing of valve (74-2) is not required by Appendix J. We also agree that such testing is not practical in certain modes of plant operation and could be dangerous to test personnel in the remaining modes. However, this valve can be tested, and is tested, as required by 10 CFR 50, 50A and the ASME Boiler and Pressure Vessel Code Section XI, at least once every nine months when coming back from cold shutdown. In this test, the leakage rate is determined by a method which is different from, and cannot be substituted for, the method required by Appendix J. Nevertheless, the Section XI test gives some assurance of limiting valve leakage.

For the above reasons, and those set forth in the TVA submittal, deleting the Appendix J test of containment isolation valve (74-2) does not add further safety risks to the facility.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any signigicant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR §51.5 (d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

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CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: September 8, 1981

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-327 AND 50-328

TENNESSEE VALLEY AUTHORITY

NOTICE OF ISSUANCE OF AMENDMENTS

FACILITY OPERATING LICENSE NOS. DPR-77 AND DPR-79

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 9 to Facility Operating License No. DPR-77 and Amendment No. 1 to Facility Operating License DPR-79, issued to Tennessee Valley Authority (licensee) for the Sequoyah Plant, Units 1 and 2 (the facilities) located in Hamilton County, Tennessee. These amendments extend the date by which ceiling tiles in the control rooms must be replaced, authorize replacement of certain flow rate monitors no later than the next refueling instead of September 1, 1981, and delete 10 CFR Part 50, Appendix J requirements on an RHR supply line isolation valve. The amendments are effective as of their dates of issuance.

The applications for the amendments comply with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR §51.5 (d)(4) environmental impact statements, or negative declarations and environmental impact appraisals need not be prepared in connection with issuance of these amendments.

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

For further details with respect to this action, see (1) Tennessee Valley Authority letters dated August 14, August 20, and August 25, 1981; (2) Amendment No. 9 to Facility Operating License No. DPR-77 with Appendix A Technical Specification page changes; (3) Amendment No. 1 to Facility Operating License No. DPR-79 with Appendix A Technical Specification page changes; and (4) the Commission's related Safety Evaluation.

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Chattanooga Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37402. A copy of Amendment No. 9 and Amendment No. 1 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. c. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 8th day of September 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

S/

Elinor G. Adensam, Acting Chief
Licensing Branch No. 4
Division of Licensing

OFFICE	LA:DL:LB #4	DL:LB #4	OELD (Legal Only)	DL:LB #4			
SURNAME	MDungan/hmc	CStahle		EAdensam			
DATE	8/3/81	8/3/81	8/7/81	8/5/81			

AMENDMENT NO. 9 TO FACILITY OPERATING LICENSE - SEQUOYAH NUCLEAR PLANT - 1
AMENDMENT NO. 1 TO FACILITY OPERATING LICENSE - SEQUOYAH NUCLEAR PLANT - 2

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