



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

July 29, 1996

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: ISSUANCE OF AMENDMENTS - MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M94904 AND M94905)

Dear Mr. McMeekin:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 168 to Facility Operating License NPF-9 and Amendment No. 150 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated March 4, 1996.

The amendments delete Flow Monitoring System from TS 3.4.6.1 and associated surveillance requirements.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Victor Nerses".

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosures:

1. Amendment No. 168 to NPF-9
2. Amendment No. 150 to NPF-17
3. Safety Evaluation

cc w/encl: See next page

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Mr. T. C. McMeekin
 Vice President, McGuire Site
 Duke Power Company
 12700 Hagers Ferry Road
 Huntersville, NC 28078-8985

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Sincerely,

original signed by
 Victor Nerses, Senior Project Manager
 Project Directorate II-2
 Division of Reactor Projects - I/II
 Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

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cc w/encl: See next page

DOCUMENT NAME: G:\MCGUIRE\M94904.AMD

OFFICE	PDII-2/A	PDII-2/PM	SPLB	OGC	PDII-2/D
NAME	L.BERRY	V.NERSES	T.MARSH	HOLLER	H.BERKOW
DATE	6/23/196	7/3/196	7/16/196	7/24/196	7/26/196
COPY	YES	YES NO	YES NO	YES NO	YES NO

OFFICIAL RECORD COPY

Mr. T. C. McMeekin
Duke Power Company

McGuire Nuclear Station

cc:

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UNITED STATES
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DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 168
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated March 4, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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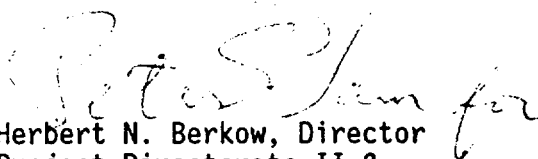
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 168, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: July 29, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 168

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove Page

3/4 4-19

Insert Page

3/4 4-19

REACTOR COOLANT SYSTEM

3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE

LEAKAGE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.4.6.1 The following Reactor Coolant System Leakage Detection Systems shall be OPERABLE:

- a. The Containment Atmosphere Gaseous Radioactivity Monitoring System,
- b. The Containment Floor and Equipment Sump Level System, and
- c. Either the Containment Ventilation Condensate Drain Tank Level Monitoring System or a Containment Atmosphere Particulate Radioactivity Monitoring System.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With only two of the above required Leakage Detection Systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required Gaseous or Particulate Radioactivity Monitoring System is inoperable; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.1 The Leakage Detection Systems shall be demonstrated OPERABLE by:

- a. Containment Atmosphere Gaseous and Particulate Radioactivity Monitoring Systems-performance of CHANNEL CHECK, CHANNEL CALIBRATION and ANALOG CHANNEL OPERATIONAL TEST at the frequencies specified in Table 4.3-3.
- b. Containment Floor and Equipment Sump Level System-performance of CHANNEL CALIBRATION at least once per 18 months, and
- c. Containment Ventilation Condensate Drain Tank Level Monitoring System-performance of CHANNEL CALIBRATION at least once per 18 months.



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DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 150
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated March 4, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

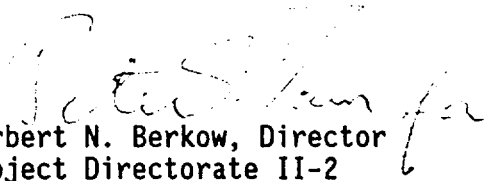
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 150 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION


Herbert N. Berkow, Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: July 29, 1996

ATTACHMENT TO LICENSE AMENDMENT NO. 150

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains vertical lines indicating the areas of change.

Remove Page

3/4 4-19

Insert Page

3/4 4-19

REACTOR COOLANT SYSTEM

3/4.4.6 REACTOR COOLANT SYSTEM LEAKAGE

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LIMITING CONDITION FOR OPERATION

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- a. The Containment Atmosphere Gaseous Radioactivity Monitoring System,
- b. The Containment Floor and Equipment Sump Level System, and
- c. Either the Containment Ventilation Condensate Drain Tank Level Monitoring System or a Containment Atmosphere Particulate Radioactivity Monitoring System.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With only two of the above required Leakage Detection Systems OPERABLE, operation may continue for up to 30 days provided grab samples of the containment atmosphere are obtained and analyzed at least once per 24 hours when the required Gaseous or Particulate Radioactivity Monitoring System is inoperable; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.6.1 The Leakage Detection Systems shall be demonstrated OPERABLE by:

- a. Containment Atmosphere Gaseous and Particulate Radioactivity Monitoring Systems-performance of CHANNEL CHECK, CHANNEL CALIBRATION and ANALOG CHANNEL OPERATIONAL TEST at the frequencies specified in Table 4.3-3.
- b. Containment Floor and Equipment Sump Level System-performance of CHANNEL CALIBRATION at least once per 18 months, and
- c. Containment Ventilation Condensate Drain Tank Level Monitoring System-performance of CHANNEL CALIBRATION at least once per 18 months.



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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 168 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 150 TO FACILITY OPERATING LICENSE NPF-17
DUKE POWER COMPANY
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated March 4, 1996, Duke Power Company (the licensee) submitted a request for changes to the McGuire Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would delete the reactor coolant system (RCS) flow monitoring system from the TS and Surveillance Requirements. Currently, TS 3/4.4.6 "Reactor Coolant System Leakage" has a Limiting Condition for Operation (LCO) (LCO 3.4.6.1b), that requires either the containment floor and equipment sump level system or the flow monitoring system to be OPERABLE when in Modes 1, 2, 3, and 4. The proposed change would revise LCO 3.4.6.1b to require specifically that the containment floor and equipment sump level system to be OPERABLE and eliminate the flow monitoring system as a viable alternative. The associated Surveillance Requirement 4.4.6.1b would also be revised to delete the surveillance of the flow monitoring system. The stated purpose of the proposed change is that the flow monitoring system cannot be used to monitor unidentified RCS leakage because of known inaccuracies caused by mechanical flaws in the initial design of the system.

2.0 EVALUATION

The current TS requires three RCS leakage detection systems to be operable. One of these systems, the gaseous radioactivity monitoring system, has no alternative system and must be operable in all applicable modes. For each of the other required systems, two alternatives are given in the LCO. The first alternative is a choice between the containment floor and equipment sump level system and the flow monitoring system. The second alternative is a choice between the containment ventilation condensate drain tank level monitoring system and the containment atmosphere particulate radioactivity monitoring system. The proposed change would only eliminate the first alternative with no changes to the rest of the required RCS leakage detection system TS requirements. The proposed change still requires three RCS leakage detection systems to be OPERABLE and is, therefore, more restrictive because one of the alternatives has been removed. The proposed change is also conservative in nature because it will prevent reliance on an inaccurate leakage detection method to satisfy

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an LCO. The staff, therefore, concludes that the proposed change to delete the flow monitoring system from the TS is acceptable.

Based on its evaluation, the staff concludes that the proposed change to eliminate the flow monitoring system and its surveillance requirements from the plant TS is necessary to prevent the potential for reliance on an inaccurate monitoring system and is more restrictive and more conservative than the current specification. The staff, therefore, concludes that the proposed changes to TS 3/4.4.6 are acceptable and should be approved.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (61 FR 18166 dated April 24, 1996). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: W. LeFave

Date: July 29, 1996