

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

Dear Mr. McMeekin:

SUBJECT: ISSUANCE OF AMENDMENTS - McGUIRE NUCLEAR STATION, UNITS 1 AND 2
(TAC NOS. M88366 AND M88367)

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 148 to Facility Operating License NPF-9 and Amendment No. 130 to Facility Operating License NPF-17 for the McGuire Nuclear Station, Units 1 and 2. The amendments consist of changes to the Technical Specifications (TS) in response to your application dated November 11, 1993, as supplemented February 23, April 12 and July 29, 1994.

The amendments reflect the consolidation of the Quality Verification Department with the Nuclear Generation Department that realigned the Nuclear Safety Review Board to report to the Senior Nuclear Officer, change a reference from Semi-Annual to Annual, change an organizational unit term from "group" to "division," modify titles of positions designated to approve modifications and clarify the responsibilities of the Safety Assurance Manager.

A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

[Original] signed by Robert E. Martin
Robert E. Martin, Acting Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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P PDR

Enclosures:

- 1. Amendment No. 148 to NPF-9
- 2. Amendment No. 130 to NPF-17
- 3. Safety Evaluation

cc w/enclosures:

See next page
DISTRIBUTION

NRC/Local PDRs	PA17F2
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FILE NAME: G:\MCGUIRE\MCG88366.AMD

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DR



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

September 22, 1994

Docket Nos. 50-369
and 50-370

Mr. T. C. McMeekin
Vice President, McGuire Site
Duke Power Company
12700 Hagers Ferry Road
Huntersville, North Carolina 28078-8985

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A copy of the related Safety Evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Robert E. Martin".

Robert E. Martin, Acting Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 148 to NPF-9
2. Amendment No. 130 to NPF-17
3. Safety Evaluation

cc w/enclosures:
See next page

Mr. T. C. McMeekin
Duke Power Company

McGuire Nuclear Station

cc:

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 148
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-9 filed by the Duke Power Company (licensee) dated November 11, 1993, as supplemented February 23, April 12 and July 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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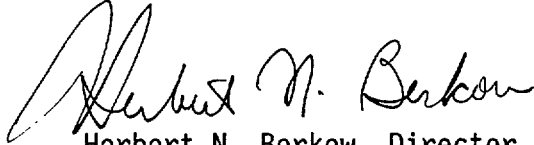
2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 148 , are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance .

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 22, 1994



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility), Facility Operating License No. NPF-17 filed by the Duke Power Company (licensee) dated November 11, 1993, as supplemented February 23, April 12 and July 29, 1994, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 130, are hereby incorporated into this license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Herbert N. Berkow, Director
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Attachment:
Technical Specification
Changes

Date of Issuance: September 22, 1994

ATTACHMENT TO LICENSE AMENDMENT NO. 148

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 130

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

<u>Remove Pages</u>	<u>Insert Pages</u>
6-8	6-8
6-9	6-9
6-10	6-10
6-11	6-11
6-12	6-12
6-13	6-13
6-14	6-14
6-15	6-15
3/4 11-7	3/4 11-7

ADMINISTRATIVE CONTROLS

6.5 REVIEW AND AUDIT

6.5.1 TECHNICAL REVIEW AND CONTROL

ACTIVITIES

6.5.1.1 Each procedure and program required by Specification 6.8 and other procedures which affect nuclear safety, and changes thereto, shall be prepared by a qualified individual/organization. Each such procedure, and changes thereto, shall be reviewed by an individual/group other than the individual/group which prepared the procedure, or changes thereto, but who may be from the same organization as the individual/group which prepared the procedure, or changes thereto.

6.5.1.2 Proposed changes to the Appendix A Technical Specifications shall be prepared by a qualified individual/organization. The preparation of each proposed Technical Specifications change shall be reviewed by an individual/group other than the individual/group which prepared the proposed change, but who may be from the same organization as the individual/group which prepared the proposed change. Proposed changes to the Technical Specifications shall be approved by the Station Manager.

6.5.1.3 Proposed modifications to unit nuclear safety-related structures, systems and components shall be designed by a qualified individual/organization. Each such modification shall be reviewed by an individual/group other than the individual/group which designed the modification, but who may be from the same organization as the individual/group which designed the modification. Proposed modifications to nuclear safety-related structures, systems, and components shall be approved prior to implementation by the Station Manager or the Manager of Engineering; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

6.5.1.4 Individuals responsible for reviews performed in accordance with Specifications 6.5.1.1, 6.5.1.2, and 6.5.1.3 shall be members of the site supervisory staff, previously designated by the Site Vice President to perform such reviews. Each such review shall include a determination of whether or not additional, cross-disciplinary, review is necessary. If deemed necessary, such review shall be performed by the appropriate designated site review personnel.

6.5.1.5 Proposed tests and experiments which affect station nuclear safety and are not addressed in the FSAR or Technical Specifications shall be reviewed by the Station Manager; or for the Station Manager by the Mechanical Maintenance Superintendent, the Operations Superintendent, the I and E/Maintenance Support Superintendent, or the Work Control Superintendent, as previously designated by the Station Manager.

ADMINISTRATIVE CONTROLS

ACTIVITIES (Continued)

6.5.1.6 ALL REPORTABLE EVENTS and all violations of Technical Specifications shall be investigated and a report prepared which evaluates the occurrence and which provides recommendations to prevent recurrence. The Manager, Safety Assurance shall assure such reports are developed and transmitted to the Site Vice President, or designee, who approves such reports, and to the Director of the Nuclear Safety Review Board.

6.5.1.7 The Manager, Safety Assurance shall assure the performance of special reviews and investigations, and the preparation and submittal of reports thereon, as requested by the Site Vice President.

6.5.1.8 Deleted

6.5.1.9 Deleted

6.5.1.10 The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of every unplanned onsite release of radioactive material to the environs including the preparation and forwarding of reports covering evaluation, recommendations, and disposition of the corrective ACTION to prevent recurrence to the Site Vice President, and to the Nuclear Safety Review Board.

6.5.1.11 The Manager, Safety Assurance shall assure the performance of a review by a qualified individual/organization of changes to the PROCESS CONTROL PROGRAM, OFFSITE DOSE CALCULATION MANUAL, and Radwaste Treatment Systems.

6.5.1.12 The Manager, Safety Assurance shall ensure the performance of a review by a qualified individual/organization of the Fire Protection Program and implementing procedures and submittal of recommended changes to the Nuclear Safety Review Board and Manager, Human Resources.

6.5.1.13 Reports documenting each of the activities performed under Specifications 6.5.1.1 through 6.5.1.12 shall be maintained. Copies shall be provided to the Site Vice President, and the Nuclear Safety Review Board.

6.5.2 NUCLEAR SAFETY REVIEW BOARD (NSRB)

FUNCTION

6.5.2.1 The NSRB shall function to provide independent review and audit of designated activities in the areas of:

- a. Nuclear power plant operations,
- b. Nuclear engineering,
- c. Chemistry and radiochemistry,

ADMINISTRATIVE CONTROLS

FUNCTION (Continued)

- d. Metallurgy,
- e. Instrumentation and control,
- f. Radiological safety,
- g. Mechanical and electrical engineering, and
- h. Administrative control and quality assurance practices.

ORGANIZATION

6.5.2.2 The Director, members and alternate members of the NSRB shall be appointed in writing by the Senior Vice President, Nuclear Generation and shall have an academic degree in an engineering or physical science field; and in addition, shall have a minimum of 5 years technical experience, of which a minimum of 3 years shall be in one or more areas given in Specification 6.5.2.1. In special cases, candidates for appointment without an academic degree in engineering or physical science may be qualified with a minimum of ten years experience in one of the areas in Specification 6.5.2.1. No more than two alternates shall participate as voting members in NSRB activities at any one time.

6.5.2.3 The NSRB shall be composed of at least five members, including the Director. Members of the NSRB may be from the Nuclear Generation Department, from other departments within the Company, or from external to the Company. A maximum of one member of the NSRB may be from the McGuire Nuclear Site staff.

6.5.2.4 Consultants shall be utilized as determined by the NSRB Director to provide expert advice to the NSRB.

6.5.2.5 Staff assistance may be provided to the NSRB in order to promote the proper, timely, and expeditious performance of its functions.

6.5.2.6 The NSRB shall meet at least once per calendar quarter during the initial year of unit operation following fuel loading and at least once per 6 months thereafter.

6.5.2.7 The quorum of the NSRB necessary for the performance of the NSRB review and audit functions of these Technical Specifications shall consist of the Director, or his designated alternate, and at least four other NSRB members including alternates. No more than a minority of the quorum shall have line responsibility for operation of McGuire Nuclear Station.

ADMINISTRATIVE CONTROLS

REVIEW

6.5.2.8 The NSRB shall review:

- a. The safety evaluations for: (1) changes to procedures, equipment, or systems, and (2) tests or experiments completed under the provision of Section 50.59, 10 CFR to verify that such actions did not constitute an unreviewed safety question;
- b. Proposed changes to procedures, equipment or systems which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- c. Proposed tests or experiments which involve an unreviewed safety question as defined in Section 50.59, 10 CFR;
- d. Proposed changes in Technical Specifications or this Operating License;
- e. Violations of Codes, regulations, orders, Technical Specifications, license requirements, or of internal procedures or instructions having nuclear safety significance;
- f. Significant operating abnormalities or deviations from normal and expected performance of unit equipment that affect nuclear safety;
- g. All REPORTABLE EVENTS;
- h. All recognized indications of an unanticipated deficiency in some aspect of design or operation of structures, systems or components that could affect nuclear safety;
- i. Quality Assurance Program audits relating to station operations and actions taken in response to these audits; and
- j. Reports of activities performed under the provisions of Specifications 6.5.1.1 through 6.5.1.12.

AUDITS

6.5.2.9 Audits of site activities shall be performed under the cognizance of the NSRB. These audits shall encompass:

- a. The conformance of unit operation to provisions contained within the Technical Specifications and applicable license conditions;
- b. The performance, training, and qualifications of the entire station staff;

ADMINISTRATIVE CONTROLS

AUDITS (Continued)

- c. The results of actions taken to correct deficiencies occurring in unit equipment, structures, systems, or method of operation that affect nuclear safety;
- d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix B, 10 CFR Part 50;
- e. The Emergency Plan and implementing procedures;
- f. The Security Plan and implementing procedures;
- g. The Facility Fire Protection programmatic controls including the implementing procedures;
- h. The fire protection equipment and program implementation utilizing either a qualified offsite licensee fire protection engineer or an outside independent fire protection consultant. An outside independent fire protection consultant shall be used at least every third year;
- i. The Radiological Environmental Monitoring Program and the results thereof;
- j. The OFFSITE DOSE CALCULATION MANUAL and implementing procedures;
- k. The PROCESS CONTROL PROGRAM and implementing procedures for SOLIDIFICATION of radioactive wastes;
- l. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring and;
- m. Any other area of site operation considered appropriate by the NSRB or the Senior Vice President, Nuclear Generation.

AUTHORITY

6.5.2.10 The NSRB shall report to and advise the Senior Vice President, Nuclear Generation on those areas of responsibility specified in Specifications 6.5.2.8 and 6.5.2.9.

ADMINISTRATIVE CONTROLS

RECORDS

6.5.2.11 Records of NSRB activities shall be prepared, approved, and distributed as indicated below:

- a. Minutes of each NSRB meeting shall be prepared, approved, and forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President, within 14 days following each meeting;
- b. Reports of reviews encompassed by Specification 6.5.2.8 above, shall be prepared, approved and forwarded to the Senior Vice President, Nuclear Generation, and to the Site Vice President, within 14 days following completion of the review; and
- c. Audit reports encompassed by Specification 6.5.2.9 above, shall be forwarded to the Senior Vice President, Nuclear Generation and to the Site Vice President, and to the management positions responsible for the areas audited within 30 days after completion of the audit by the auditing organization.

6.6 REPORTABLE EVENT ACTION

6.6.1 The following actions shall be taken for REPORTABLE EVENTS:

- a. The Commission shall be notified and a report submitted pursuant to the requirements of Section 50.73 to 10 CFR Part 50, and
- b. Each REPORTABLE EVENT shall be reviewed by the Station Manager; or for the Station Manager by: (1) the Operations Superintendent, (2) the Maintenance Superintendents, or (3) the Work Control Superintendent, as previously designated by the Station Manager, and the results of the review shall be submitted to the NSRB and the Site Vice President.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

- a. The NRC Operations Center shall be notified by telephone as soon as possible and in all cases within 1 hour. The Site Vice President, and the NSRB shall be notified within 24 hours;
- b. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the Operations Superintendent and the Station Manager. This report shall describe: (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems, or structures, and (3) corrective action taken to prevent recurrence;

ADMINISTRATIVE CONTROLS

SAFETY LIMIT VIOLATION (Continued)

- c. The Safety Limit Violation Report shall be submitted to the Commission, the NSRB and the Site Vice President, within 14 days of the violation; and
- d. Critical operation of the unit shall not be resumed until authorized by the Commission.

6.8 PROCEDURES AND PROGRAMS

6.8.1 Written procedures shall be established, implemented, and maintained covering the activities referenced below:

- a. The applicable procedures recommended in Appendix A of Regulatory Guide 1.33, Revision 2, February 1978;
- b. The applicable procedures required to implement the requirements of NUREG-0737;
- c. Deleted
- d. Deleted
- e. PROCESS CONTROL PROGRAM implementation;
- f. OFFSITE DOSE CALCULATION MANUAL implementation; and
- g. Quality Assurance Program for effluent and environmental monitoring.
- h. Fire Protection Program implementation.
- i. Commitments contained in FSAR Chapter 16.0

6.8.2 Each procedure of Specification 6.8.1 above, and changes thereto, shall be reviewed and approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports prior to implementation and shall be reviewed periodically as set forth in administrative procedures. For procedures which implement offsite environmental, technical, and laboratory activities, the above review and approval may be performed by the General Manager, Environmental Services or designee.

6.8.3 Temporary changes to procedures of Specification 6.8.1 above may be made provided:

- a. The intent of the original procedure is not altered;
- b. The change is approved by two members of the plant management staff, at least one of whom holds a Senior Operator license on the unit affected; and

ADMINISTRATIVE CONTROLS

PROCEDURES AND PROGRAMS (Continued)

- c. The change is approved by an appropriate division manager, superintendent/manager, or one of their designated direct reports within 14 days of implementation.

6.8.4 The following programs shall be established, implemented, and maintained:

- a. Reactor Coolant Sources Outside Containment

A program to reduce leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to as low as practical levels. The systems include RHR, Boron Recycle, Refueling Water, Liquid Waste, Waste Gas, Safety Injection, Chemical and Volume Control, Containment Spray, and Nuclear Sampling. The program shall include the following:

- 1) Preventive maintenance and periodic visual inspection requirements, and
- 2) Integrated leak test requirements for each system at refueling cycle intervals or less.

- b. In-Plant Radiation Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions. This program shall include the following:

- 1) Training of personnel,
- 2) Procedures for monitoring, and
- 3) Provisions for maintenance of sampling and analysis equipment.

- c. Secondary Water Chemistry

A program for monitoring of secondary water chemistry to inhibit steam generator tube degradation. This program shall include:

- 1) Identification of a sampling schedule for the critical variables and control points for these variables,
- 2) Identification of the procedures used to measure the values of the critical variables,
- 3) Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in-leakage,

RADIOACTIVE EFFLUENTS

LIQUID HOLDUP TANKS*

LIMITING CONDITION FOR OPERATION

3.11.1.4 The quantity of radioactive material contained in each temporary unprotected outdoor tank shall be limited to less than or equal to 10 Curies, excluding tritium and dissolved or entrained noble gases.

APPLICABILITY: At all times.

ACTION:

- a. With the quantity of radioactive material in any of the above tanks exceeding the above limit, immediately suspend all additions of radioactive material to the tank, within 48 hours reduce the tank contents to within the limit, and describe the events leading to this condition in the next Annual Radioactive Effluent Release Report.
- b. The provisions of Specification 3.0.3 are not applicable.

SURVEILLANCE REQUIREMENTS

4.11.1.4 The quantity of radioactive material contained in each of the above tanks shall be determined to be within the above limit by analyzing a representative sample of the tank's contents at least once per 7 days when radioactive materials are being added to the tank.

*Tanks included in this specification are those outdoor tanks that are not surrounded by liners, dikes, or walls capable of holding the tank contents and that do not have tank overflows and surrounding area drains connected to the Liquid Radwaste Treatment System.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 148 TO FACILITY OPERATING LICENSE NPF-9
AND AMENDMENT NO. 130 TO FACILITY OPERATING LICENSE NPF-17
DUKE POWER COMPANY
MCGUIRE NUCLEAR STATION, UNITS 1 AND 2
DOCKET NOS. 50-369 AND 50-370

1.0 INTRODUCTION

By letter dated November 11, 1993, as supplemented February 23, April 12 and July 29, 1994, Duke Power Company (the licensee) submitted a request for changes to the McGuire Nuclear Station, Units 1 and 2, Technical Specifications (TS). The amendments reflect the consolidation of the Quality Verification Department with the Nuclear Generation Department that realigned the Nuclear Safety Review Board to report to the Senior Nuclear Officer, change a reference from Semi-Annual to Annual, change an organizational unit term from "group" to "division," modify titles of positions designated to approve modifications and clarify the responsibilities of the Safety Assurance Manager.

The February 23, April 12 and July 29, 1994, letters provided clarifying information that did not change the scope of the November 11, 1993, application and the initial proposed no significant hazards consideration determination.

2.0 EVALUATION

CONSOLIDATION OF DEPARTMENTS

The consolidation of the Quality Verification Department with the Nuclear Generation Department is reflected on the subject TS pages in TS 6.5.2.8.i wherein Quality Verification Department audits are renamed Quality Assurance Program audits. The audit group is maintained as before with the exception that it is now moved into the Safety Assurance Group. The specification of the list of audits in TS 6.5.2.9 and the requirement that the NSRB review these audits are not changed by this amendment. Since the envelope of the audits to be performed and the NSRB review requirement are unchanged, this change is acceptable.

REALIGNMENT OF NSRB REPORTING RELATIONSHIP- TS 6.5.2.2, 6.5.2.9.m & 6.5.2.10

The licensee requested changes in an earlier application and the staff approved TS changes in amendments numbered 132 and 114 that elevated certain TS listed reporting relationships to a licensee management level that is

generally one level higher than the level addressed by NRC staff guidance including the Standard Technical Specification (STS). Included in these changes was the elevation of the NSRB reporting level to the licensee's Executive Vice President. The licensee requests, in the current amendment application, to return this NSRB reporting relationship and associated responsibilities to the Senior Vice President, Nuclear Generation. This change is reflected in TS 6.5.2.2, 6.5.2.9.m and 6.5.2.10. The Senior Vice President is the licensee's Senior Nuclear Officer and the staff concludes that this proposal is consistent with general staff guidance in the Standard Review Plan and the STS and is acceptable.

RECORDS OF NSRB ACTIVITIES - TS 6.5.2.11

Consistent with the change in the NSRB reporting relationship discussed above the Executive Vice President is removed from the list for receipt of NSRB meeting minutes, reports and audit reports and the Site Vice President is added to this list. This change provides comparable management levels as contained in the STS and is acceptable.

TECHNICAL REVIEW AND CONTROL ACTIVITIES - TS 6.5.1.3, 6.5.1.5 and 6.6.1

The licensee initially proposed to modify the authority for approval of modifications by deleting specifically named management positions and substituting a generalized delegation authority by the Station Manager. This was subsequently revised in the submittal of July 29, 1994 to a more restrictive proposal to augment the Station Manager's authority for approving modifications by adding the Manager of Engineering and by revising the position titles in these TS to reflect the titles of these positions in the current licensee organization. The title of Mechanical Superintendent is revised to the Mechanical Maintenance Superintendent and the I and E Superintendent is revised to the I and E/Maintenance Support Superintendent. The staff finds these changes to be acceptable as they provide for approval by the appropriate division manager or superintendent level.

SAFETY ASSURANCE MANAGER RESPONSIBILITIES - TS 6.5.1.6

The Safety Assurance Manager's responsibilities are clarified to those of ensuring the development of TS 6.5.1.6 reports and to provide for a designee who may approve these reports from the prior requirement that the Safety Assurance Manager approve all such reports. This change is acceptable, as the reports are approved and sent to appropriate management levels.

PROCEDURE APPROVAL AND CHANGE AUTHORITY - TS 6.8.2 AND 6.8.3

The licensee proposed to change the approval authority from "group" manager to "appropriate division" manager. This change reflects the retitling of comparable management positions due to the reorganization. Either the former or the current title is consistent with the STS guidance for the appropriate level of management responsibility for this function and is acceptable.

SEMI-ANNUAL TO ANNUAL EFFLUENT REPORT- TS 3.11.1.4

The licensee proposes to change the reference from "Semiannual" to "Annual" Radioactive Effluent Release Report in the Radioactive Effluents section to provide consistency with reference to this report in other TS sections. The staff finds these changes acceptable, as the staff has approved the change from a semi-annual to an annual report in Amendments numbered 132 and 119 for McGuire Units 1 and 2, respectively.

ANNUAL INDEPENDENT FIRE PROTECTION AUDIT

The reference in the licensee's joint application for all three of its nuclear facilities to delete the annual independent fire protection audit applies only to another facility and not to the McGuire Nuclear Station. Therefore, there is no change to the McGuire TS in this regard. This is acceptable.

3.0 OVERALL SUMMARY

The staff finds the licensee's proposed changes acceptable, as they meet the appropriate acceptance criteria of Section 13.4 of NUREG-0800, the Standard Review Plan, and are consistent with the Standard Technical Specifications.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the North Carolina State official was notified of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments change recordkeeping, reporting, or administrative procedures or requirements. Accordingly, the amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendments.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

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