

October 22, 1998

Mr. H. B. Barron
Vice President, McGuire Site
Duke Energy Corporation
12700 Hagers Ferry Road
Huntersville, NC 28078-8985

SUBJECT: REVISION TO TECHNICAL SPECIFICATIONS BASES - MCGUIRE NUCLEAR
STATION, UNITS 1 AND 2 (TAC NOS. MA3362 AND MA3363)

Dear Mr. Barron:

By letter dated June 29, 1998, Duke Energy Corporation (DEC) provided changes to the Technical Specifications (TS) Bases Section 3/4.6.3. This revision corrected editorial errors in the TS Bases. DEC provided corrected page B 3/4 6-4 for information only. As you are aware, the TS Bases are not part of the TS as defined by Title 10 of the Code of Federal Regulations (10 CFR) Section 50.36. Changes to the TS Bases may voluntarily be made in accordance with the provisions of 10 CFR 50.59. Should the proposed change involve an unreviewed safety question, pursuant to 10 CFR 50.59(a)(2), or involve a change in the interpretation of implementation of the TS (i.e., constitute a TS change), then the proposed change is to be provided to the staff pursuant to the provisions of 10 CFR 50.59(c) and 10 CFR 50.90 for prior staff review and approval.

The subject TS Bases pages you provided are hereby returned to you for insertion in the TS to ensure that both the staff and DEC have the identical TS Bases pages. The staff did not perform an evaluation of your TS Bases revisions, and staff concurrence with the revisions is not implied by virtue of this letter. The staff may review the evaluations that support these TS Bases revisions during the next inspection of McGuire, Units 1 and 2, implementation of 10 CFR 50.59.

Sincerely,

ORIGINAL SIGNED BY:

Frank Rinaldi, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000369
P PDR

Docket Nos. 50-369 and 50-370

Enclosure: As stated

cc w/encl: See next page

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NAME	FRinaldi:cn	LBerry	HBerkow				
DATE	10/22/98	10/12/98	10/12/98	1/198	1/198	1/197	



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in black ink that reads "Frank Rinaldi".

Frank Rinaldi, Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosure: As stated

cc w/encl: See next page

McGuire Nuclear Station

cc:

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McGuire Nuclear Station, Units 1 and 2

Instructions for Bases page Insert and Removal

- | | |
|--------|--|
| Unit 1 | Remove Bases page B 3/4 6-4
Replace with revised Bases page B 3/4 6-4 |
| Unit 2 | Remove Bases page B 3/4 6-4
Replace with revised Bases page B 3/4 6-4 |

CONTAINMENT SYSTEMS

BASES

3/4.6.1.9 CONTAINMENT VENTILATION SYSTEM (Continued)

Leakage integrity tests with a maximum allowable leakage rate for containment purge supply and exhaust supply valves will provide early indication of resilient material seal degradation and will allow opportunity for repair before gross leakage failures could develop. The 0.60 L_a leakage limit of Specification 3.6.1.2b. shall not be exceeded when the leakage rates determined by the leakage integrity tests of these valves are added to the previously determined total for all valves and penetrations subject to Type B and C tests.

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

CONTAINMENT SPRAY SYSTEM

The OPERABILITY of the Containment Spray System ensures that containment depressurization and cooling capability will be available in the event of a LOCA. The pressure reduction and resultant lower containment leakage rate are consistent with the assumptions used in the accident analyses.

3/4.6.3 CONTAINMENT ISOLATION VALVES

The OPERABILITY of the containment isolation valves ensures that the containment atmosphere will be isolated from the outside environment in the event of a release of radioactive material to the containment atmosphere or pressurization of the containment and is consistent with the requirements of GDC 54 thru 57 of Appendix A to 10 CFR Part 50. Containment isolation within the time limits specified for those isolation valves designed to close automatically ensures that the release of radioactive material to the environment will be consistent with the assumptions used in the analyses for a LOCA.

Containment isolation valves are listed in UFSAR Tables 6-111 through 113. Those valves with a required isolation time have a value given in the "MAX ISOLATION TIME (SEC)" column. Penetration test type (type B, type C, or None) is listed in the "TEST TYPE" column of the table for each containment penetration. Changes to the UFSAR are made in accordance with 10 CFR 50.59.

3/4.6.4 COMBUSTIBLE GAS CONTROL

The OPERABILITY of the equipment and systems required for the detection and control of hydrogen gas ensures that this equipment will be available to maintain the hydrogen concentration within containment below its flammable limit during post-LOCA conditions. Either recombiner unit is capable of controlling the expected hydrogen generation associated with: (1) zirconium-water reactions, (2) radiolytic decomposition of water, and (3) corrosion of metals within containment. These hydrogen control systems are consistent with the recommendations of Regulatory Guide 1.7, "Control of Combustible Gas Concentrations in Containment Following a LOCA", March 1971.

CONTAINMENT SYSTEMS

BASES

3/4.6.1.9 CONTAINMENT VENTILATION SYSTEM (Continued)

Leakage integrity tests with a maximum allowable leakage rate for containment purge supply and exhaust supply valves will provide early indication of resilient material seal degradation and will allow opportunity for repair before gross leakage failures could develop. The 0.60 L_v leakage limit of Specification 3.6.1.2b. shall not be exceeded when the leakage rates determined by the leakage integrity tests of these valves are added to the previously determined total for all valves and penetrations subject to Type B and C tests.

3/4.6.2 DEPRESSURIZATION AND COOLING SYSTEMS

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