July 18, 1994

Docket Nos. 50-369
and 50-370

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

Distribution
Docket File
NRC/Local PDRs
PDII-3 Reading
S.Varga
H.Berkow
V.Nerses
D.Hagan T-4 A43
G.Hill(4) T-5 C3
C.Grimes O-11 F23
ACRS(10) P-135
PA O-17 F2
OC/LFDCB T-9 E10
E.Merschoff,RII
OGC O-15 B18
L.Berry

Dear Mr. McMeekin:

SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. M89020 AND M89021)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 144 to Facility Operating License NPF-9 and Amendment No. 126 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated February 25, 1994.

The amendments revise TS Tables 3.3-10 and 4.3-7 to add four instruments as part of the accident monitoring instrumentation, and delete five instruments from the TS Tables that are not part of the accident monitoring instrumentation.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

R. MARTIN for/

Victor Nerses, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 144 to NPF-9
2. Amendment No. 126 to NPF-17
3. Safety Evaluation

cc w/enclosures:
See next page

OFF	LA:PD23:DRPE	PM:PD23:DRPE	OGC	D:PD23:DRPE
NAME	LBerry	VNerses:dt	RBachmann	HBerkow
DATE	7/12/94	7/13/94	7/14/94	7/18/94

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P PDR

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Sincerely,

R. MARTIN for/

Victor Nerses, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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FILE NAME:G:\MCGUIRE\MCG89020.AMD

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

July 18, 1994

Docket Nos. 50-369
and 50-370

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

Dear Mr. McMeekin:

SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. M89020 AND M89021)

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Sincerely,

Robert Martin

Victor Nerses, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:
1. Amendment No. 144 to NPF-9
2. Amendment No. 126 to NPF-17
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. T. C. McMeekin
Duke Power Company

McGuire Nuclear Station

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 144
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated February 25, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 144 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: July 18, 1994

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 126
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated February 25, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 126 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: July 18, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 144

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 126

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages
3/4 3-55	3/4 3-55
3/4 3-56	3/4 3-56
3/4 3-57	3/4 3-57

INSTRUMENTATION

ACCIDENT MONITORING INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The accident monitoring instrumentation channels shown in Table 3.3-10 shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

a. With the number of OPERABLE accident monitoring instrumentation channels less than the Required Number of Channels shown in Table 3.3-10, restore the inoperable channel(s) to OPERABLE status within 7 days, or be in at least HOT SHUTDOWN within the next 12 hours.

b. With the number of OPERABLE accident monitoring instrumentation channels less than the Minimum Channels OPERABLE requirements of Table 3.3-10, restore the inoperable channel(s) to OPERABLE status according to b.1 or b.2 below:

   b.1 Instruments 1-15: within 48 hours or be in at least HOT SHUTDOWN within the next 12 hours.

   b.2 Instruments 16 and 17: according to Technical Specification 3.7.4.a.

c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each accident monitoring instrumentation channel shall be demonstrated OPERABLE by performance of the CHANNEL CHECK and CHANNEL CALIBRATION operations at the frequencies shown in Table 4.3-7.

TABLE 3.3-10

ACCIDENT MONITORING INSTRUMENTATION

INSTRUMENT	REQUIRED NO. OF CHANNELS	MINIMUM CHANNEL OPERABLE
1. Containment Pressure	2	1
2. Reactor Coolant Temperature - T_{HOT} and T_{COLD} (Wide Range)	$2/T_{HOT}$ $2/T_{COLD}$	$1/T_{HOT}$ $1/T_{COLD}$
3. Reactor Coolant Pressure - Wide Range	2	1
4. Pressurizer Water Level	2	1
5. Steam Line Pressure	2/steam generator	1/steam generator
6. Steam Generator Water Level - Narrow Range	2/steam generator	1/steam generator
7. Refueling Water Storage Tank Water Level	2	1
8. Auxiliary Feedater Flow Rate	2/steam generator	1/steam generator
9. Reactor Coolant System Subcooling Margin Monitor	2	1
10. Containment Water Level (Wide Range)	2	1
11. In Core Thermocouples	4/core quadrant	2/core quadrant
12. Containment Atmosphere - High Range Monitor (EMF-51a or 51b)	1	
13. Reactor Vessel Level Instrumentation		
a. Dynamic Head (D/P) Range	2	1
b. Lower Range	2	1
14. Neutron Flux - Wide Range	2	1
15. Containment Hydrogen Concentration	2	1
16. Diesel Generator Cooling Water Heat Exchanger RN Flow*	1/diesel generator	1/diesel generator
17. Containment Spray Heat Exchanger RN Flow*	1/train	1/train

*Not applicable if the associated outlet valve is set to its flow balance position with power removed or if the associated outlet valve's flow balance position is fully open.

TABLE 4.3-7

ACCIDENT MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

INSTRUMENT	CHANNEL CHECK	CHANNEL CALIBRATION
1. Containment Pressure	M	R
2. Reactor Coolant Temperature - T_{HOT} and T_{COLD} (Wide Range)	M	R
3. Reactor Coolant Pressure - Wide Range	M	R
4. Pressurizer Water Level	M	R
5. Steam Line Pressure	M	R
6. Steam Generator Water Level - Narrow Range	M	R
7. Refueling Water Storage Tank Water Level	M	R
8. Auxiliary Feedwater Flow Rate	M	R
9. Reactor Coolant System Subcooling Margin Monitor	M	R
10. Containment Water Level (Wide Range)	M	R
11. In Core Thermocouples	M	R
12. Containment Atmosphere - High Range Monitor (EMF-51a or 51b)	M	R
13. Reactor Vessel Level Instrumentation		
a. Dynamic Head (D/P) Range	M	R
b. Lower Range	M	R
14. Neutron Flux - Wide Range	M	R
15. Containment Hydrogen Concentration	M	R
16. Diesel Generator Cooling Water Heat Exchanger RN Flow	M	R
17. Containment Spry Heat Exchanger RN Flow	M	R

Amendment No. 144 (Unit 1)
Amendment No. 126 (Unit 2)

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 144 TO FACILITY OPERATING LICENSE NPF-9

AND AMENDMENT NO. 126 TO FACILITY OPERATING LICENSE NPF-17

DUKE POWER COMPANY

MCGUIRE NUCLEAR STATION, UNITS 1 AND 2

DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

On March 17, 1988, the NRC staff issued a Revised Safety Evaluation Report (RSER) regarding Duke Power Company's (the licensee) conformance to Regulatory Guide (R.G.) 1.97, Revision 2 for McGuire Nuclear Station, Units 1 and 2. The staff accepted the licensee's deviation from the guidance of R.G. 1.97.

In a submittal dated February 25, 1994, the licensee documented additional deviations from the guidance in R.G. 1.97 and proposed related changes to the McGuire Nuclear Station, Units 1 and 2, Technical Specifications (TS) regarding post-accident monitoring (PAM) instrumentation in TS 3.3.3.6, and TS Tables 3.3-10 and 4.3-7.

2.0 EVALUATION

The NRC staff has reviewed the licensee's February 25, 1994, submittal which identified deviations and TS changes with regard to the instrumentation that monitors 1) neutron flux, 2) containment hydrogen concentration, 3) diesel generator cooling water heat exchanger nuclear service water flow, 4) containment spray heat exchanger nuclear service water flow, 5) power operated relief valve (PORV) position, 6) PORV block valve position, 7) safety relief valve position, 8) unit vent high-high range noble gas concentration, 9) releases from steam generator safety relief valves or atmospheric dump valves, 10) high range containment atmosphere radiation, 11) auxiliary feedwater flow, and 12) containment isolation valve position. Our evaluation of the above items is as follows:

1) The licensee's neutron flux monitoring instrumentation is Category 1, but had not been previously included in the TS. The licensee has added this instrumentation to TS Tables 3.3-10 and 4.3-7. This addition to TS Tables 3.3-10 and 4.3-7 is acceptable.

2) The licensee's containment hydrogen concentration monitoring instrumentation is Category 1, but had not been previously included in the TS. The licensee has added this instrumentation to TS Tables 3.3-10 and 4.3-7. This addition to TS Tables 3.3-10 and 4.3-7 is acceptable.

3) The licensee has determined that diesel generator cooling water heat exchanger nuclear service water flow monitoring is required for performing manual actions for which no automatic actions are provided and the licensee has added this instrumentation to TS Tables 3.3-10 and 4.3-7.

   The diesel generator cooling water heat exchanger nuclear service water flow monitoring instrumentation is part of the nuclear service water system which is governed by TS 3.7.4 and the Limiting Condition for Operation (LCO) 3.7.4a that allows 72 hours for restoring one inoperable nuclear service water channel to operable status before the affected unit is taken out of service. The current diesel generator cooling water LCO 3.3.3.6b allows 48 hours for restoring one inoperable flow indicator to operable status. The licensee revised LCO 3.3.3.6b to be consistent with the requirements of LCO 3.7.4. This revision is acceptable.

4) The licensee has determined that containment spray heat exchanger nuclear service water flow monitoring is required for performing manual actions for which no automatic actions are provided and the licensee has added this instrumentation to TS Tables 3.3-10 and 4.3-7. The licensee has also added PAM instrument labels on the control room nuclear service water flow instrumentation.

   The containment spray heat exchanger nuclear service water flow monitoring instrumentation is part of the nuclear service water system which is governed by TS 3.7.4 and the LCO 3.7.4a that allows 72 hours for restoring one inoperable nuclear service water channel to operable status before the affected unit is taken out of service. The current containment spray LCO 3.3.3.6b allows 48 hours for restoring one inoperable flow indicator to operable status. The licensee revised LCO 3.3.3.6b to be consistent with the requirements of LCO 3.7.4. This revision is acceptable.

5) PORV position indication is considered Type D, Category 2 instrumentation and is not required to be incorporated in the PAM TS. Therefore, this instrumentation has been deleted from TS Tables 3.3-10 and 4.3-7. This deletion is acceptable.

6) PORV block valve position indication is considered Type D, Category 2 instrumentation and is not required to be incorporated in the PAM TS. Therefore, this instrumentation has been deleted from TS Tables 3.3-10 and 4.3-7. This deletion is acceptable.

7) Safety relief valve position indication is considered Type D, Category 2 instrumentation and is not required to be incorporated in the PAM TS. Therefore, this instrumentation has been deleted from TS Tables 3.3-10 and 4.3-7. This deletion is acceptable.

8) Unit vent high-high range noble gas concentration monitoring is considered Type E, Category 2 instrumentation and is not required to be incorporated in the PAM TS. Therefore, this instrumentation has been deleted from TS Tables 3.3-10 and 4.3-7. This deletion is acceptable.

9) Monitors of releases from steam generator safety relief valves or atmospheric dump valves (steam relief) are considered Type E, Category 2 instrumentation and are not required to be incorporated in the PAM TS. Therefore, this instrumentation has been deleted from TS Tables 3.3-10 and 4.3-7. This deletion is acceptable.

10) High range containment atmosphere radiation is Category 1. The licensee has made editorial corrections to the instrument numbers that are called out in TS Tables 3.3-10 and 4.3-7. This is an administrative change only and is acceptable.

11) In the staff's RSER, Category 3 condensate storage tank level instrumentation was found acceptable because the safety-related source for auxiliary feedwater is the nuclear service water system. Auxiliary feedwater flow is considered Type D, Category 2 instrumentation. Since auxiliary feedwater flow instrumentation is used to monitor the nuclear service water flow rate, the licensee has proposed that auxiliary feedwater flow remain in TS Tables 3.3-10 and 4.3-7. The licensee also will maintain the PAM labeling on the auxiliary feedwater flow instrumentation in the control room. The staff finds this instrumentation configuration acceptable.

12) R.G. 1.97 recommends that Type A, B, and C instruments designated as Categories 1 and 2 be specifically identified with a common designator on the control panels so that the operator can easily discern that these instruments are intended for use under accident conditions. The licensee has stated that they have provided each Type A, B, and C instrument designated as Category 1 with a PAM label, except for containment isolation valve position instrumentation.

The intent of R.G. 1.97 in identifying instrumentation in the control room is to provide the operator with identification of a minimum set of instrumentation to monitor during a post-accident situation. The licensee has provided operator training that includes information on the purpose of the containment isolation valve position monitoring instrumentation. This training, along with the awareness that the containment isolation valve position monitoring instrumentation should be operational post-accident, meets the intent of R.G. 1.97. Therefore, since the licensee's training provides this assurance of operator knowledge, lack of control room labeling of containment isolation valve position instrumentation is acceptable.

Based on our review of the licensee's submittal, we conclude that the licensee has provided adequate justification for deviations from and exceptions to R.G. 1.97, Revision 2, and changes in TS 3.3.3.6 and Tables 3.3-10 and 4.3-7 for the instrumentation that monitors 1) neutron flux, 2) containment hydrogen concentration, 3) diesel generator cooling water heat exchanger nuclear service water flow, 4) containment spray heat exchanger nuclear service water flow, 5) PORV position, 6) PORV block valve position, 7) safety relief valve position, 8) unit vent high-high range noble gas concentration, 9) releases from steam generator safety relief valves or atmospheric dump valves, 10) high range containment atmosphere radiation, 11) auxiliary feedwater flow, and 12) containment isolation valve position. Therefore, we find the above deviations from R.G. 1.97 and the changes to TS 3.3.3.6, and TS Tables 3.3-10 and 4.3-7 acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 17597 dated April 13, 1994). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: B. Marcus

Date: July 18, 1994