

August 2, 1994

Docket Nos. 50-369
and 50-370

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

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Dear Mr. McMeekin:

SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M81162 AND M81163)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 146 to Facility Operating License NPF-9 and Amendment No. 128 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated May 5, 1994, as supplemented June 13, 1994.

The amendments revise the TS to increase Main Steam and Pressurizer Code Safety Valve Setpoint Tolerances.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

Original signed by:
Robert Martin for:

Victor Nerses, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 146 to NPF-9
2. Amendment No. 128 to NPF-17
3. Safety Evaluation

cc w/enclosures:
See next page

OFF	LA:PD23:DRPE	PM:PD23:DRPE	OGC <i>Wolfe</i>	D:PD23:DRPE
NAME	LBerry <i>LB</i>	VNerses:dt	<i>Uttal</i>	HBerkow <i>HB</i>
DATE	7/13/94	7/13/94	7/20/94	8/12/94

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 2, 1994

Docket Nos. 50-369
and 50-370

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

Dear Mr. McMeekin:

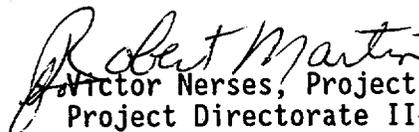
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Sincerely,


Victor Nerses, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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3. Safety Evaluation

cc w/enclosures:
See next page

Mr. T. C. McMeekin
Duke Power Company

McGuire Nuclear Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 146
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility), Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated May 5, 1994, as supplemented June 13, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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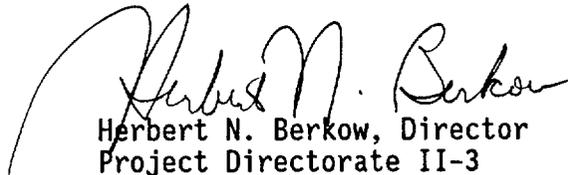
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 146 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: August 2, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 128
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated May 5, 1994, as supplemented June 13, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

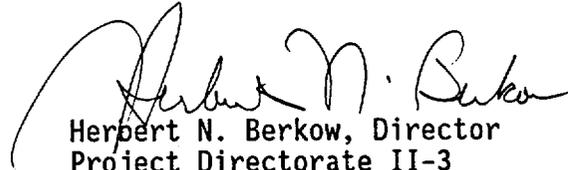
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 128, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: August 2, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 146

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 128

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
3/4 4-7	3/4 4-7
3/4 4-8	3/4 4-8
3/4 7-1	3/4 7-1
3/4 7-3	3/4 7-3
B 3/4 4-2	B 3/4 4-2
B 3/4 7-1	B 3/4 7-1

REACTOR COOLANT SYSTEM

3/4.4.2 SAFETY VALVES

SHUTDOWN

LIMITING CONDITION FOR OPERATION

3.4.2.1 A minimum of one pressurizer Code safety valve shall be OPERABLE with a lift setting of 2485 psig + 3%, - 2%. *

APPLICABILITY: MODES 4 and 5.

ACTION:

With no pressurizer Code safety valve OPERABLE, immediately suspend all operations involving positive reactivity changes and place an OPERABLE RHR loop into operation in the shutdown cooling mode.

SURVEILLANCE REQUIREMENTS

4.4.2.1 No additional requirements other than those required by Specification 4.0.5. Following testing, lift settings shall be within $\pm 1\%$.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

REACTOR COOLANT SYSTEM

OPERATING

LIMITING CONDITION FOR OPERATION

3.4.2.2 All pressurizer Code safety valves shall be OPERABLE with a lift setting of 2485 psig + 3%, - 2%. *

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

With one pressurizer Code safety valve inoperable, either restore the inoperable valve to OPERABLE status within 15 minutes or be in at least HOT STANDBY within 6 hours and in at least HOT SHUTDOWN within the following 6 hours.

SURVEILLANCE REQUIREMENTS

4.4.2.2 No additional requirements other than those required by Specification 4.0.5. Following testing, lift settings shall be within $\pm 1\%$.

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

PLANT SYSTEMS

3/4.7 PLANT SYSTEMS

3/4.7.1 TURBINE CYCLE

SAFETY VALVES

LIMITING CONDITION FOR OPERATION

3.7.1.1 All main steam line Code safety valves associated with each steam generator shall be OPERABLE with lift settings as specified in Table 3.7-3.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With four reactor coolant loops and associated steam generators in operation and with one or more main steam line code safety valves inoperable, operation in MODES 1, 2, and 3 may proceed provided, that within 4 hours, either the inoperable valve is restored to OPERABLE status or the Power Range Neutron Flux High Trip Setpoint is reduced per Table 3.7-1; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With three reactor coolant loops and associated steam generators in operation and with one or more main steam line code safety valves associated with an operating loop inoperable, operation in MODES 1, 2, and 3 may proceed provided, that within 4 hours, either the inoperable valve is restored to OPERABLE status or the Power Range Neutron Flux High Trip Setpoint is reduced per Table 3.7-2; otherwise, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.7.1.1 No additional requirements other than those required by Specification 4.0.5. Following testing, lift settings shall be within $\pm 1\%$.

TABLE 3.7-3
STEAM LINE SAFETY VALVES PER LOOP

	<u>VALVE NUMBER</u>				<u>LIFT SETTING ($\pm 3\%$)*</u>	<u>ORIFICE SIZE</u>
	<u>Loop A</u>	<u>Loop B</u>	<u>Loop C</u>	<u>Loop D</u>		
1.	SV 20	SV 14	SV 8	SV 2	1170 psig	12.174 in ²
2.	SV 21	SV 15	SV 9	SV 3	1190 psig	12.174 in ²
3.	SV 22	SV 16	SV 10	SV 4	1205 psig	16.00 in ²
4.	SV 23	SV 17	SV 11	SV 5	1220 psig	16.00 in ²
5.	SV 24	SV 18	SV 12	SV 6	1225 psig	16.00 in ²

*The lift setting pressure shall correspond to ambient conditions of the valve at nominal operating temperature and pressure.

McGUIRE - UNITS 1 and 2

3/4 7-3

Amendment No. 146
Amendment No. 128
(Unit 1)
(Unit 2)

REACTOR COOLANT SYSTEM

BASES

3/4.4.2 SAFETY VALVES

The pressurizer Code safety valves operate to prevent the RCS from being pressurized above its Safety Limit of 2735 psig. Each safety valve is designed to relieve 420,000 lbs per hour of saturated steam at the valve Setpoint. The relief capacity of a single safety valve is adequate to relieve any overpressure condition which could occur during shutdown. In the event that no safety valves are OPERABLE, an operating RHR loop, connected to the RCS, provides overpressure relief capability and will prevent RCS overpressurization. In addition, the Overpressure Protection System provides a diverse means of protection against RCS overpressurization at low temperatures.

During operation, all pressurizer Code safety valves must be OPERABLE to prevent the RCS from being pressurized above its Safety Limit of 2735 psig. The combined relief capacity of all of these valves is greater than the maximum surge rate resulting from a complete loss of load assuming no Reactor trip until the first Reactor Trip System Setpoint is reached (i.e., no credit is taken for a direct Reactor trip on the loss of load) and also assuming no operation of the power-operated relief valves or steam dump valves.

Demonstration of the safety valves' lift settings will occur only during shutdown and will be performed in accordance with the provisions of Section XI of the ASME Boiler and Pressure Code. Specifications 3.4.2.1 and 3.4.2.2 allow a + 3% and - 2% setpoint tolerance for OPERABILITY; however, the valves are reset to $\pm 1\%$ during surveillance testing to allow for drift.

3/4.4.3 PRESSURIZER

The limit on the maximum water volume in the pressurizer assures that the parameter is maintained within the normal steady-state envelope of operation assumed in the SAR. The limit is consistent with the initial SAR assumptions. The 12 hour periodic surveillance is sufficient to ensure that the parameter is restored to within its limit following expected transient operation. The maximum water volume also ensures that a steam bubble is formed and thus the RCS is not a hydraulically solid system. The requirement that a minimum number of pressurizer heaters be OPERABLE enhances the capability of the plant to control Reactor Coolant System pressure and establish natural circulation.

3/4.7 PLANT SYSTEMS

BASES

3/4.7.1 TURBINE CYCLE

3/4.7.1.1 SAFETY VALVES

The OPERABILITY of the main steam line Code safety valves ensures that the Secondary Coolant System pressure will be limited to within 110% of its design pressure of 1185 psig during the most severe anticipated system operational transient. The maximum relieving capacity is associated with a Turbine trip from 100% RATED THERMAL POWER coincident with an assumed loss of condenser heat sink (i.e., no steam bypass to the condenser).

The specified valve lift settings and relieving capacities are in accordance with the requirements of Section III of the ASME Boiler and Pressure Code, 1971 Edition. Table 3.7-3 allows a $\pm 3\%$ setpoint tolerance for OPERABILITY; however, the valves are reset to $\pm 1\%$ during surveillance testing to allow for drift. The total relieving capacity for all valves on all of the steam lines is 15.9×10^6 lbs/hr which is 105% of the total secondary steam flow of 15.14×10^6 lbs/hr at 100% RATED THERMAL POWER. A minimum of two OPERABLE safety valves per steam generator ensures that sufficient relieving capacity is available for the allowable THERMAL POWER restriction in Tables 3.7-1 and 3.7-2.

STARTUP and/or POWER OPERATION is allowable with safety valves inoperable within the limitations of the ACTION requirements on the basis of the reduction in Secondary Coolant System steam flow and THERMAL POWER required by the reduced Reactor Trip Settings of the Power Range Neutron Flux channels. The Reactor Trip Setpoint reductions are derived on the following bases:

For four loop operation:

$$SP = \frac{(X) - (Y)(V)}{X} \times (109).$$

For three loop operation:

$$SP = \frac{(X) - (Y)(U)}{X} \times (*).$$

Where:

SP = Reduced Reactor Trip Setpoint in percent of RATED THERMAL POWER,

V = Maximum number of inoperable safety valves per steam line,

U = Maximum number of inoperable safety valves per operating, steam line



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 146 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 128 TO FACILITY OPERATING LICENSE NPF-17
DUKE POWER COMPANY
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-369 AND 50-370

INTRODUCTION

By letter dated dated May 5, 1994, as supplemented June 13, 1994, Duke Power Company (the licensee) submitted a request for changes to the McGuire Nuclear Station, Units 1 and 2 Technical Specifications (TS). The requested changes would increase the pressurizer safety valve (PSV) and main steam safety valve (MSSV) lift tolerance ranges. The licensee made a similar proposal in an application dated June 26, 1991. The staff did not complete its review of this item in the amendments for McGuire Units 1 and 2 numbered 128 and 110 in response to that application. The staff has considered the restated proposal on this issue in the May 5, 1994 submittal, as supplemented on June 13, 1994, to supercede and replace in its entirety the earlier version of the request and has reviewed it accordingly.

The specific changes include TS Sections 3/4.4.2 and 3/4.7.1 to reflect an increased PSV setpoint tolerance of + 3%, - 2% and an increased MSSV setpoint tolerance of $\pm 3\%$. Specifically, the licensee proposes that during normal surveillance, if the valves are found to be within the proposed tolerances, they will be within the bases of the accident analyses; however, the valves will be reset to $\pm 1\%$ to prevent the accumulation of setpoint drift outside the allowable tolerance.

At McGuire 1 and 2, there are a total of three PSVs set at 2485 psig. The four main steam lines have a total of 20 MSSVs (i.e., five per main steam line), each set at increments which range from 1170 psig to 1225 psig. The licensee is proposing to revise the McGuire Units 1 and 2, TS to allow a relaxation in setpoint tolerance for the as-found test condition from $\pm 1\%$ to + 3%, - 2% for the PSVs and from $\pm 1\%$ to $\pm 3\%$ for the MSSVs. The amount by which the PSV setpoint is allowed to drift downward is restricted to 2% of nominal in order to ensure the PSVs do not open before a reactor trip on high pressure. The licensee is proposing to reset the PSVs and MSSVs to $\pm 1\%$ prior to declaring the valves operable if the as-found lift settings exceed $\pm 1\%$. Further, the licensee states that these proposed changes will reduce the amount of work performed in a radiological or otherwise dangerous environment.

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EVALUATION

The licensee is proposing to relax the as-found setpoint tolerances for the PSVs and MSSVs from $\pm 1\%$ to $\pm 3\%$. This relaxation is applicable for PSV and MSSV drift which occurs during the operating cycle. The licensee's submittals include the results of the Unit 1 Cycle 8 reload analysis (which is also applicable to Unit 2.) This reload analysis examined the effects of relaxing the PSV and MSSV setpoint tolerance on the plant safety analysis including the effects on the maximum primary and secondary system pressures. This analysis was reviewed and approved by the NRC staff in a letter to the licensee dated November 27, 1991.

The licensee's reload analysis considered the necessary changes in the analysis parameters for the relaxed PSV and MSSV setpoint tolerance. An increased PSV or MSSV setpoint affects the peak primary or secondary system pressure. The licensee evaluated the limiting events which cause primary and secondary system overpressure conditions including the feedline break, locked rotor, rod ejection, turbine trip, MSIV closure, and uncontrolled bank withdrawal events and determined that the peak system pressures do not exceed 110% of the system design pressures. The minimum allowable PSV pressure is restricted to - 2% of the nominal setpoint to ensure that the PSVs do not open before a reactor trip on high pressure. The licensee has also determined that decreasing the MSSV setpoint to - 3% of the nominal setpoint has the potential to affect the steam generator tube rupture accident; however, the existing licensing basis analysis remains bounding.

The licensee has determined that the proposed TS changes do not result in a significant reduction in the margin of safety. The limiting transient in each accident category has been analyzed to determine the effect of the change in the setpoint tolerances. The licensee determined that the results of these analyses meet all acceptance criteria and demonstrate that the peak pressures in the primary and secondary systems do not exceed 110% of the system design pressures in accordance with the ASME code. Further, in order to prevent the setpoints from drifting outside the $\pm 3\%$ range, the licensee will continue to require both PSV and MSSV setpoint tolerances to be restored to $\pm 1\%$ when the as-found lift setting exceeds $\pm 1\%$ prior to declaring the PSVs and MSSVs operable. This will prevent excessive setpoint drift which would cause the peak system pressures to exceed the allowable limits.

Based on the above evaluation, the staff agrees that the analysis which the licensee has provided demonstrates the acceptability of the proposed TS changes. The proposed increase in the setpoint tolerances of the PSVs and the MSSVs has been shown to be acceptable for meeting the plant design basis. Also, for those occurrences where the as-found setpoints of PSVs or MSSVs are in excess of $\pm 1\%$, resetting to within $\pm 1\%$ of the nominal setpoint will be required prior to declaring them operable. Therefore, these proposed TS changes have no significant safety impact to the operation of McGuire, Units 1 and 2 and are acceptable.

3.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

4.0 ENVIRONMENTAL CONSIDERATION

The amendments change requirements with respect to installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20 and change surveillance requirements. The NRC staff has determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluent that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, and there has been no public comment on such finding (59 FR 32029 dated June 21, 1994). Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

5.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: C. Hammer

Date: August 2, 1994