

December 19, 1995

Mr. M. S. Tuckman
Senior Vice President
Nuclear Generation
Duke Power Company
P.O. Box 1006
Charlotte, North Carolina 28201

Distribution

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SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M91353 AND M91354)

Dear Mr. Tuckman:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 161 to Facility Operating License NPF-9 and Amendment No. 143 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated January 12, 1995, as supplemented by letter dated June 29, 1995. The amendments revise and clarify portions of TS Section 6.0, "Administrative Controls."

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

Enclosures:

- 1. Amendment No. 161 to NPF-9
- 2. Amendment No. 143 to NPF-17
- 3. Safety Evaluation

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

December 19, 1995

Mr. M. S. Tuckman
Senior Vice President
Nuclear Generation
Duke Power Company
P.O. Box 1006
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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Victor Nerses".

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-370

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1. Amendment No. 161 to NPF-9
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cc w/encl: See next page

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Duke Power Company

McGuire Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 161
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated January 12, 1995, as supplemented by letter dated June 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 161, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Leonard A. Wiens, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: December 19, 1995



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 143
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated January 12, 1995, as supplemented by letter dated June 29, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 143, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Leonard A. Wiens, Acting Director
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: December 19, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 161

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 143

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages

6-8
6-9
6-10
6-11
6-14
6-16
6-23

Insert Pages

6-8
6-9
6-10
6-11
6-14
6-16
6-23

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

Programs shall be established for the preparation, review, approval, and retention of documents required by the activities described in Specifications 6.5.1.1 through 6.5.1.11. Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls.

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a knowledgeable individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/organization other than the individual/organization which prepared the procedure, or changes thereto. Procedures, or changes thereto, shall be approved in accordance with Specifications 6.8.2 and 6.8.3.

6.5.1.2 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a knowledgeable individual/organization. Each such modification shall be reviewed by an individual/organization other than the individual/organization which designed the modification.

6.5.1.3 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1 and 6.5.1.2 shall be members of the supervisory staff assigned to the site, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.

6.5.1.4 Proposed changes to the Appendix A Technical Specifications shall be prepared by a knowledgeable individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/organization other than the individual/organization which prepared the proposed change.

6.5.1.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be prepared and approved in a manner identical to that of Specification 6.5.1.1. These proposed tests and experiments shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the proposed tests and experiments.

ADMINISTRATIVE CONTROLS

ACTIVITIES (Continued)

6.5.1.6 ALL REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. Such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report.

6.5.1.7 Special reviews and investigations, and the preparation of reports thereon, shall be performed by a knowledgeable individual/organization.

6.5.1.8 A knowledgeable individual/organization shall review every unplanned onsite release of radioactive material to the environs and prepare reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence.

6.5.1.9 A knowledgeable individual/organization shall review changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.

6.5.1.10 A knowledgeable individual/organization shall review the Fire Protection Program and implementing procedures.

6.5.1.11 Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.10 shall be maintained.

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

ORGANIZATION

6.5.2.2 The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification

6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the McGuire Nuclear Site staff.

6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least twice per year thereafter.

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of McGuire Nuclear Station.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.8 The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- e. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- f. All REPORTABLE EVENTS;
- g. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- h. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- i. Reports of activities performed under the provisions of Specifications 6.5.1.1 through 6.5.1.10.

AUDITS

6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRB and the Site Vice President, within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The applicable procedures required to implement the requirements of NUREG-0737;
- c. Deleted
- d. Deleted
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation; and
- g. Quality Assurance Program for effluent and environmental monitoring.
- h. Technical Review and Control Program implementation. |
- i. Fire Protection Program implementation. |
- j. Plant Operations Review Committee implementation. |
- k. Commitments contained in FSAR Chapter 16.0 |

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- 4) Procedures for the recording and management of data,
- 5) Procedures defining corrective actions for all off-control point chemistry conditions, and
- 6) A procedure identifying: (a) the authority responsible for the interpretation of the data, and (b) the sequence and timing of administrative events required to initiate corrective action.

d. Backup Method for Determining Subcooling Margin

A program which will ensure the capability to accurately monitor the Reactor Coolant System subcooling margin. This program shall include the following:

- 1) Training of personnel, and
- 2) Procedures for monitoring.

e. Post-accident Sampling

A program which will ensure the capability to obtain and analyze reactor coolant, radioactive iodines, and particulates in plant gaseous effluents, and containment atmosphere samples under accident conditions. The program shall include the following:

- 1) Training of personnel,
- 2) Procedures for sampling and analysis, and
- 3) Provisions for maintenance of sampling and analysis equipment.

f. Radioactive Effluent Controls Program

A program shall be provided conforming with 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to MEMBERS OF THE PUBLIC from radioactive effluents as low as reasonably achievable. The program (1) shall be contained in FSAR Chapter 16, (2) shall be implemented by operating procedures, and (3) shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- 1) Limitations on the operability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the Offsite Dose Calculation Manual (ODCM),
- 2) Limitations on the concentrations of radioactive material released in liquid effluents to UNRESTRICTED AREAS conforming to 10 times 10 CFR Part 20.1001-20.2401, Appendix B, Table 2, Column 2,

ADMINISTRATIVE CONTROLS

RECORD RETENTION (Continued)

- g. Records of training and qualification for current members of the unit staff;
- h. Records of inservice inspections performed pursuant to these Technical Specifications;
- i. Records of reviews performed for changes made to procedures or equipment or reviews of tests and experiments pursuant to 10 CFR 50.59;
- j. Records of meetings of the NSRB and reports required by Specification 6.5.1.11;
- k. Records of the service lives of all snubbers including the date at which the service life commences and associated installation and maintenance records;
- l. Records of secondary water sampling and water quality; and
- m. Records of analyses required by the Radiological Environmental Monitoring Program that would permit evaluation of the accuracy of the analysis at a later date. This should include procedures effective at specified times and QA records showing that these procedures were followed.
- n. Records of reviews performed for changes made to the ODCM and the PCP.

6.10.3 Records of quality assurance activities required by the QA Manual shall be retained for a period of time required by ANSI N45.2.9-1974.

6.11 RADIATION PROTECTION PROGRAM

Procedures for personnel radiation protection shall be prepared consistent with the requirements of 10 CFR Part 20 and shall be approved, maintained and adhered to for all operations involving personnel radiation exposure.

6.12 HIGH RADIATION AREA

6.12.1 In lieu of the "control device" or "alarm signal" required by paragraph 20.203(c)(2) of 10 CFR 20, each high radiation area, as defined in 10 CFR Part 20, in which the intensity of radiation is equal to or less than 1000 mrem/hr at 45 CM (18 in.) from the radiation source or from any surface which the radiation penetrates shall be barricaded and conspicuously posted as a high radiation area and entrance thereto shall be controlled by requiring issuance of a Radiation Work Permit (RWP). Individuals qualified in radiation protection procedures (e.g., Radiation Protection Technician) or personnel continuously escorted by such individuals may be exempt from the RWP issuance requirement during the performance of their assigned duties in high radiation areas with exposure rates equal to or less than 1000 mrem/hr provided they are otherwise following plant radiation protection procedures for entry into high radiation areas.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 161 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 143 TO FACILITY OPERATING LICENSE NPF-17
DUKE POWER COMPANY
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated January 12, 1995, as supplemented by letter dated June 29, 1995, Duke Power Company (the licensee) submitted a request for changes to the McGuire Nuclear Station, Units 1 and 2, Technical Specifications (TS). The requested changes would revise and clarify portions of TS Section 6.0, "Administrative Controls." The June 29, 1995, letter provided clarifying information that did not change the scope of the January 12, 1995, application and the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

Some of the licensee's requested changes would relocate the requirements for the review functions from the Administrative Controls section of the TS to the respective licensee-controlled documents.

Section 182a of the Atomic Energy Act (the "Act") requires applicants for nuclear power plant operating licenses to include TS as part of the license. The Commission's regulatory requirements related to the content of TS are set forth in 10 CFR 50.36. That regulation requires that the TS include items in five specific categories, including (1) safety limits, limiting safety system settings and limiting control settings, (2) limiting conditions for operation, (3) surveillance requirements, (4) design features, and (5) administrative controls. However, the regulation does not specify the particular requirements to be included in a plant's TS.

The Commission has provided guidance for the contents of TS in its "Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors" ("Final Policy Statement"), 58 FR 39132 (July 22, 1993), in which the Commission indicated that compliance with the Final Policy Statement satisfies §182a of the Act. In particular, the Commission indicated that certain items could be relocated from the TS to licensee-controlled documents, consistent with the standard enunciated in *Portland General Electric Co.* (Trojan Nuclear Plant), ALAB-531, 9 NRC 263, 273 (1979). In that case, the Atomic Safety and Licensing Appeal Board indicated that "technical specifications are to be reserved for those matters as to which the imposition

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of rigid conditions or limitations upon reactor operation is deemed necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety."

Consistent with this approach, the Final Policy Statement identified four criteria to be used in determining whether particular limiting conditions for operation are required to be included in the TS, as follows: (1) installed instrumentation that is used to detect, and indicate in the control room, a significant abnormal degradation of the reactor coolant pressure boundary; (2) a process variable, design feature, or operating restriction that is an initial condition of a design basis accident or transient analysis that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (3) a structure, system, or component that is part of the primary success path and which functions or actuates to mitigate a design basis accident or transient that either assumes the failure of or presents a challenge to the integrity of a fission product barrier; (4) a structure, system, or component which operating experience or probabilistic safety assessment has shown to be significant to public health and safety. The Commission recently adopted amendments to 10 CFR 50.36, pursuant to which the rule was revised to codify and incorporate these criteria. See Final Rule, "Technical Specifications," 60 FR 36953 (July 19, 1995). The Commission indicated that the intent of these criteria can be utilized to identify the optimum set of administrative controls in the TS.

The Commission's policy statement provides that many of the existing TS limiting conditions for operation which do not satisfy these four specified criteria may be relocated to the Updated Safety Analysis Report (USAR), such that future changes could be made to these provisions pursuant to 10 CFR 50.59. Other requirements may be relocated to more appropriate documents (e.g. Security Plan, Quality Assurance (QA) plan, and Emergency Plan) and controlled by the applicable regulatory requirement. While the content of the TS administrative controls is specified in 10 CFR 50.36(c)(5), particular details of the administrative controls may be relocated to licensee-controlled documents where §50.59 or comparable regulatory controls exist.

Administrative controls in existing TS related to the review functions should be relocated to a licensee-controlled document that provides adequate control over changes to these provisions and which provides an appropriate change control mechanism. As such, these review provisions should be relocated to the Quality Assurance Program described or referenced in the facility's USAR and controlled pursuant to 10 CFR 50.54.

The licensee proposed to relocate some TS administrative control provisions to the Duke Power Company Quality Assurance Program Topical Report, Amendment 20. These particular TS provisions are not necessary to assure safe operation of the facility, given that the requirements in the QA program implement the Commission's regulations pertaining to these aspects as set forth below. As documented in the Final Policy Statement, these aspects constitute requirements that can be relocated to the Quality Assurance plan and controlled by the applicable regulatory requirement. Such an approach would

result in an equivalent level of regulatory authority while providing for a more appropriate change control process.

On this basis, the staff concludes that the relocated provisions are not required to be in the TS under 10 CFR 50.36 or §182a of the Act, and are not required to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety. In addition, the staff finds that sufficient regulatory controls exist under 10 CFR 50.54 to adequately control future modifications to these provisions. Accordingly, the staff has concluded that these requirements may be relocated from the TS to the respective licensee-controlled documents.

A. McGuire

1. Section 6.5.1 - Technical Review and Control

With respect to the review and approval of subjects within Section 6.5.1, Duke proposes to delete the titles of individuals where specified. The titles are to be replaced by a general statement that "Approvals shall be by the head of the appropriate site organization, the head of the appropriate station organization, the head of the appropriate site engineering organization, the head of the environmental organization, or an alternate as specified in other applicable regulatory documents or administrative controls." The Duke Power Quality Assurance Program Topical Report, Amendment 20 (QAP) provides amplifying details on the station organizational structure. QAP section 17.3.2.2 defines the final approval authority for station modifications. Section 17.3.2.14 of the QAP defines the approval authority for changes to station Technical Specifications and for station procedures. Future changes to the QAP are under the control of 10 CFR 50.54(a). Duke stated that personnel performing the reviews will continue to meet the same qualification requirements of ANSI N18.1-1971. The staff finds the use of generic titles in the TS acceptable as the QAP contains sufficient controls to ensure that the reviews are performed by equally qualified personnel.

Duke Power also proposed to replace the description "qualified individual" with "knowledgeable individual" and the word "group" with "organization." The changes were proposed by Duke to eliminate confusion with the Duke Qualified Reviewer Program. These changes are acceptable as they are considered an editorial change.

2. Section 6.5.1 - Technical Review and Control

Duke Power proposes to delete a qualifying phrase with respect to the review of items by individuals/organization that states "but who may be from the same organization as the individual/group which prepared the ..." In addition, subsections 6.5.1.2, 6.5.1.3, and 6.5.1.4 have been renumbered. Section 6.5.1.4 has also been revised to state the reviews will be performed by "supervisory staff assigned to the site" rather than "site supervisory staff."

The staff finds these changes acceptable as the deletion of the phrases does not reduce the licensee's commitment and the renumbered sections are an editorial change. The requirement that the staff performing the reviews are assigned to the site is deemed an editorial change as the staff will have the same technical reporting relationship as the present arrangement; therefore, this change is also acceptable.

3. Section 6.5.1.6 - Reportable Events

Duke Power has deleted the requirement that the Manager, Safety Assurance shall assure that reports of Reportable Events are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board. This is replaced with a statement that "such reports shall be reviewed by a knowledgeable individual/organization other than the individual/organization which prepared the report."

The staff finds this change acceptable as we do not require that this subject be included in the TS.

4. Sections 6.5.1.8 and 6.5.1.9 (Previously deleted)

Duke proposes to renumber subsections 6.5.1.10 through 6.5.1.13 as 6.5.1.8 through 6.5.1.11.

The staff finds these proposed renumbering changes acceptable as the text of subsections 6.5.1.8 and 6.5.1.9 had been previously deleted and the renumbering is editorial in nature.

5. Sections 6.5.10 - 6.5.12

Duke Power proposes to revise the statement in each of these sections to state that "The Manager, Safety Assurance shall assure the performance..." with a statement "A knowledgeable individual/organization shall review..." and delete reference to who receives reports and recommendations.

The staff finds these requested changes acceptable as the staff does not require these review activities as part of the TS Technical Review and Control Program.

6. Section 6.5.2.6 - Nuclear Safety Review Board (NSRB)

Duke proposes to revise the NSRB meeting frequency from at least once per 6 months to twice per year.

The staff finds this change acceptable as it is consistent with that described in Section 4.3 of ANSI N18.7-1976, "Administrative Controls for the Operational Phase of Nuclear Power Plants" that has been endorsed by Regulatory Guide 1.33.

7. Section 6.5.2.8 - NSRB

Duke Power proposes to delete the requirement that the NSRB shall review proposed changes to TS or the Operating License. This requirement will remain in the Final Safety Analysis Report (FSAR) Chapter 16 Selected Licensee Commitments (SLC) 16.13-2 and will be subject to the requirements of 50.54(a). Duke added a statement in the QAP, Revision 20, Section 17.3.3.2.1 that the NSRB "reviews proposed changes to the stations' technical specifications and operating licenses."

The staff finds this change acceptable as the requirement for the NSRB to review proposed changes to the TS and Operating License will be relocated to the QAP. Future changes to the QAP will be controlled in accordance with 10 CFR 50.54(a).

8. Section 6.8.1 - Procedures and Programs

Duke has added requirements to this section that procedures shall be established, implemented, and maintained for the implementation of the Technical Review and Control Program and the Plant Operations Review Committee.

The staff finds this proposed addition acceptable.

9. Miscellaneous

Duke has made editorial changes to Sections 6.5.2.7, 6.5.2.8, 6.8.3.f, and 6.10.2.j.

The staff finds these changes acceptable as they are editorial in nature.

3.0 STAFF CONCLUSION

The staff finds the requested changes acceptable as noted above as they meet the appropriate acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan, and conform to the Improved Standard Technical Specifications (ISTS).

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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Date: December 19, 1995