

August 17, 1982

Docket Nos: 50-327
and 50-328

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

Dear Mr. Parris:

Subject: Issuance of Amendment No. 15 to Facility Operating License
No. DPR-77 and Amendment No. 6 to Facility Operating
License No. DPR-79 - Sequoyah Nuclear Plant, Units 1 and 2

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 15 to Facility Operating License No. DPR-77 and Amendment No. 6 to Facility Operating License No. DPR-79 for the Sequoyah Nuclear Plant, Units 1 and 2, located in Hamilton County, Tennessee. The amendments consist of changes to the Technical Specifications in response to your application telecopied to us on August 2, 1982. The amendments were authorized by telephone on August 3, 1982, and were confirmed by letter dated August 4, 1982.

The amendments delete the requirement for pressurizer relief tank level instrumentation. These amendments were issued on an expedited basis to permit Unit 2 to remain at 100 percent power and avoid any potential disruption of Unit 1 operations.

A copy of the related safety evaluation supporting Amendment No. 15 to Facility Operating License DPR-77 and Amendment No. 6 to Facility Operating License No. DPR-79 is enclosed. Also enclosed is a copy of the Federal Register Notice which has been forwarded to the Office of the Federal Register for publication.

Sincerely,

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

8209020039 820817
PDR ADOCK 05000327
P PDR

Enclosures:

1. Amendment No. 15 to DPR-77
2. Amendment No. 6 to DPR-79
3. Safety Evaluation
4. Federal Register Notice

cc w/enclosures:

See next page

OFFICE	LA:DL LB #4	DL:LB #4	DL:LB #4				
SURNAME	MDuncan/hmc	CSchale	EAdensam				
DATE	8/5/82	8/5/82	8/5/82				

SEQUOYAH

Mr. H. G. Parris
Manager of Power
Tennessee Valley Authority
500A Chestnut Street, Tower II
Chattanooga, Tennessee 37401

cc: Herbert S. Sanger, Jr., Esq.
General Counsel
Tennessee Valley Authority
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E 11B 33
Knoxville, Tennessee 37902

Mr. H. N. Culver
Tennessee Valley Authority
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Mr. Bob Faas
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Pittsburgh, Pennsylvania 15230

Mr. Jerry Wills
Tennessee Valley Authority
400 Chestnut Street, Tower II
Chattanooga, Tennessee 37401

Mr. J. F. Cox
Tennessee Valley Authority
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Knoxville, Tennessee 37902

Resident Inspector/Sequoyah NPS
c/o U.S. Nuclear Regulatory
Commission
2600 Igou Ferry Road
Soddy Daisy, Tennessee 37379

Attorney General
Supreme Court Building
Nashville, Tennessee 37219

U.S. Environmental Protection
Agency
ATTN: EIS Coordinator
345 Courtland Street
Atlanta, Georgia 30308

Honorable Don Moore, Jr.
County Judge
Hamilton County Courthouse
Chattanooga, Tennessee 37402

Regional Administrator
Nuclear Regulatory Commission,
Region II
101 Marietta Street, Suite 3100
Atlanta, Georgia 30303

OFFICE ▶
SURNAME ▶
DATE ▶

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 15
License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 1 (the facility) Facility Operating License No. DPR-77 filed by the Tennessee Valley Authority (licensee), dated August 2, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 15, are hereby incorporated into the license.

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The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective August 3, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

sl

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: August 17, 1982

No legal objection

OFFICE	LA:DL:LB #4	DL:LB #4	OELD	DL:LB #4	AD:DL		
SURNAME	<i>MD</i> MDuncan/hmc	CS CStahle	<i>N. Paul</i> N. Paul	E EAdensam	T TNovak		
DATE	8/5/82	8/5/82	8/9/82	8/11/82	8/11/82		

ATTACHMENT TO LICENSE AMENDMENT NO. 15

FACILITY OPERATING LICENSE NO. DPR-77

DOCKET NO. 50-327

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

<u>Overleaf</u> <u>Page</u>	<u>Amended</u> <u>Page</u>
3/4 3-51	3/4 3-52
3/4 3-54	3/4 3-53

TABLE 3.3-9

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
1. Source Range Nuclear Flux	NOTE 1	1 to 1×10^6 cps	1
2. Reactor Trip Breaker Indication	at trip switchgear	OPEN-CLOSE	1/trip breaker
3. Reactor Coolant Temperature - Hot Leg	NOTE 1	0-650°F	1
4. Pressurizer Pressure	NOTE 1	0-3000 psig	1
5. Pressurizer Level	NOTE 1	0-525 in	1
6. Steam Generator Pressure	NOTE 1	0-1200 psig	1/steam generator
7. Steam Generator Level	NOTE 2 or near Auxiliary F. W. Pump	0-100%	1/steam generator
8. Full Length Control Rod Position Limit Switches	Auxiliary Instrument Room: Racks R41-44	On-off	1 insertion limit switch/rod
9. RHR Flow Rate	NOTE 1	0-4500 gpm	1
10. RHR Temperature	NOTE 1	50-400°F	1
11. Auxiliary Feedwater Flow Rate	NOTE 1	0-880 gpm.	1

TABLE 3.3-9 (Continued)

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
12. Pressurizer Relief Tank Pressure	NOTE 1	0-100 psig	1
13. Containment Pressure	NOTE 1	-1 to +15 psig	1

NOTE 1: Auxiliary Control Room Panel 1-L-10

NOTE 2: Auxiliary Control Room Panels 1-L-11A and 1-L-11B

TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Source Range Nuclear Flux	M	R
2. Reactor Trip Breaker Indication	M	N.A.
3. Reactor Coolant Temperature - Hot Leg	M	R
4. Pressurizer Pressure	M	R
5. Pressurizer Level	M	R
6. Steam Generator Pressure	M	R
7. Steam Generator Level	M	R
8. Full Length Control Rod Position Limit Switches	M	R
9. RHR Flow Rate	M	R
10. RHR Temperature	M	R
11. Auxiliary Feedwater Flow Rate	M	R
12. Pressurizer Relief Tank Pressure	M	R
13. Containment Pressure	M	R

INSTRUMENTATION

CHLORINE DETECTION SYSTEMS

LIMITING CONDITION FOR OPERATION

3.3.3.6 Two independent chlorine detection systems, with their alarm/trip setpoints adjusted to actuate at a chlorine concentration of less than or equal to 5 ppm, shall be OPERABLE.

APPLICABILITY: ALL MODES

ACTION:

- a. With one chlorine detection system inoperable, restore the inoperable detection system to OPERABLE status within 7 days or within the next 6 hours initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.
- b. With both chlorine detection systems inoperable, within 1 hour initiate and maintain operation of the control room emergency ventilation system in the recirculation mode of operation.
- c. The provisions of Specification 3.0.4 are not applicable.
- d. The provisions of Specification 3.0.3 are not applicable in MODE 6.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each chlorine detection system shall be demonstrated OPERABLE by performance of a CHANNEL CHECK at least once per 12 hours, a CHANNEL FUNCTIONAL TEST at least once per 31 days and a CHANNEL CALIBRATION at least once per 18 months.

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 6
License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Sequoyah Nuclear Plant, Unit 2 (the facility) Facility Operating License No. DPR-79 filed by the Tennessee Valley Authority (licensee), dated August 2, 1982, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the license, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is hereby amended by page changes to the Appendix A Technical Specifications as indicated in the attachments to this license amendment and paragraph 2.C.(2) of Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 6, are hereby incorporated into the license.

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DATE ▶

The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment was effective August 3, 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

S/

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

Attachment:
Appendix A Technical
Specification Changes

Date of Issuance: August 17, 1982

*No legal action
p/f*

OFFICE	LA:DL:LB #4	DL:LB #4	OELD	DL:LB #4	AD:DL		
SURNAME	MDuncan/hmc	CStahle	<i>[Signature]</i>	EAdensam	TNovak		
DATE	8/5/82	8/5/82	8/9/82	8/17/82	8/16/82		

ATTACHMENT TO LICENSE AMENDMENT NO. 6

FACILITY OPERATING LICENSE NO. DPR-79

DOCKET NO. 50-328

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change. The corresponding overleaf pages are also provided to maintain document completeness.

Overleaf
Page

Amended
Page

3/4 3-53

3/4 3-54

TABLE 3.3-9 (Continued)

REMOTE SHUTDOWN MONITORING INSTRUMENTATION

<u>INSTRUMENT</u>	<u>READOUT LOCATION</u>	<u>MEASUREMENT RANGE</u>	<u>MINIMUM CHANNELS OPERABLE</u>
12. Pressurizer Relief Tank Pressure	NOTE 1	0-100 psig	1
13. Containment Pressure	NOTE 1	-1 to +15 psig	1

NOTE 1: Auxiliary Control Room Panel 2-L-10

NOTE 2: Auxiliary Control Room Panels 2-L-11A and 2-L-11B

TABLE 4.3-6

REMOTE SHUTDOWN MONITORING INSTRUMENTATION
SURVEILLANCE REQUIREMENTS

<u>INSTRUMENT</u>	<u>CHANNEL CHECK</u>	<u>CHANNEL CALIBRATION</u>
1. Source Range Nuclear Flux	M	R
2. Reactor Trip Breaker Indication	M	N.A.
3. Reactor Coolant Temperature - Hot Leg	M	R
4. Pressurizer Pressure	M	R
5. Pressurizer Level	M	R
6. Steam Generator Pressure	M	R
7. Steam Generator Level	M	R
8. Full Length Control Rod Position Limit Switches	M	R
9. RHR Flow Rate	M	R
10. RHR Temperature	M	R
11. Auxiliary Feedwater Flow Rate	M	R
12. Pressurizer Relief Tank Pressure	M	R
13. Containment Pressure	M	R

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE DPR-77
AND AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE DPR-79
TENNESSEE VALLEY AUTHORITY

INTRODUCTION

On July 30, 1982, TVA informed the NRC that the pressurizer relief tank water level indication in the auxiliary control room for Unit 2 was inoperative. The technical specifications require that an inoperable instrument must be restored within seven days or be in a hot shutdown within the next twelve hours. It was determined that the level transmitter failed and to make repairs or replace the instrument requires entrance into the lower part of the containment which cannot be made with the reactor at power.

EVALUATION

The auxiliary control room is designed for emergency shutdown of the plant from outside the main control room in the unlikely event the main control room requires evacuation. Tables 3.3-6 and 4.3-6 of the technical specifications identify the instrumentation in the auxiliary control room and their surveillance requirements. The staff agrees that item 12, Pressurizer Relief Tank Level, need not be in these tables of instrumentation for shutdown and surveillance requirements. The basis for this conclusion is that in the emergency operating instructions for shutdown outside the control room no operator reliance, action, or decision is based on the pressurizer relief tank level. Also, there are no technical specification requirements for tank level requirements in the main control room. Also Westinghouse Standard Technical Specifications do not identify this instrument as necessary.

Discussions with the applicant indicate that this item was included in the technical specification tables simply because it was one of the instruments in the auxiliary control room. Pressurizer relief tank water level is regarded as a useful piece of information, but not essential to the safe shutdown of the plant. This instrument will be put back in operation at the next extended forced outage of Unit 2. The instrument in Unit 1 is still operational.

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ENVIRONMENTAL CONSIDERATION

We have determined that the amendment does not authorize a change in effluent types or total amounts nor an increase in power level and will not result in any significant environmental impact. Having made this determination, we have further concluded that the amendment involves an action which is insignificant from the standpoint of environmental impact and, pursuant to 10 CFR Section 51.5(d)(4), that an environmental impact statement or negative declaration and environmental impact appraisal need not be prepared in connection with the issuance of this amendment.

CONCLUSION

We have concluded, based on the consideration discussed above, that: (1) because the amendment does not involve a significant increase in the probability or consequences of accidents previously considered, does not create the possibility of an accident of a type different from any evaluated previously, and does not involve a significant decrease in a safety margin, the amendment does not involve a significant hazards consideration, (2) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (3) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 17, 1982

Principal Contributors: Frank Orr, Reactor Systems Branch, DSI
Martin Virgilio, Instrumentation and Control
Systems Branch, DSI
Carl Stahle, Licensing Branch No. 4, DL

OFFICE ▶
SURNAME ▶
DATE ▶

UNITED STATES NUCLEAR REGULATORY COMMISSION

DOCKET NOS. 50-327 AND 50-328

TENNESSEE VALLEY AUTHORITY

NOTICE OF ISSUANCE OF AMENDMENTS

FACILITY OPERATING LICENSE NOS. DPR-77 AND DPR-79

The U. S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 15 to Facility Operating License No. DPR-77 and Amendment No. 6 to Facility Operating License No. DPR-79, issued to Tennessee Valley Authority (licensee) for the Sequoyah Nuclear Plant, Units 1 and 2 (the facilities) located in Hamilton County, Tennessee.

The amendments were authorized by telephone on August 3, 1982, and were confirmed by letter on August 4, 1982. The amendments delete the requirement for pressurizer relief tank level instrumentation. These amendments were issued on an expedited basis to permit Unit 2 to remain at 100 percent power and avoid any potential disruption of Unit 1 operations.

The application for the amendments complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations. The Commission has made appropriate findings as required by the Act and the Commission's regulations in 10 CFR Chapter I, which are set forth in the license amendments. Prior public notice of these amendments was not required since the amendments do not involve a significant hazards consideration.

The Commission has determined that the issuance of these amendments will not result in any significant environmental impact and that pursuant to 10 CFR 51.5(d)(4) environmental impact statements, or negative declarations and environmental impact appraisals need not be prepared in connection with issuance of these amendments.

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OFFICE ▶
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DATE ▶

For further details with respect to this action, see (1) Tennessee Valley Authority letter dated August 2, 1982; (2) Amendment No. 15 to Facility Operating License No. DPR-77 with Appendix A Technical Specification page changes; (3) Amendment No. 6 to Facility Operating License No. DPR-79 with Appendix A Technical Specification page changes; and (4) the Commission's related Safety Evaluation.

All of these items are available for public inspection at the Commission's Public Document Room, 1717 H Street, N.W., Washington, D. C., and the Chattanooga Hamilton County Bicentennial Library, 1001 Broad Street, Chattanooga, Tennessee 37402. A copy of Amendment No. 15 and Amendment No. 6 may be obtained upon request addressed to the U. S. Nuclear Regulatory Commission, Washington, D. C. 20555, Attention: Director, Division of Licensing.

Dated at Bethesda, Maryland, this 17th day of August 1982.

FOR THE NUCLEAR REGULATORY COMMISSION

S/

Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing

No legal signature

OFFICE	LA:DL:LB #4	DL:LB #4	OELD	DL:LB #4			
SURNAME	MDuncan/hmc	CStahle	<i>Rt</i>	EAdensam			
DATE	8/5/82	8/6/82	8/9/82	8/7/82			

Docket Nos: 50-327
and 50-328

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Docket Nos. 50-327/328
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E. Adensam
C. Stahle
M. Duncan
D. Eisenhut
OELD

MEMORANDUM FOR: Thomas M. Novak, Assistant Director
for Licensing
Division of Licensing, NRR

THRU: Elinor G. Adensam, Chief
Licensing Branch No. 4
Division of Licensing, NRR

FROM: Carl Stahle, Project Manager
Licensing Branch No. 4
Division of Licensing, NRR

SUBJECT: ISSUANCE OF AMENDMENT NO. 15 TO FACILITY OPERATING
LICENSE DPR-77 AND AMENDMENT NO. 6 TO FACILITY
OPERATING LICENSE DPR-79, SEQUOYAH NUCLEAR PLANT,
UNITS 1 AND 2

Regarding the issuance of subject amendments, there is no known public
correspondence or irreversible impact associated with this subject.

Carl Stahle, Project Manager
Licensing Branch No. 4
Division of Licensing
Office of Nuclear Reactor Regulation

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PDR ADDCK 05000327
P PDR

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SURNAME	MDuncan/hc	CStahle	EAdensam				
DATE	8/5/82	8/5/82	8/17/82				

AMENDMENT NO. 15 TO FACILITY OPERATING LICENSE DPR-77 - SEQUOYAH UNIT 1
AMENDMENT NO. 6 TO FACILITY OPERATING LICENSE DPR-79 - SEQUOYAH UNIT 2

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