

December 31, 1981

Docket No. 50-327

Mr. H. G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 10 to Facility Operating License  
No. DPR-77 - Sequoyah Nuclear Plant, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-77.

This amendment is in response to a portion of your letter dated November 23, 1981. The remaining items will be addressed in a subsequent amendment. The amendment revises the implementation dates of six items from January 1, 1982, to no later than startup following the first refueling outage.

A copy of the related safety evaluation supporting Amendment No. 10 to Facility Operating License DPR-77 is enclosed. Also enclosed is a copy of the Federal Register notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

*S/*

*CP*  
*1*

Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

8201150138 811231  
PDR ADOCK 05000327  
P PDR

Enclosures:

1. Amendment No. 10
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures:  
See next page

\*See previous white

OFFICE	DL:LB#4	DL:LB#4	DL:LB#4	ELD			
SURNAME	*MDuncan:eb	*CStahle	EAdensam	*RPerlis			
DATE	*12/16/81	*12/16/81	12/20/81	*12/21/81			

Docket No. 50-327

Mr. H. G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 10 to Facility Operating License  
No. DPR-77 - Sequoyah Nuclear Plant, Unit 1

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 10 to Facility Operating License No. DPR-77.

This amendment is in response to a portion of your letter dated November 23, 1981. The remaining item will be addressed in a subsequent amendment. The amendment revises the implementation dates of six items from January 1, 1982, and one from January 31, 1982, to no later than startup following the first refueling outage.

A copy of the related safety evaluation supporting Amendment No. 10 to Facility Operating License DPR-77 is enclosed. Also enclosed is a copy of the Federal Register Notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

Enclosures:

1. Amendment No. 10
2. Safety Evaluation
3. Federal Register Notice

cc w/enclosures:  
See next page

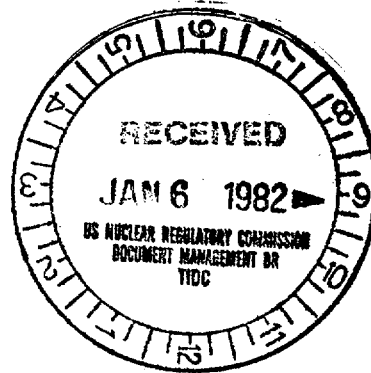
*Notice of Amendment only*

OFFICE ▶	...DL:LB#4.....	...DL:LB#4.....	...DL:LB#4.....	...ELD.....			
SURNAME ▶	...MDuncan:md.....	...OSTahle.....	...EAdensam.....	...R... ..			
DATE ▶	...12/16/81.....	...12/16/81.....	...12/...../81.....	...12/21/81.....			

AMENDMENT NO.10 TO FACILITY OPERATING LICENSE - SEQUOYAH NUCLEAR PLANT - 1

DISTRIBUTION:

✓ Docket Nos. 50-327  
NRC PDR  
Local PDR  
LB #4 r/f  
Attorney, ELD  
DEisenhut  
RPurple  
RLTedesco  
CStahle  
TKenyon  
MDuncan  
IE (5)  
RHartfield  
IBailey, OA (4)  
RDiggs, ADM  
SHanauer  
TMurley  
RMattson  
RVollmer  
TMNovak  
GLainas  
JKnight  
DMuller  
PCheck  
LRubenstein  
WKreger  
MErnst  
FMiraglia  
ASchwencer  
BJYoungblood  
JRMiller  
BScharf, ADM (10)  
ACRS (16)  
JRBuchanan, NSIC  
TERA  
ARosenthal, ASLAB  
PCotter, ASLBP



SEQUOYAH

Mr. H. G. Parris  
Manager of Power  
Tennessee Valley Authority  
500A Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

cc: Herbert S. Sanger, Jr., Esq.  
General Counsel  
Tennessee Valley Authority  
400 Commerce Avenue  
E 11B 33  
Knoxville, Tennessee 37902

Mr. H. N. Culver  
Tennessee Valley Authority  
400 Commerce Avenue, 249A HBB  
Knoxville, Tennessee 37902

Mr. Bob Faas  
Westinghouse Electric Corp.  
P.O. Box 355  
Pittsburgh, Pennsylvania 15230

Mr. Mark Burzynski  
Tennessee Valley Authority  
400 Chestnut Street, Tower II  
Chattanooga, Tennessee 37401

Mr. J. F. Cox  
Tennessee Valley Authority  
400 Commerce Avenue, W10C131C  
Knoxville, Tennessee 37902

Resident Inspector/Sequoyah NPS  
c/o U.S. Nuclear Regulatory  
Commission  
2600 Igou Ferry Road  
Soddy Daisy, Tennessee 37379

Director, Office of Urban  
& Federal Affairs  
108 Parkway Towers  
404 James Robertson Way  
Nashville, Tennessee 37219

Attorney General  
Supreme Court Building  
Nashville, Tennessee 37219

U.S. Environmental Protection  
Agency  
ATTN: EIS Coordinator  
345 Courtland Street  
Atlanta, Georgia 30308

Honorable Don Moore, Jr.  
County Judge  
Hamilton County Courthouse  
Chattanooga, Tennessee 37402

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 10  
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated November 23, 1981, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, Facility Operating License DPR-77 is amended by the following changes:
  - A. Change paragraph 2.C.(23)D. to read as follows:

D. Additional Accident Monitoring Instrumentation (Section 22.3, II.F.1)

(1) TVA shall install interim noble gas monitors at the first outage of sufficient duration.

8201150151 811231  
PDR ADOCK 05000327  
P PDR

OFFICE							
SURNAME							
DATE							

- (2) At the first outage of sufficient duration, but no later than startup following the first refueling outage, TVA shall install the following qualified monitoring instrumentation:
  - (a) Integrated monitoring assembly which will accomplish particulate, iodine and noble gas monitoring.
  - (b) Containment high range radiation monitor.
  - (c) Containment pressure monitor.
  - (d) Containment water level monitor.
  - (e) Containment hydrogen monitor.

B. Delete paragraph 2.C(23)G.

C. Change paragraph 2.C.(23)E. to read as follows:

E. Reactor Coolant System Vents (Section 22.3, II.B 1)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor coolant system and reactor vessel head highpoint vents that are remotely operable from the control room

D. Change paragraph 2.C.(23)F. to read as follows.

F. Post Accident Sampling (Section 22.3, II.B.3)

At the first outage of sufficient duration, but no longer than startup following first refueling outage, TVA shall complete corrective actions needed to provide the capability to promptly obtain and perform radioisotopic and chemical analysis of reactor coolant and containment atmosphere samples under degraded core conditions with excessive exposure.

E. Change paragraph 2.C.(23)H. to read as follows:

H. Instruments of Inadequate Core Cooling (Section 22.3, II.F.2)

- (1) By January 1, 1982, TVA shall install a backup indication for incore thermocouples. This display shall be in the control room and cover the temperature range of 200 F - 2000 F.

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

- (2) At the first outage of sufficient duration, but no later than startup following first refueling outage. TVA shall install reactor vessel water level instrumentation and the system will meet seismic and environmental requirements.

F. Change paragraph 2.C.(23)I. to read as follows:

I. Upgrade Emergency Support Facilities (Section 22.3, II.A.1.2)

- (1) The installation of the TSC shall be completed prior to startup after the first refueling. However, if an outage scheduled to last more than five weeks occurs after May 1, 1982, installation of the necessary modifications to the control room and plant instrumentation will be completed at that time and the TSC hardware installation will be completed within eleven weeks of the start of this scheduled outage.
- (2) TVA shall maintain interim emergency support facilities (Technical Support Center, Operations Support Center and the Emergency Operations Facility) until the final facilities are complete.

G. Change paragraph 2.C.(23)J. to read as follows:

J. Relief and Safety Valve Test Requirements (Section 22.3, II.D.1)

TVA shall conform to the results of the EPRI test program. TVA shall provide documentation for qualifying (a) reactor coolant system relief and safety valves, (b) piping and supports, and (c) block valves in accordance with the review schedule given in SECY-81-491 as approved by the Commission.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

*5/ Gus Larios*  
for  
Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Date of Issuance:

\*NOTE: SEE PREVIOUS WHITE FOR CONCURRENCE

OFFICE	DL:LB.#4/LA	DL:LB.#4	OELD	DL:LB.#4	DL	DL DIR
SURNAME	*MDuncan/hmc	*CStahle	*	*EAdensam	*Rtedesco	*DEisenhut
DATE	12/16/81	12/16/81	12/21/81	12/30/81	12/30/81	12/31/81

(2) At the first outage of sufficient duration, but no later than startup following the first refueling outage, TVA shall install the following qualified monitoring instrumentation:

- (a) Integrated monitoring assembly which will accomplish particulate, iodine and noble gas monitoring.
- (b) Containment high range radiation monitor.
- (c) Containment pressure monitor.
- (d) Containment water level monitor.
- (e) Containment hydrogen monitor.

B. Delete paragraph 2.C.(23)G.

C. Change paragraph 2.C.(23)E. to read as follows:

E. Reactor Coolant System Vents (Section 22.3, II.B.1)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor coolant system and reactor vessel head highpoint vents that are remotely operable from the control room.

D. Change paragraph 2.C.(23)F. to read as follows.

F. Post Accident Sampling (Section 22.3, II.B.3)

At the first outage of sufficient duration, but no longer than startup following first refueling outage, TVA shall complete corrective actions needed to provide the capability to promptly obtain and perform radioisotopic and chemical analysis of reactor coolant and containment atmosphere samples under degraded core conditions with excessive exposure.

E. Change paragraph 2.C.(23)H. to read as follows:

H. Instruments of Inadequate Core Cooling (Section 22.3, II.F.2)

- (1) By January 1, 1982, TVA shall install a backup indication for incore thermocouples. This display shall be in the control room and cover the temperature range of 200 F - 2000 F.

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....



(d) Containment water level monitor.

(e) Containment hydrogen monitor.

C. Delete paragraph 2.C.(23)G.

D. Change paragraph 2.C.(23)E. to read as follows:

E. Reactor Coolant System Vents (Section 22.3, II.B.1)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor coolant system and reactor vessel head highpoint vents that are remotely operable from the control room.

E. Change paragraph 2.C.(23)F. to read as follows:

F. Post Accident Sampling (Section 22.3, II.B.3)

At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall complete corrective actions needed to provide the capability to promptly obtain and perform radioisotopic and chemical analyses of reactor coolant and containment atmosphere samples under degraded core conditions without excessive exposure.

F. Change paragraph 2.C.(23)H. to read as follows:

H. Instruments for Inadequate Core Cooling (Section 22.3, II.F.2)

- (1) By January 1, 1982, TVA shall install a backup indication for incore thermocouples. This display shall be in the control room and cover the temperature range of 200 F - 2000 F.
- (2) At the first outage of sufficient duration, but no later than startup following first refueling outage, TVA shall install reactor vessel water level instrumentation and the system will meet seismic and environmental requirements.

G. Change paragraph 2.C.(23)I. to read as follows:

I. Upgrade Emergency Support Facilities (Section 22.3, II.A.1.2)

- (1) The installation of the TSC shall be completed prior to startup after the first refueling. However, if an outage scheduled to last more than five weeks occurs after May 1, 1982, installation of the necessary modifications to the control room and plant instrumentation will be completed at that time and the TSC hardware installation will be completed within eleven weeks of the start of this scheduled outage.

OFFICE ▶	.....	.....	.....	.....	.....	.....	.....
SURNAME ▶	.....	.....	.....	.....	.....	.....	.....
DATE ▶	.....	.....	.....	.....	.....	.....	.....

(2) TVA shall maintain interim emergency support facilities (Technical Support Center, Operations Support Center and the Emergency Operations Facility) until the final facilities are complete.

H. Change paragraph 2.C.(23)J. to read as follows:

J. Relief and Safety Valve Test Requirements (Section 22.3, II.D.1)

TVA shall conform to the results of the EPRI test program. TVA shall provide documentation for qualifying (a) reactor coolant system relief and safety valves, (b) piping and supports, and (c) block valves in accordance with the review schedule given in SECY-81-491 as approved by the Commission.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Darrell G. Eisenhut, Director  
Division of Licensing  
Office of Nuclear Reactor Regulation

Date of Issuance:

*notice and audit*

OFFICE	DL:LB#4	DL:LB#4	OEID	DL:LB#4	AD/DL	DL	
SURNAME	MDuncan:md	CStahle	<i>Mark...</i>	EAdamsam	RTedesco	DEisenhut	
DATE	12/16/81	12/16/81	12/16/81	12/19/81	12/19/81	12/19/81	

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION  
RELATED TO AMENDMENT NO. 10 TO FACILITY OPERATING LICENSE DPR-77

TENNESSEE VALLEY AUTHORITY

INTRODUCTION

On November 23, 1981, TVA proposed eight revisions to the Sequoyah Unit 1 operating license. With the exception of item 1 of their enclosure, the bases for the revisions are identical to those presented for Unit 2, and those changes were discussed and found acceptable to the Commission at the meeting on September 8, 1981. Item 1 and Item 2 will be covered in a separate amendment.

DISCUSSION

The proposed revisions to the license request that the implementation dates of 6 items be changed from January 1, 1982, to no later than startup following the first refueling outage. Additional interim measures are also requested for some of these items; however, all are consistent with the Commission approved license conditions on Unit 2. TVA has provided "good cause" for the delays encountered to date on the items. Furthermore, an extensive outage time would be required to install the required items which is considered unwarranted. For this reason, the next refueling outage was chosen as an appropriate implementation date in order to avoid an unnecessary plant outage.

ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

DATED: December 31, 1981

8201150163 811231  
PDR ADDCK 05000327  
P PDR

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NO. 50-327

TENNESSEE VALLEY AUTHORITY

NOTICE OF ISSUANCE OF AMENDMENT

FACILITY OPERATING LICENSE NO. DPR-77

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 10 to Facility Operating License No. DPR-77, issued to Tennessee Valley Authority (licensee) for the Sequoyah Nuclear Plant, Unit 1 (the facility) located in Hamilton County, Tennessee. This amendment revises implementation dates of several items from January 1 ~~and 31~~, 1982, to no later than startup following the first refueling outage.

The application for the amendment complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendment. Prior public notice of this amendment was not required since the amendment does not involve a significant hazards consideration.

The Commission has determined that the issuance of this amendment will not result in any significant environmental impact and that pursuant to 10 CFR 51.5 (d)(4) an environmental impact statement, or negative declaration and environmental impact appraisal need not be prepared in connection with issuance of this amendment.

For further details with respect to this action, see (1) Tennessee Valley Authority letter dated November 23, 1981, (2) Amendment No. 10 to Facility Operating License No. DPR-77, and (3) the Commission's related Safety Evaluation.

OFFICE							
SURNAM	8201150172	811231					
	PDR ADOCK	05000327					
DATE	P	PDR					

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Chattanooga Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37402. A copy of Amendment No. 10 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 31 day of December 1981.

FOR THE NUCLEAR REGULATORY COMMISSION

*S/*

Elinor G. Adebsam, Chief  
Licensing Branch No. 4  
Division of Licensing

OFFICE	DL:LB#4	DL:LB#4	OELD	DL:LB#4			
SURNAME	M. Duncan	C. Stahl	<i>noted</i>	E. Adebsam			
DATE	12/16/81	12/17/81	12/24/81	12/30/81			

DISTRIBUTION:

Docket No.  
LB #4 r/f  
E. Adensam  
C. Stahle  
M. Duncan  
OELD

Docket No. 50-327

December 21, 1981

MEMORANDUM FOR: Robert L. Tedesco, Assistant Director  
for Licensing  
Division of Licensing

THRU: Elinor G. Adensam, Chief  
Licensing Branch No. 4  
Division of Licensing

FROM: Carl Stahle, Project Manager  
Licensing Branch No. 4  
Division of Licensing

SUBJECT: ISSUANCE OF AMENDMENT NO. 10 TO FACILITY OPERATING  
LICENSE DPR-77, SEQUOYAH NUCLEAR PLANT, UNIT 1

There is no known public correspondence or irreversible impact associated with the issuance of the subject amendment.

5/

Carl Stahle, Project Manager  
Licensing Branch No. 4  
Division of Licensing

B201150175 B11231  
PDR ADDCK 05000327  
P PDR

*notice + comment only*

OFFICE	DL:LB#4	DL:LB#4				
SURNAME	MDuncan:hmc.	CStahle				
DATE	12/18/81	12/18/81	12/21/81			